



Nebraska Public Power District
Nebraska's Energy Leader

NLS2001110
December 14, 2001

U.S. Nuclear Regulatory Commission
Attention: Document Control Desk
Washington, D.C. 20555-0001

Gentlemen:

Subject: Correction of Typographical Error in Technical Specifications
Cooper Nuclear Station, NRC Docket 50-298, DPR-46

- References:**
1. Nebraska Public Power District letter (NLS970002) from G. R. Horn to the U.S. Nuclear Regulatory Commission (NRC) Document Control Desk dated March 27, 1997, "Proposed Change to CNS Technical Specifications – Conversion to Improved Standard Technical Specifications."
 2. NRC letter from J. N. Donohew, NRC, to G. R. Horn dated July 31, 1998, "Conversion to Improved Technical Specifications for the Cooper Nuclear Station – Amendment No. 178 to Facility Operating License No. DPR-46 (TAC No. M98317)."

The purpose of this letter is to notify the U.S. Nuclear Regulatory Commission (NRC) of a typographical error in the Cooper Nuclear Station (CNS) Technical Specifications (TS) recently discovered by the Nebraska Public Power District (District) and to request that the NRC issue a corrected page to the CNS TS. The error involves a Surveillance Requirement (SR) referenced in TS Table 3.3.6.1-1. Attachment 1 provides the corrected TS page in final typed page format.

By letter dated March 27, 1997 (Reference 1) the District requested a license amendment to adopt the Standard Technical Specifications (STS), consistent with NUREG-1433, Revision 1 for CNS. The District has verified that the SR referenced in Table 3.3.6.1-1 as submitted by Reference 1 was correct. By Amendment No. 178 (Reference 2) to the CNS operating license the STS were issued for adoption at CNS. Table 3.3.6.1-1 as issued with Reference 2 contained the incorrect SR.

The District has confirmed that the appropriate surveillance has been performed in the past and is scheduled for future performance. Based on the circumstances the District believes that a license amendment is not necessary for resolution of this error.

A001
David
01/23/02

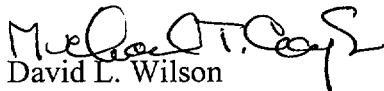
The District has confirmed that the appropriate surveillance has been performed in the past and is scheduled for future performance. Based on the circumstances the District believes that a license amendment is not necessary for resolution of this error.

The typographical error is documented in the CNS corrective action program. The date for correction of the error is February 13, 2002. In order to incorporate the revised page into the controlled copies of CNS TS by that date, the District requests that the NRC issue the corrected page by February 6, 2002. The District considers this review period as reasonable based on the simplicity of this issue and previous discussions with the NRC project manager for CNS.

Copies of this letter are being provided to the NRC Region IV office and the CNS Resident Inspector in accordance with 10 CFR 50.4(b)(1).

Should you have any questions concerning this matter, please contact Mr. Paul Fleming at (402) 825-2774.

Sincerely,

for 
David L. Wilson
Vice President of Nuclear Energy

/rer

Attachment: Corrected Technical Specification Page 3.3-53 - Final Typed Format

cc: Regional Administrator w/ attachment
USNRC – Region IV

Senior Project Manager w/ attachment
USNRC – NRR Project Directorate IV-1

Senior Resident Inspector w/ attachment
USNRC

NPG Distribution w/o attachment

Records w/ attachment

ATTACHMENT

CORRECTED TECHNICAL SPECIFICATION PAGE 3.3-53

FINAL TYPED FORMAT

**COOPER NUCLEAR STATION
FACILITY OPERATING LICENSE NUMBER DPR-46
NEBRASKA PUBLIC POWER DISTRICT**

Primary Containment Isolation Instrumentation
3.3.6.1

Table 3.3.6.1-1 (page 3 of 3)
Primary Containment Isolation Instrumentation

FUNCTION	APPLICABLE MODES OR OTHER SPECIFIED CONDITIONS	REQUIRED CHANNELS PER TRIP SYSTEM	CONDITIONS REFERENCED FROM REQUIRED ACTION C.1	SURVEILLANCE REQUIREMENTS	ALLOWABLE VALUE
6. RHR Shutdown Cooling System Isolation					
a. Reactor Pressure - High	1,2,3	1	F	SR 3.3.6.1.2 SR 3.3.6.1.4 SR 3.3.6.1.6	≤ 72 psig
b. Reactor Vessel Water Level - Low (Level 3)	3,4,5	4 ^(b)	I	SR 3.3.6.1.1 SR 3.3.6.1.2 SR 3.3.6.1.4 SR 3.3.6.1.6	≥ 3 inches

(b) Only one trip system is required in MODES 4 and 5 when RHR Shutdown Cooling System integrity maintained.