

# **INITIAL SUBMITTAL**

**CATAWBA EXAM  
50-413, 414/2001-301**

**APRIL 2 - 6 & 16 - 20, 2001**

**INITIAL SUBMITTAL  
RO/~~SEE~~ WRITTEN EXAMINATION**

**Nuclear Regulatory Commission  
Reactor Operator Licensing  
Examination**

**Catawba Nuclear Station**

**RO Exam**

**With ANSWER KEY**

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# Catawba Sample Plan

Facility: Catawba				Date of Exam: 5/19/00				Exam Level: RO							
		K/A Category Points											Point		
Tier	Group	K 1	K 2	K 3	K 4	K 5	K 6	A 1	A 2	A 3	A 4	G	Total	Target	
1 Emergency & Abnormal Plant Evolutions	1	1	5	3				4	2			1	16	16	
	2	1	2	4				5	1			4	17	17	
	3	0	1	0				1	1			0	3	3	
	Tier Totals	2	8	7				10	4			5	36	36	
2 Plant Systems	1	1	1	1	1	3	1	1	5	2	3	4	23	23	
	2	1	1	3	4	1	1	3	3	2	1	0	20	20	
	3	1	1	2	1	0	0	1	0	0	1	1	8	8	
	Tier Totals	3	3	6	6	4	2	5	8	4	5	5	51	51	
3	Generic Knowledge and Abilities	Cat 1			Cat 2			Cat 3			Cat 4		13	13	
		3			3			4			3				
<p><b>Note:</b></p> <ul style="list-style-type: none"><li>* Attempt to distribute topics among all K/A categories; select at least one topic from every K/A category within each tier.</li><li>* Actual point totals must match those specified in the table.</li><li>* Select topics from many systems; avoid selecting more than two or three K/A topics from a system unless they relate to plant-specific priorities.</li><li>* Systems /evolutions within each group are identified on the associated outline.</li><li>* The shaded areas are not applicable to the category/tier.</li><li>** Denotes plant specific, high priority K/As</li></ul>															Totals: 100

Catawba Sample Plan

E/APE # / Name / Safety Function	K 1	K 2	K 3	A 1	A 2	G	K/A Topic(s)	Imp.	Points	Question
000005 Inoperable/Stuck Control Rod / I				1.02			Ability to operate and/or monitor ...rod selection switches	3.7/3.5	1	779
000015/17 RCP Malfunction / IV		2.07					Knowledge of the interrelationships between...RCP seals	2.9/2.9	1	786
000024 Emergency Boration / I						4.1	Knowledge of EOP entry conditions and immediate action steps	4.3/4.6	1	764
000026 Loss of Component Cooling Water / VIII				1.07			Ability to operate and/or monitor ...flow rates to the components and systems that are serviced by the CCWS; interactions among components	2.9/3.0	1	782
000027 Pressurizer Pressure Control System Malfunction / III		2.03					Knowledge of the interrelationships between...controllers and positioners	2.6/2.8	1	783
000040 Steam Line Rupture - Excessive Heat Transfer / IV				1.12			Ability to operate and/or monitor ...RCS pressure and temperature	4.2/4.2	1	791
000051 Loss of Condenser Vacuum / IV					2.02		Ability to determine and interpret...conditions requiring a reactor or turbine trip	3.9/4.1	1	453a
000055 Station Blackout / VI				1.01			Ability to operate and/or monitor ...in-core thermocouple temperatures	3.7/3.9	1	788
000057 Loss of Vital Ac Elec. Inst. Bus. / VI							randomly deselected			
000062 Loss of Nuclear Service Water / IV					2.02		Ability to determine and interpret...the cause of possible SWS loss	2.9/3.6	1	757
000068 Control Room Evac. / VIII			3.12				Knowledge of the reasons for the following responses ... Required sequence of actions for emergency evacuation of the control room	4.1/4.5	1	596a
000076 High Reactor Coolant Activity / IX		2.01					Knowledge of the interrelationships between...process radiation monitors	2.6/3.0	1	380
K/A Category Totals:	1	5	3	4	2	1	Group Point Total:	16	16	16

