



**UNITED STATES  
NUCLEAR REGULATORY COMMISSION  
REGION IV  
611 RYAN PLAZA DRIVE, SUITE 400  
ARLINGTON, TEXAS 76011-8064**

November 30, 2001

S. K. Gambhir, Division Manager  
Nuclear Operations  
Omaha Public Power District  
Fort Calhoun Station FC-2-4 Adm.  
P.O. Box 399  
Hwy. 75 - North of Fort Calhoun  
Fort Calhoun, Nebraska 68023-0399

SUBJECT: FORT CALHOUN STATION - NRC INSPECTION REPORT 50-285/01-03

Dear Mr. Gambhir:

On October 22 through November 2, 2001, the NRC completed an inspection at your Fort Calhoun Station. The enclosed report documents the inspection findings, which were discussed on November 2, 2001, with Mr. Clemens and other members of your staff.

This inspection was an examination of activities conducted under your license as they relate to the identification and resolution of problems, and compliance with the Commission's rules and regulations, and with the conditions of your operating license. Within these areas, the inspection involved selected examination of procedures and representative records, observations of activities, and interviews with personnel.

On the basis of the sample selected for review, there were no findings of significance identified during the inspection. The inspectors concluded that problems were properly identified, evaluated and resolved within the problem identification and resolution program.

In accordance with 10 CFR 2.790 of the NRC's "Rules of Practice," a copy of this letter and its enclosure will be available electronically for public inspection in the NRC Public Document Room or from the Publicly Available Records (PARS) component of NRC's document system (ADAMS). ADAMS is accessible from the NRC Web site at <http://www.nrc.gov/NRC/ADAMS/index.html> (the Public Electronic Reading Room).

Sincerely,

/RA/

Charles S. Marschall, Chief  
Engineering and Maintenance Branch  
Division of Reactor Safety

Docket: 50-285

License: DPR-40

Enclosure:

NRC Inspection Report

50-285/01-03

cc w/enclosure:

Mark T. Frans, Manager

Nuclear Licensing

Omaha Public Power District

Fort Calhoun Station FC-2-4 Adm.

P.O. Box 399

Hwy. 75 - North of Fort Calhoun

Fort Calhoun, Nebraska 68023-0399

James W. Chase, Division Manager

Nuclear Assessments

Fort Calhoun Station

P.O. Box 399

Fort Calhoun, Nebraska 68023

Richard P. Clemens, Manager - Fort Calhoun Station

Omaha Public Power District

Fort Calhoun Station FC-1-1 Plant

P.O. Box 399

Hwy. 75 - North of Fort Calhoun

Fort Calhoun, Nebraska 68023

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Chairman

Washington County Board of Supervisors

Washington County Courthouse

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Sue Semerena, Section Administrator

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Electronic distribution from ADAMS by RIV:

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DRS Director (**ATH**)

Senior Resident Inspector (**WCW**)

Branch Chief, DRP/C (**KMK**)

Senior Project Engineer, DRP/C (**JMK**)

Staff Chief, DRP/TSS (**PHH**)

RITS Coordinator (**NBH**)

Jim Isom, Pilot Plant Program (**JAI**)

**RidsNrrDipmLipb**

Scott Morris (**SAM1**)

NRR Event Tracking System (**IPAS**)

FCS Site Secretary (**NJC**)

Dale Thatcher (**DFT**)

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SRI:EMB	SOE:OB	SRI:DRP/C	C:EMB	C:OB	C:PBC	C:EMB
RPMullikin:Imb	HFBundy	WCWalker	CSMarschall	ATGody	KMKennedy	CSMarschall
/RA/	/RA/ E	/RA/ E	/RA/	/RA/	/RA/	/RA/
11/26/01	11/21/01	11/20/01	11/27/01	11/28/01	11/28/01	11/30/01

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**ENCLOSURE**

U.S. NUCLEAR REGULATORY COMMISSION  
REGION IV

Docket: 50-285

License: DPR-40

Report: 50-285/01-03

Licensee: Omaha Public Power District

Facility: Fort Calhoun Station

Location: Fort Calhoun Station  
FC-2-4 Adm., P.O. Box 399  
Hwy. 75 - North of Fort Calhoun  
Fort Calhoun, Nebraska

Dates: October 22 through November 2, 2001

Inspectors: R. Mullikin, Senior Reactor Inspector, Engineering and Maintenance Branch  
H. Bundy, Senior Operations Engineer, Operations Branch  
W. Walker, Senior Resident Inspector, Projects Branch C

Approved By: Charles S. Marschall, Chief, Engineering and Maintenance Branch

## SUMMARY OF FINDINGS

IR 05000285/01-03 on 10/22/2001 through 11/02/2001, Omaha Public Power District, Fort Calhoun Station, annual baseline inspection of the identification and resolution of problems.

