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November 9, 2001

U. S. Nuclear Regulatory Commission
Document Control Desk
Washington, DC 20555

Subject: River Bend Station - Unit 1
Docket No. 50-458
License No. NPF-47
10 CFR 50.46 (a)(3) Report

File Nos. G9.5, G9.25.1

RBG-45865
RBF1-01-0243

Ladies and Gentlemen:

In accordance with 10 CFR 50.46(a)(3), River Bend Station (RBS) is required to report changes to or errors discovered in acceptable ECCS evaluation models. This report will identify: (1) A change to the evaluation model utilized to analyze the Loss-of-Coolant Accident (LOCA) for River Bend Station Cycle 11, and (2) the changes and errors found prior to Cycle 11 for the period from the last report to the present.

As a result of the use of the Framatome ATRIUM™-10 fuel, RBS has adopted the Advanced Nuclear Fuels EXEM BWR ECCS Evaluation Model for LOCA evaluation to determine the cycle specific core operating limits. The implementation of this method meets the requirements set forth in Section 5.6.5 of the RBS Technical Specifications (Amendment No. 112), specifically, that the analytical methods used to determine the core operating limits shall be those previously reviewed and approved by the NRC as listed in Section 5.6.5. The change to the LOCA analysis methodology is reflected in the Cycle 11 River Bend Station Core Operating Limits Report (submitted under separate cover as letter RBG-45841 dated October 15, 2001). Also, the limiting Peak Clad Temperature (PCT) for RBS has changed from 1560 °F to 1875 °F due to the change in ECCS evaluation methodology.

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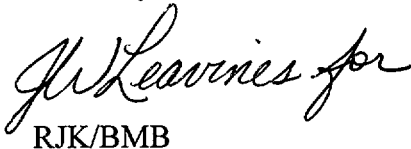
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Moreover, River Bend Station (RBS) is required to report changes to or errors discovered in acceptable ECCS evaluation models on an annual basis. RBS is obligated to report the nature of the change or error, under the GE LOCA methodology which was previously the RBS licensing basis methodology, and its estimated effect on calculated PCT for the period from the last report (submitted under letter RBG-45489 dated September 18, 2001) to the end of Cycle 10. The impact on PCT reported herein is a maximum change of 15 °F, which is due to the subsequent actions being performed by GE following the BWROG audit on the ECCS methodology, and due to an inconsistent core exit steam flow used in SAFER.

Individual changes and errors affecting the ECCS evaluation model have also been reported to the NRC by General Electric (GE). The related GE correspondence, letter to NRC Document Control Desk dated August 10, 2001, discusses these changes and/or errors in detail.

For further information, contact Mr. B. M. Burmeister at (225) 381-4148.

Sincerely,



RJK/BMB

cc: U. S. Nuclear Regulatory Commission
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