Catawba Sample Plan

											Bank
E/APE # / Name / Safety Function	K 1	K 2	K 3	A 1	A 2	G	K/A Topic(s)	Imp.	Points	Question	
000001 Continuous Rod Withdrawal / I						1.32	Ability to explain all system limits and precautions	3.4/3.8	1	503a	
000003 Dropped Control Rod /II						4.1	Knowledge of EOP entry conditions and immediate action steps	4.3/4.6	1	766	
000007 Reactor Trip - Stabilization - Recovery / I							randomly deselected				
000008 Pressurizer Vapor Space Accident / III	1.01						Knowledge of the operational implications of the following concepts ...thermodynamics and flow characteristics of open or leaking valves	3.2/3.7	1	311	
000009 Small Break LOCA / III						3.9	Knowledge of process for performing a containment purge	2.5/3.9	1	767	
000011 Large Break LOCA /III			3.10				Knowledge of the reasons for the following responses ... PTS limits on RCS pressure and temperature	3.7/3.9	1	080a	
W/E04 LOCA Outside Containment / III		2.1					Knowledge of the interrelationships between...components and functions of control and safety systems including instrumentation, signals, interlocks, failure modes, and automatic and manual features	3.5/3.9	1	388	
W/E11 Loss of Emergency Coolant Recirc / IV							randomly deselected				
W/E02 SI Termination / III					2.1		Ability to determine and interpret the following... facility conditions and selection of appropriate procedures during abnormal and emergency operations	3.3/4.2	1	185b	
000022 Loss of Reactor Coolant Makeup / II				1.09			Ability to operate and/or monitor ...RCP seal flows, temperatures, pressures and vibrations	3.2/3.3	1	647a	
000025 Loss of RHR System / IV							randomly deselected				
000029 Anticipated Transient w/o Scram / I							randomly deselected				
000032 Loss of Source Range NI / VII				1.01			Ability to operate and/or monitor ...manual restoration of power	3.1*/3.4*	1	393	
000033 Loss of Intermediate Range NI / VII							randomly deselected				
000037 Steam Generator Tube Leak / III						4.34	Knowledge of RO tasks performed outside the main control room during emergency operations including system geography and system implications	3.8/3.6	1	300b	
000038 Steam Generator Tube Rupture / III				1.38			Ability to operate and/or monitor ...PZR heaters	3.3/3.3	1	193a	
000054 Loss of Main Feedwater / IV			3.03				Knowledge of the reasons for the following responses ... manual control of APFW flow control valves	3.8/4.1	1	768	
W/E05 Inadequate Heat Transfer - Loss of Secondary Heat Sink / IV				1.1			Components and functions of control and safety systems, including instrumentation, signals, interlocks, failure modes and automatic and manual features	4.1/4.0	1	769	
000058 Loss of DC Power / VI				1.01			Ability to operate and/or monitor...cross-tie of the affected DC bus with the alternate supply	3.4*/3.5	1	770	
000059 Accidental Liquid Radwaste Rel. / IX							randomly deselected				
000060 Accidental Gaseous Radwaste Rel. / IX			3.02				Knowledge of the reasons for the following responses ... Isolation of auxiliary building ventilation	3.3*/3.5*	1	771	
000061 ARM System Alarms / VII							randomly deselected				
W/E16 High Containment Radiation / IX		2.1					Knowledge of the interrelationships between...components and functions of control and safety systems including instrumentation, signals, interlocks, failure modes, and automatic and manual features	3.0/3.3	1	797	
K/A Category Totals:	1	2	4	5	1	4	Group Point Total:			17	
										17	17