The inspection was conducted by one senior reactor inspector, one senior operations engineer, and one senior resident inspector. No findings of significance were identified.

### **Identification and Resolution of Problems**

The licensee identified, evaluated, prioritized, and corrected problems in a timely manner, consistent with risk and safety significance. Licensee audits and assessments critically assessed the licensee's problem identification and resolution activity. During inspection interviews, workers at the site expressed no reservations to input safety issues into the corrective action program.

## Report Details

### 4. OTHER ACTIVITIES (OA)

#### **4OA2** Problem Identification and Resolution

##### a. Effectiveness of Problem Identification

###### (1) Inspection Scope

The inspectors reviewed items selected across the seven cornerstones of safety to determine if problems were being properly identified, characterized, and entered into the corrective action program for evaluation and resolution. Specifically, the inspectors' review included a selection of 172 condition reports (CRs) and 5 work orders that had been opened since May 1, 2000. The inspectors also reviewed a total of 24 licensee audits, assessments, trending reports, and selected system report cards, as well as licensee operating experience reviews (NRC Information Notices and 10 CFR Part 21 notifications). The inspectors compared the audit and assessment results with self-revealing and NRC-identified issues to determine the effectiveness of the audits and assessments.

The inspectors evaluated the CRs and the items selected to determine the licensee's threshold for identifying problems and entering them into the corrective action program. Also, the licensee's efforts in establishing the scope of problems were evaluated by reviewing pertinent work orders, operating experience reviews, and audit and self-assessment results. A listing of specific documents reviewed during the inspection is included in the Attachment to this report.

The inspectors also conducted plant walkdowns and interviewed plant personnel to identify other processes by which problems and issues could be identified.

###### (2) Issues and Findings

The inspectors determined that the licensee was effective at identifying problems and entering them into the corrective action program. Licensee audits and assessments were self-critical and thorough and identified issues similar to those that were self-revealing or raised during NRC inspections.

No findings of significance were identified.

##### b. Prioritization and Evaluation of Issues

###### (1) Inspection Scope

The inspectors reviewed 172 CRs and supporting documentation, including analyses of the problem causes, to ascertain whether the licensee's evaluation of the problems identified considered the full extent of conditions, generic implications, common causes, and previous occurrences. In addition, the inspectors reviewed the licensee's evaluation of selected industry experience information, including operating event reports and NRC and vendor

generic notices, to assess if issues applicable to the Fort Calhoun Station were appropriately addressed. Specific items reviewed are listed in the Attachment to this report.

(2) Issues and Findings

Based on a review of the licensee's records, the inspectors concluded that the licensee effectively prioritized and evaluated issues. For the more risk significant CRs, the inspectors determined that the evaluations were of sufficient depth, the root cause determinations were accurate, and risk aspects of the conditions had been appropriately considered.

No findings of significance were identified.

c. Effectiveness of Corrective Actions

(1) Inspection Scope

The inspectors reviewed the CRs, audits, self-assessments, and trending reports described in Section 4OA2.a.(1) above to verify that corrective actions related to the issues were identified and implemented in a timely manner commensurate with safety, including corrective actions to address common cause or generic concerns. The inspectors also conducted plant walkdowns and interviewed plant personnel to independently verify and assess the effectiveness of corrective actions implemented by the licensee. A listing of specific documents reviewed during the inspection is included in the Attachment to this report.

(2) Issues and Findings

The inspectors concluded that the implemented corrective actions for those conditions reviewed were effective. The inspectors did not identify any pattern of recurrence of conditions for which previous corrective actions to prevent recurrence were ineffective.

No findings of significance were identified.

d. Assessment of Safety-Conscious Work Environment

(1) Inspection Scope

The inspectors interviewed approximately 10 individuals from the licensee's staff, which represented a cross-section of functional organizations and supervisory and non-supervisory personnel, regarding their willingness to identify safety issues. These interviews assessed whether conditions existed that would challenge the establishment of a safety-conscious work environment.

