

January 17, 2002

Mr. J. V. Parrish
Chief Executive Officer
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P.O. Box 968 (Mail Drop 1023)
Richland, WA 99352-0968

SUBJECT: REQUEST FOR ADDITIONAL INFORMATION (RAI) - COLUMBIA GENERATING
STATION (TAC NO. MB1777)

Dear Mr. Parrish:

By letter dated April 16, 2001, Energy Northwest submitted for NRC staff review, an amendment request to change the facility descriptions as originally evaluated in the Columbia Generating Station's Final Safety Analysis Report. The requested change would allow an unisolable drain line between the reactor core isolation cooling and the control rod drive/condensate pump room. As a result of the review, the NRC staff has determined that additional information is needed to complete the review. The information needed is detailed in the enclosure.

The enclosed request was discussed with Mr. Brownlee of your staff on January 10, 2002. A mutually agreeable target date of February 15, 2002, was established for responding to the RAI. If circumstances result in the need to revise the target date, please call me at your earliest opportunity at (301) 415-1424.

Sincerely,

/RAI/

Jack Cushing, Project Manager, Section 2
Project Directorate IV
Division of Licensing Project Management
Office of Nuclear Reactor Regulation

Docket No. 50-397

Enclosure: Request for Additional Information

cc w/encl: See next page

Columbia Generating Station

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REQUEST FOR ADDITIONAL INFORMATION

UNISOLABLE DRAIN LINE

COLUMBIA GENERATING STATION

DOCKET NO. 50-397

1. Discuss the augmented piping inspection programs, if any, for the service water system (SWS) and the condensate system.
2. Provide an estimate of the amount of SWS piping (in feet) and the number of welds in the control rod drive/condensate (CRD/COND) room. Also, provide an estimate of how much condensate storage and transfer (CST) piping and how many welds are in the CRD/COND room.
3. It is stated on page 3 of the November 8, 2001, letter, that "The analysis assumes if the plant operators cannot terminate the flooding in one hour, then core damage occurs." Based on this statement, it appears that the estimated probability of failure to isolate within one hour of $1E-4$ /demand for the SWS piping flooding dominates the conditional core damage probability (CCDP) for ruptures 1, 1R, 2, and 2R (and doesn't change between scenarios) and that this assumption is not used in the CST piping ruptures 3 and 3R. Is this conclusion correct? Discuss why there is not a one-hour time for the CST piping failure.
4. Discuss how high the water is expected to be in the CRD/COND room one hour after a SWS pipe break and one hour after a CST pipe break. State what equipment in the CRD/COND room modeled in the plant's probable risk assessment (PRA) may be expected to fail within the one hour because of the water from the pipe breaks.
5. Discuss how high the water is expected to reach in the reactor core isolation cooling (RCIC) room (caused by drainage through the unisolable drain line) within one hour after a SWS pipe break and one hour after a CST pipe break. State what equipment in the RCIC room that is modeled in the plant PRA may be expected to fail within the one hour because of the water from the pipe breaks. Discuss how long before equipment, other than RCIC modeled in the plant PRA, would be expected to fail.