Catawba Sample Plan

E/APE # / Name / Safety Function	K 1	K 2	K 3	A 1	A 2	G	K/A Topic(s)	Imp.	Points	Question
000028 Pressurizer Level Malfunction / II				1.02			Ability to operate and/or monitor ...CVCS	3.4/3.4	1	104a
000036 Fuel Handling Accident / VIII				2.02			Ability to determine and interpret...occurrence of a fuel handling accident	3.4/4.1	1	33
000056 Loss of Off-site Power / VI							randomly deselected			
000065 Loss of Instrument Air / VIII							randomly deselected			
W/E 13 Steam Generator Over-pressure / IV							randomly deselected			
W/E 15 Containment Flooding / V		2.1					Knowledge of the interrelationships between...components and functions of control and safety systems including instrumentation, signals, interlocks, failure modes, and automatic and manual features	2.8/2.9	1	518
K/A Category Totals:	0	1	0	1	1	0		Group Point Total:	3	3

															Bank
System # / Name	K 1	K 2	K 3	K 4	K 5	K 6	A 1	A 2	A 3	A 4	G	K/A Topic(s)	Imp.	Points	Question
001 Control Rod Drive											4.22	Knowledge of the bases for prioritizing safety functions during abnormal and emergency operations	3.0/4.0		1714
003 Reactor Coolant Pump					5.04							Knowledge of the following operational implications...effects of RCP shutdown on secondary parameters such as steam pressure, steam flow and feed flow	3.2/3.5		1715
004 Chemical Volume Control		2.02										Knowledge of bus power supplies to...makeup pumps	2.9/3.1		1725
013 Engineered Safety Features Actuation									3.02			Ability to monitor automatic operation of the...operation of actuated equipment	4.1/4.2		1716
015 Nuclear Instrumentation											2.2	Ability to manipulate the console controls as required to operate the facility between shutdown and designated power levels	4.0/3.5		1718
017 In-core Temperature Monitor								2.01				Ability to predict the impacts of the following malfunction or operation...and based on those predictions, use procedures to correct, control or mitigate the consequences of...thermocouple open and short circuits	3.1/3.5		1793
022 Containment Cooling									3.01			Knowledge of the effect that a loss or malfunction will have on...initiation of safeguards mode of operation	4.1/4.3		1720
056 Condensate								2.04				Ability to predict the impacts of the following malfunction or operation...and based on those predictions, use procedures to correct, control or mitigate the consequences of...loss of condensate pumps	2.6/2.8*		1722
059 Main Feedwater											4.12	Ability to manually operate and/or monitor in the control room...initiation of automatic feedwater isolation	3.4/3.5		1723
061 Auxiliary/Emergency Feedwater					5.01							Knowledge of the following operational implications...relationship between AFW flow and RCS heat transfer	3.6/3.9		1724
071 Waste Gas Disposal				4.04								Knowledge of design feature(s) and/or interlock(s) which provide for...isolation of waste gas release tanks	2.9/3.4		1726
072 Area Radiation Monitoring											2.28	Knowledge of new and spent fuel movement procedures	2.6/3.5		1263
003 Reactor Coolant Pump											4.03	Ability to manually operate and/or monitor in the control room...RHR temperature, PZR heaters and flow, and Nitrogen	2.8*/2.7*		1728
072 Area Radiation Monitoring											4.01	Ability to manually operate and/or monitor in the control room...alarm and interlocks setpoint checks and adjustments	3.0*/3.3		1789
K/A Category Totals:	1	1	1	1	3	1	1	5	2	3	4		23	23	23