(2) Issues and Findings

The inspectors concluded, based on information collected from these interviews, that

employees were willing to identify issues and accepted the responsibility to proactively identify and enter safety issues into the corrective action program.

No findings of significance were identified.

**4OA3** Event Followup (71153)

(Closed) Licensee Event Report 50-285/2001-001-00: primary safety valves outside lift setting acceptance range as specified in Technical Specification 2.1.6(1). The licensee initiated corrective action program document, CR 200101012. The licensee event report was reviewed and no findings of significance were identified. This licensee event report is closed.

(Closed) Licensee Event Report 50-285/2000-002-00: plant operation outside of low temperature overpressure analysis assumptions as specified in Technical Specification 2.1.2. The licensee initiated corrective action program document, CR 200002200. The licensee event report was reviewed and no findings of significance were identified. This licensee event report is closed.

**4OA6** Exit Meeting

The inspectors discussed these findings with Mr. R. Clemens, Plant Manager, and other members of the licensee's staff, on November 2, 2001. Licensee management provided no further comment on the findings.

Licensee management did not identify any materials examined during the inspection as proprietary.



## ATTACHMENT

### PARTIAL LIST OF PERSONS CONTACTED

#### Licensee

M. Burggraf, Operations Engineer, Corrective Action Group  
G. Cavanaugh, Supervisor, Station Licensing  
J. Chase, Manager, NAD  
R. Clemens, Plant Manager  
M. Frans, Manager, Nuclear Licensing  
J. Goodell, Manager, Shift Operations  
R. Haug, Manager, Chemistry  
J. Herman, Manager, P&S  
J. McManis, Manager, Design Engineering Nuclear  
R. Mueller, Supervisor, Design Engineering Nuclear  
K. Naser, Supervisor, System Engineering  
B. Pence, Nuclear Safety Review Group Specialist, NAD  
K. Perdue, Manager, Nuclear Administrative Services  
R. Phelps, Division Manager, Nuclear Engineering  
J. Sefick, Manager, Security and Emergency Preparedness  
J. Spilker, Manager, Corrective Action Group  
K. Steele, Supervisor, Radiation Protection Operations  
M. Tesar, Division Manager, NSS  
J. Tills, Manager, Maintenance  
D. Trausch, Manager, Quality  
R. Westcott, Manager, Nuclear Training  
W. Woerner, Coordinator, Corrective Action Group

### ITEMS OPENED, CLOSED, AND DISCUSSED

#### Closed

2001-001-00	LER	Primary Safety Valves Outside Lift Setting Acceptance Range as Specified in Technical Specification 2.1.6. (Section 4OA4)
2000-002-00	LER	Plant Operation Outside of Low Temperature Overpressure Analysis Assumptions as Specified in Technical Specification 2.1.2. (Section 4OA4).

### DOCUMENTS REVIEWED

The following documents were selected and reviewed by the inspectors to accomplish the objectives and scope of the inspection and to support any findings.

## PROCEDURES

<u>NUMBER</u>	<u>TITLE</u>	<u>REVISION</u>
NOD-QP-21	Operating Experience Review (OER) Program	14
OI-RC-4A	Pressurizer Cooldown	10
OP-2A	Plant Startup	36
OP-3A	Plant Shutdown	31
PED-QP-3	Calculation Preparation, Review and Approval	7
PED-QP-5	Engineering Analysis Preparation, Review and Approval	14
SO-M-100	Conduct of Maintenance	32
SO-M-101	Maintenance Work Control	51 & 54
SO-O-25	Temporary Modification Control	61
SO-R-2	Condition Reporting and Corrective Action	17 & 18
TAP-2	Training Administrative Procedure-2	27
TAP-10A	Training Program Configuration Management Document Tracking Form	11

## OPERATING EXPERIENCE REVIEWS

### NRC Information Notices (INs)

IN 00-08	IN 00-12	IN 00-16	IN 00-20	IN 01-04	IN 01-09
IN 00-09	IN 00-13	IN 00-17	IN 00-21	IN 01-05	IN 01-10
IN 00-10	IN 00-14	IN 00-18	IN-01-01	IN 01-06	IN 01-12
IN 00-11	IN 00-15	IN 00-19	IN 01-02	IN 01-07	IN 01-14

