

January 28, 2002

Mr. R. T. Ridenoure
Division Manager - Nuclear Operations
Omaha Public Power District
Fort Calhoun Station FC-2-4 Adm.
Post Office Box 550
Fort Calhoun, NE 68023-0550

SUBJECT: FORT CALHOUN STATION, UNIT NO. 1 - CORRECTIONS TO SAFETY
EVALUATION SUPPORTING AMENDMENT NO. 201 (TAC NO. MB1221)

Dear Mr. Ridenoure:

The Commission issued Amendment No. 201 to Facility Operating License No. DPR-40 for the Fort Calhoun Station, Unit No. 1 on December 5, 2001. The amendment consisted of changes to the Technical Specifications in response to your application dated February 7, 2001. Based on conversations with your staff, typographical errors were noted on page 4 of Table 1, "DBA Analysis Assumptions" and page 1 of Table 2, "Analysis Results" of the safety evaluation. Marginal lines indicate where the changes were made. Enclosed are the corrected pages. If you have any questions, please call me at (301) 415-1445.

Sincerely,

/RA/

Alan B. Wang, Project Manager, Section 2
Project Directorate IV
Division of Licensing Project Management
Office of Nuclear Reactor Regulation

Docket No. 50-285

Enclosures: 1. Page 4 of Table 1
2. Page 1 of Table 2

cc w/encls: See next page

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Fort Calhoun Station FC-2-4 Adm.

Post Office Box 550

Fort Calhoun, NE 68023-0550

Iodine species fractions		
Elemental	0.9985	
Organic	0.0015	
Particulates	none	
Water depth, ft		
Case 1	23	
Iodine pool scrubbing factor, effective		
Case 1	200	
Release rate from fuel	puff	
Release duration to environment, hours	2	
Control room HVAC switchover, sec	0	
Containment surface χ/Q used for containment FHA		
Auxiliary building vent stack χ/Q used for fuel pool area FHA		

Assumptions for Heavy Load Drop Analysis

Number of fuel assemblies in core	133
Number of damaged assemblies	133
Radial peaking factor	1.0
Decay period, hours	72
Fuel rod gap fractions	
Iodines	0.05
Noble gases	0.05
Alkali metals	0.05
Iodine species fractions	
Elemental	0.9985
Organic	0.0015
Particulates	none
Containment volume, ft ³	1.05E+6
Containment mixing, percent	50
Water depth, ft	
Case 1	11.15
Case 2	23

TABLE 2
ANALYSIS RESULTS

	<u>EAB</u> <u>rem, TEDE</u>	<u>LPZ</u> <u>rem, TEDE</u>	<u>Control Room</u> <u>rem, TEDE</u>	
Loss of coolant accident (25)	2.5	0.5	4.5	
Fuel handling accident (6.3)				
Pool area	1.5	0.5	0.5	
Containment	1.5	0.5	0.5	
Heavy drop in containment (6.3)				
23 feet water over	3.5	0.5	1.5	
11 feet water over	5.0	0.5	2.0	
Seized rotor accident (2.5)	0.5	0.5	4.7	
Control rod ejection accident (6.3)	2.0	0.5	3.0	
Main steam line break				
Pre-incident spike (25)	0.5	0.5	2.5	
Co-incident spike (2.5)	1.5	0.5	2.5	
Steam generator tube rupture				
Pre-incident spike (25)	1.5	0.5	1.5	
Co-incident spike (2.5)	1.5	0.5	1.5	
Waste gas decay tank failure (0.5)	0.14	0.01	0.04	
Liquid waste tank failure (0.5)	0.08	0.01	0.32	

*With exception of WGDT failure/LWT failure, doses were rounded up to nearest 0.5 rem
() = Dose acceptance criterion for EAB and LPZ
Control room acceptance criterion is <5 rem TEDE