K/A Category Totals:																
1	1	1	1	1	1	1	1	1	1	1	1	1	1	1	1	1
2	2	2	2	2	2	2	2	2	2	2	2	2	2	2	2	2
3	3	3	3	3	3	3	3	3	3	3	3	3	3	3	3	3
4	4	4	4	4	4	4	4	4	4	4	4	4	4	4	4	4
5	5	5	5	5	5	5	5	5	5	5	5	5	5	5	5	5
6	6	6	6	6	6	6	6	6	6	6	6	6	6	6	6	6
7	7	7	7	7	7	7	7	7	7	7	7	7	7	7	7	7
8	8	8	8	8	8	8	8	8	8	8	8	8	8	8	8	8
9	9	9	9	9	9	9	9	9	9	9	9	9	9	9	9	9
10	10	10	10	10	10	10	10	10	10	10	10	10	10	10	10	10
11	11	11	11	11	11	11	11	11	11	11	11	11	11	11	11	11
12	12	12	12	12	12	12	12	12	12	12	12	12	12	12	12	12
13	13	13	13	13	13	13	13	13	13	13	13	13	13	13	13	13
14	14	14	14	14	14	14	14	14	14	14	14	14	14	14	14	14
15	15	15	15	15	15	15	15	15	15	15	15	15	15	15	15	15
16	16	16	16	16	16	16	16	16	16	16	16	16	16	16	16	16
17	17	17	17	17	17	17	17	17	17	17	17	17	17	17	17	17
18	18	18	18	18	18	18	18	18	18	18	18	18	18	18	18	18
19	19	19	19	19	19	19	19	19	19	19	19	19	19	19	19	19
20	20	20	20	20	20	20	20	20	20	20	20	20	20	20	20	20
Group Point Total:																
20	20	20	20	20	20	20	20	20	20	20	20	20	20	20	20	20

System # / Name	K 1	K 2	K 3	K 4	K 5	K 6	1 A	2 A	3 A	4 G	K/A Topic(s)	Imp.	Points	Question
010 Pressurizer Pressure Control											Knowledge of physical connections and/or cause and effect relationships...ESFAS	3.9/4.1	1	736
011 Pressurizer Level Control				3.03							Knowledge of the effect that a loss or malfunction...will have on...PZR PCS	3.2/3.7	1	737
012 Reactor Protection										2.07	Ability to predict the impacts of the following malfunction or operation...and based on those predictions, use procedures to correct, control or mitigate the consequences of...loss of DC control power	3.2/3.7	1	738
014 Rod Position Indication				4.02							Knowledge of design feature(s) and/or interlock(s) which provide for...lower electrical limit	2.5/2.7	1	739
016 Non-nuclear Instrumentation				4.01							Knowledge of design feature(s) and/or interlock(s) which provide for...reading of NNIS channels outside control room	2.8/2.9	1	740
026 Containment Spray				4.07							Knowledge of design feature(s) and/or interlock(s) which provide for...adequate level in containment sump for suction (interlock)	3.8/4.1	1	741
028 Containment Purge										3.01	Knowledge of the effect that a loss or malfunction will have on...CPS isolation	3.8/4.0	1	762a
033 Spent Fuel Pool Cooling										1.01	Ability to manually operate and/or monitor in the control room...spent fuel pool water level	2.7/3.3	1	743
035 Steam Generator										6.03	Knowledge of the effect that a loss or malfunction will have on...S/G level detector	2.8/3.0	1	744a



K/A Category Totals:																Group Point Total:			
System # / Name	K 1	K 2	K 3	K 4	K 5	K 6	A 1	A 2	A 3	A 4	G	K/A Topic(s)	Imp.	Points	Question				
005 Residual Heat Removal	1.10											Knowledge of physical connections and/or cause and effect relationships...CSS	3.2/3.4		1 751				
008 Component Cooling Water			3.03									Knowledge of the effect that a loss or malfunction ...will have on ...RCP	4.1/4.2		1 752				
027 Containment Iodine Removal										4.03		Ability to manually operate and/or monitor in the control room...CIRS fans	3.3/3.2		1 444a				
028 Hydrogen Recombiner and Purge Control		2.01										Knowledge of bus power supplies to...Hydrogen recombiners	2.5*/2.8*		1 798				
034 Fuel Handling Equipment												randomly deselected							
041 Steam Dump/Turbine Bypass Control											2.1	Ability to perform pre-startup procedures for the facility, including operating those controls associated with plant equipment that could affect reactivity	3.7/3.8		1 754				
045 Main Turbine Generator												randomly deselected							
103 Containment												randomly deselected							
	</																		