### Part 21s

2000-18	2001-01	2001-05	2001-08	2001-12	2001-16
2000-19	2001-02	2001-07	2001-11	2001-15	2001-23
2000-21	2001-03				

CONDITION REPORTS (CRs)

199902452	200002402	200100441	200101043	200101663	200102288
200000207	200002414	200100469	200101057	200101665	200102320
200000975	200002474	200100473	200101071	200101724	200102340
200000993	200002475	200100503	200101073	200101772	200102409
200000988	200002518	200100515	200101106	200101889	200102410
200001064	200002526	200100516	200101110	200101904	200102426
200001105	200002529	200100594	200101128	200102026	200102435
200001159	200002530	200100596	200101134	200102028	200102450
200001254	200002531	200100598	200101137	200102029	200102526
200001302	200100003	200100611	200101144	200102089	200102553
200001333	200100155	200100628	200101357	200102099	200102555
200001346	200100178	200100657	200101381	200102154	200102582
200001381	200100192	200100669	200101383	200102164	200102607
200001459	200100216	200100710	200101421	200102186	200102624
200001535	200100241	200100762	200101471	200102195	200102632
200001536	200100250	200100765	200101477	200102199	200102742
200001956	200100274	200100773	200101520	200102209	200102776
200001974	200100275	200100782	200101521	200102217	200102794
200001980	200100289	200100842	200101522	200102220	200102817
200001989	200100291	200100930	200101523	200102426	200102860
200002087	200100328	200100951	200101529	200102435	200102947
200002088	200100329	200100953	200101606	200102254	200102953
200002093	200100330	200100954	200101607	200102263	200102974
200002126	200100388	200101002	200101609	200102282	200103076
200002167	200100390	200101010	200101610	200102283	200103189
200002200	200100392	200101012	200101611	200102285	200103194
200002211	200100394	200101030	200101613	200102286	200103280
200002275	200100421	200101032	200101614	200102287	200103284
200002371	200100425	200101042	200101627		

CONDITION REPORTS (CRs) WRITTEN AS A RESULT OF THIS INSPECTION

200103340  
200103355  
200103378  
200103410  
200103426

ROOT CAUSE ANALYSES

200001013	200001473	200001661	200002402
200001064	200001535	200002200	200100332

#### TEMPORARY MODIFICATION PACKAGES

<u>NUMBER</u>	<u>DESCRIPTION</u>	<u>DATE</u>
10375/10333	Place FW-10 Throttle Valve Positioner YC-1039-2 in Bypass Without Taking FW-10 OOS	June 12, 2000
10148	Remove FE-417 from VA-1B Containment Cooling Coil	May 20, 1999

#### WORK ORDERS

<u>NUMBER</u>	<u>DESCRIPTION</u>	<u>DATE</u>
00068532	Water Leaking from Raw Water Header Upstream of FE-2890	March 30, 2001
00082906	Repair #2 Motor Lead on HCV-2954	September 5, 2000
00083601	HCV-2882B Open Limit Switch is not Making Contact	April 3, 2001
00083958	Switch HC-307 is Hard to Turn and will not Open Valve HCV-307	April 8, 2001
00086940	No Power to Motor for Raw Water Strainer AC-12B	May 15, 2001

#### ASSESSMENTS AND AUDITS

Chemical and Volume Control System Reliability Assessment, dated June 22, 2001

Corrective Action Group Special Report - September 2000, Organizational Overview of Human Performance at Fort Calhoun Station

Corrective Action Group Self Assessment Report, Corrective Action Program, performed August 6 - 10, 2001

Emergent Quality Assurance Surveillance Report H-00-1, Review of Corrective Actions for Condition Reports 199902076 and 1999022452 Related to RHRA Events, dated June 1, 2000

NSRG Observation of DG-1 Inoperability due to Air Start Regulator Problem, Report No. SRG-00-030, performed May 31 through June 2, 2000

NSRG Review of FCS Audits/Self-Assessments/INPO Assist Reports Collectively for Adverse Trends, Report No. SRG-00-034, performed June 23 - 29, 2000

NSRG Observation of DG-1 Inoperability due to Air Start regulator Problem, Report No.

SRG-00-030, performed May 31 through June 2, 2000

NSRG Review of Significant Historical FCS Operating Experience, dated October 26, 2001

Nuclear Safety Review Group Review of Condition Report Action Item Extensions, dated August 10, 2001

Nuclear Safety Review Group Review of Condition Report Action Item Extensions, Report No. 01-QUA-051 Rev.1, dated September 6, 2001

Quality Oversight Second Quarter Observation Report, dated June 29, 2001

Radiation Protection Program Self Assessment Report CHP-01-20, dated September 27, 2001

SARC Audit Report No. 45 (00-QA/QC-073) - Corrective Action Program, dated December 21, 2000

SARC Audit Report No. 45 (01-QUA-076) - Corrective Action Program, dated September 21, 2001

#### MISCELLANEOUS DOCUMENTS

2001 RFO Trend Report - Day 22, issued April 7, 2001

Corrective Action Group Quarterly Trending Report, dated July 25, 2000

Corrective Action Group Quarterly Trend Report, 3<sup>rd</sup> Quarter, 2000

Corrective Action Group Quarterly Trend Report and Common Cause Analysis, 2<sup>nd</sup> Quarter, 2001

Diesel Generator System Report Card for the Report Period January 1 Through June 30, 2001

Electrical Distribution System Report Card for Report Period January 1, 2001 Through June 30, 2001

Fort Calhoun Station 2000 Integrated Assessment Schedule

Fort Calhoun Station Exercise Critique, dated October 23, 2001

Fort Calhoun Station Procedure/Procedure Change Request EC-28598, Condition Reporting and Corrective Action

Letter LIC-01-0075, "Response to NRC Bulletin 2001-02, [Circumferential Cracking of Reactor Pressure Vessel Head Penetration Nozzles]," August 31, 2001

OPPD Letter , "10 CFR Part 21 "Reporting of Defects and Noncompliances," dated May 9 and 24, 2000

PRC Meeting Minutes 01-036, dated April 23, 2001

Plant Review Committee Agenda for October 30, 2001

Safety Injection System Report Card for Report Period January 1, 2001 Through June 30, 2001

System Report Card NED-SYE-01-185, "Safety Injection System Report Card for Report period January 1 Through June 30, 2001

System Update Reports, dated October 18, 2001

System Updates July 1 through September 30, 2001, Primary Systems

TSI 01-005, "Technical Specification Interpretation Providing Interim Guidance for Shutdown Required by Excessive RCS Leak Rates," January 15, 2001

### **Information Request 1**

FCS PIR Inspection (IP 71152) 01-03

The inspection will cover the period of 5/15/00 to 5/15/01. All requested information should be limited to this period unless otherwise specified. The information may be provided in either electronic or paper media or a combination of these. Information provided in electronic media may be in the form of e-mail attachment(s), CDs, or 3 ½ floppy disks. The agency's text editing software is WordPerfect 8; however, we have document viewing capability for MS Word and Adobe Acrobat (.pdf) text files.

Please provide the following information to S. McCrory in the Region IV Arlington office by 4/16/01:

1. Summary list of all currently open/active items for:
  - condition reports of significant conditions adverse to quality
  - operator work-arounds
  - engineering review requests
  - maintenance requests
  - temporary modifications
  - procedure change requests
  - training needs request/evaluation
  - control room and safety system deficiencies
2. Summary list of all items completed/resolved/closed since 5/15/00 for:
  - condition reports of significant conditions adverse to quality
  - operator work-arounds
  - engineering review requests
  - maintenance requests
  - temporary modifications
  - procedure change requests
  - training needs request/evaluation
  - control room and safety system deficiencies

3. Summary list of all condition reports generated during the specified period and sorted by:  
Chronology  
Initiating organization  
Responsible organization
4. All quality assurance audits and surveillances of corrective action activities since 5/15/00.
5. All corrective action activity and functional area self-assessments and Non-NRC third party assessments since 5/15/00.
6. Corrective action performance trending/tracking information generated since 5/15/00 and broken down by functional organization.
7. Current revision of the following procedures:  
SO-R-2 Condition Reporting and Corrective Action  
NOD-QP-19 Cause Analysis Program  
NOD-QP-20 Human Performance Enhancement System Program  
NOD-QP-21 Operating Experience Review Program
8. Any additional governing procedures/policies/guidelines for:  
Condition Reporting  
Corrective Action Program  
Root Cause Evaluation/Determination  
Operator Work-Arounds  
Work Requests  
Engineering Requests  
Temporary Modifications  
Procedure Change Requests  
Deficiency Reporting and Resolution  
Training Needs Request/Evaluation
9. For each of the items (applicable to FCS) listed below please provide the following:  
  - \* Full text of the condition report (please indicate any findings that did not result in a condition report or corrective actions)
  - \* Any "Roll-up" or "Aggregating" Conditions Reports related to the generic communication or condition report.
  - \* Root Cause analysis report (if applicable)