**Catawba Sample Plan**

Category	K/A #	Topic	Imp.	Points	Question
<b>Conduct of Operations</b>	1.7	Ability to evaluate plant performance and make operational judgments based on operating characteristics, reactor behavior, and instrument interpretation	3.7/4.4	1	556
	1.24	Ability to obtain and interpret station electrical and mechanical drawings	2.8/3.1	1	773
	1.31	Ability to locate control room switches, controls and indications to determine if they are correctly reflecting the desired plant lineup	4.2/3.9	1	792
			3	3	
<b>Equipment Control</b>					
	2.34	Knowledge of process for determining the internal and external effects on core reactivity	2.8/3.2*	1	189a
			3	3	
<b>Radiation Control</b>	3.2	Knowledge of facility ALARA program	2.5/2.9	1	124
	3.4	Knowledge of radiation exposure limits and contamination control including permissible levels in excess of those authorized	2.5/3.1	1	661a
	3.10	Ability to perform procedures to reduce excessive levels of radiation and guard against personnel exposure	2.9/3.3	1	775
	3.11	Ability to control radiation releases	2.7/3.2	1	671a
			4	4	
<b>Emergency Procedures and Plan</b>	4.17	knowledge of EOP terms and definitions	3.1/3.8	1	776
	4.19	Knowledge of EOP layout, symbols and icons	2.7/3.7	1	339a
	4.15	Knowledge of communications procedures associated with EOP implementation	3.0/3.5	1	387a
			3	3	
<b>Tier 3 Point Total</b>			<b>13</b>	<b>13</b>	<b>13</b>

The following table summarizes the changes made to the randomly selected K/As. All changes were discussed and approved with the NRC Chief Examiner.

EAPE or System	K/A Tested	Summary of Changes
T1G1 000024 Emergency Boration	G2.4.1	Randomly selected 2.3.9 (Radiation Control – knowledge of process for performing a containment purge). This K/A was not applicable for the selected EAPE. Selected next applicable K/A in sequence.
T1G2 000003 Dropped Control Rod	G2.4.1	Randomly selected 2.3.1 (Knowledge of 10CFR20 and related facility radiation requirements) and 2.3.4 (Knowledge of radiation exposure limits, contamination control including permissible limits in excess of those authorized). These K/As are PWGs that are not applicable to this EAPE. Randomly selected next K/A in sequence.
T1G2 000009 Small Break LOCA	G2.3.9	Randomly selected 2.3.4 (Knowledge of radiation exposure limits, contamination control including permissible limits in excess of those authorized). This K/A was a PWG that is not applicable to this EAPE. Randomly selected next K/A in sequence.
T1G2 000011 Large Break LOCA	K3.10	Randomly selected K3.11 (Knowledge of the reasons for the following responses ...NC and PC). The team could not determine what this K/A was testing and did not know the terminology “PC”. Randomly reselected K3.10.
T1G2 000037 Steam Generator Tube Leak	G2.4.34	Randomly selected G2.4.39 (Knowledge of ROs responsibilities in emergency plan implementation). This was a PWG that did not test any EAPE specific knowledge. Randomly reselected G2.4.34.
T1G2 000058 Loss of DC Power	A1.10	The original sample plan contained a typographical error. The stem statement was incorrect. Corrected in the attached sample plan.
T2G1 015 Nuclear Instrumentation	G2.2.2	Randomly selected 2.2.6 (Knowledge of process for making changes in procedures as described in the SAR) and 2.2.29 (Knowledge of SRO fuel handling responsibilities). These K/As are PWGs that are not applicable to this system. Randomly selected G2.2.2.