- \* Risk significance assessments
  - \* Probable Cause evaluation (if applicable)
  - \* Approved corrective actions
  - \* Basis for extending originally approved due dates
  - \* Evidence of corrective action completion (work packages, design change documentation, temporary modifications, training lesson plans/material, training attendance records, procedure revisions, etc.)
- a. Part 21 Reports 2000-16-0 through 2001-09-0.
  - b. NRC Information Notices 2000-08 through 2000-22
  - c. All LERs issued since 5/15/00
  - d. All NCVs and Violations issued since 5/15/00

## **Information Request 2**

### **PIR Inspection**

I would like the following information available when we arrive onsite:

1. Listings of all Level 1, 2, 3, and 4 Condition Reports written since 5/1/2000. I would like one separate list for each level. I would like to have the following information on the list: CR number, level number, description, owner name, owner organization, status (open or closed), and associated CR number (If possible).
2. Updated list of all Condition Reports dealing with operator workarounds and a list of reports dealing with control room deficiencies since 5/1/2000. Same information please as described in 1.
3. Copies of all Condition Reports regarding M&TE issues since 5/1/2000.
4. Copies of the following Condition Reports:

00-0993	00-1064	00-1105	00-1254	00-1333	00-1459
00-2068	01-0003	01-0192	01-0250	01-0289	01-0425
01-0503	01-0515	01-0765	01-0782	01-1010	01-1042
01-1043	01-1137	01-1471	01-1477	01-1614	01-1772
01-2282	01-2283	01-2285	01-2286	01-2287	01-2410
01-2414	01-2553				



00-1321	00-1941	00-2089	00-2200	00-2275	00-2299
00-2303	00-2403	00-2421	01-0144	01-0216	01-0332
01-0354	01-0491	01-0563	01-0621	01-1013	01-1264
01-1290	01-1292	01-2289	01-2290	01-2291	
00-1159	00-1302	00-1956	00-1956	00-1974	00-2011
00-2275	00-2518	00-2525	00-2526	01-0241	01-0388
01-0391	01-0421	01-0628	01-0710	01-0773	01-0805
01-0825	01-0933	01-0936	01-0949	01-1050	01-1128
01-1134	01-1529	01-2099	01-2254	01-2288	01-2296
01-2467	01-2491	01-2522			

### Information Request 3

#### **BUNDY DOCUMENT REQUESTS - FCS PI&R INSPECTION**

- root cause analysis and safety assessment associated with CR 200002200
- Procedures OI-RC-4A, OP-2A, and OP-3A re changes pursuant to CR 200002200
- EPIP-OSC-21 re changes made pursuant to CR 200002299
- CR 00-1473 re NCV 285/00-07
- CR 00-2402 re NCV 285/00-10
- CR 00-1401 re NCV 285/0102-2
- CR 01-1262 re NCV 285/0102-1
- CR 00-1536 re IN 00-009
- CR 00-1535 re IN 00-011
- CR 00-1979 re IN 00-013
- CR 00-2529 re IN 00-020
- CR 00-2530 re IN 00-021
- IN 01-005 (through-wall circumferential cracking of CRDM RPV head penetration) tracking report
- IN 01-009 (main FW piping degradation inside containment) tracking report

- Part 21 from Summer on 3/15/01 (flaws in RPV nozzle weld) tracking report
- CR 00-1064 re drifting of DG starting air regulators
- TSI 01-01-0 (TS interpretation providing interim guidance for shutdown required by excessive RCS leak rates)
- Root Cause Analysis associated with CR 2001 00332 re errors in nuclear power report
- Procedures PED-QP-03 and PED-QP-05
- RCA associated with CR 200101013 (incorrect incore detectors ordered)

**BUNDY INTERVIEW REQUESTS - FCS PI&R**

- System Engineer for Main Turbine re vibration reported in CR 200001941