T2G1 068 Liquid Rad Waste	A2.04	Shifted from K/A 2.03 with NRC approval (Ability to predict the impacts of the following malfunction or operation...and based on those predictions, use procedures to correct, control or mitigate the consequences of...insufficient sampling frequency of boric acid in the evaporator bottoms) because we did not know what this K/A was testing. Randomly selected A2.04.
T2G1 003 Reactor Coolant Pump	A4.03	Originally randomly sampled the Condensate System (0056). Decided to resample system to RCPs because the condensate system had already been sampled once and there were very few K/As that were < 2.5 in this system.
T2G1 013 Engineered Safety Features Actuation	G2.1.32	Randomly sampled G2.1.16 (paging system operation) and G2.1.26 (non-nuclear safety - high temp, high pres, etc). These K/As were PWGs and were not directly applicable to this system. Randomly selected G2.1.32.
T2G2 010 Pressurizer Pressure Control	K1.02	Randomly sampled K4.02. However, there was only 1 K1 that had been randomly sampled in all Tier 2 systems on the SRO exam. The ES-1021 requires a minimum of 2 questions in each category to assure that all categories are represented (assuming one question is deleted during exam review). Randomly selected this question (from all systems that had K4s selected) and randomly reselected K1.02. This increased the number of K1s from 1 to 2 in Tier 2 and decreased the number of K4s from 7 to 6 on the SRO exam.
T2G2 064 Emergency Diesel Generator	A1.08	Randomly selected A1.02 (Ability to predict and/or monitor changes in parameters (to prevent exceeding design limits) associated with operating the...fuel consumption rate with load). However, this K/A was judged not to be appropriate RO level of knowledge at Catawba.
T2G3 041 Steam Dump/Turbine Bypass Control	G2.2.1	Randomly selected K/A G2.3.6 (Radiation control: Knowledge of requirements for preparing a radiation work permit), K/A G2.3.10 (Radiation Control: Ability to perform procedures to reduce excessive levels of radiation and guard against

		personnel exposure) and G2.3.4 (Knowledge of radiation exposure limits, contamination control including permissible limits in excess of those authorized) – these K/As were determined to be inappropriate for this system. Selected G2.2.1 at the direction of the Chief Examiner.
Tier 3 Group 1 Conduct of Operations	G2.1.24	Randomly selected G2.1.27 (Knowledge of system purpose or function), which was not a PWG. Randomly resampled.
Tier 3 Group 2 Equipment Control	G2.2.22	Randomly sampled G2.2.4 (Ability to explain the variations in control board layouts, systems, instrumentation and procedural actions between units at a facility). This K/A had initially already been used in systems questions and was removed as a duplicate – but this K/A was later deselected during the sample plan review process.
Tier 3 Group 4 Emergency Procedures and Plans	G2.4.15	Randomly selected G2.4.22 (Knowledge of bases for prioritizing safety functions during abnormal and emergency operations). This K/A had initially already been used in systems questions and was removed as a duplicate from Tier 2 Group 1 Control Rod Drive system. Randomly selected G2.4.15 as replacement K/A.

## **RO Exam References:**

Steam Tables – question 311

Tech Specs:

- Tech Spec 3.4.13 - question 480A
- Tech Spec 3.8.4 - question 770
- Tech Spec 3.8.7 - question 770
- Tech Spec 3.8.8- question 770
- Tech Spec 3.9.2 - question 774

SLCs:

- SLC 16.11-2

EOPs

- E-1 page 5, Steam Tables – question 185b
- ES-0.1 Encl 3 page 21 – question 780

AOPs

- AP/1/A/5500/17 (Loss of Control Room) – question 189a

Other

- Reactor Operating Databook, Sections 4.1 & 4.2 – question 189a

# **Nuclear Regulatory Commission Reactor Operator Licensing Examination**

## **Catawba Nuclear Station RO Exam**

Answer Key

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Date of examination

**Bank Question: 033****Answer: B**

1 Pt(s) Unit 1 is in a refueling outage. Given the following events and conditions:

- A full core off-load is in progress
- One spent fuel assembly is in the fuel transfer tube being transported to the spent fuel pool
- The following annunciators alarm:
  - SPENT FUEL POOL LEVEL HI/LO
  - 1EMF-17 REACTOR BLDG REFUEL BRIDGE

Which one of the following correctly describes the type of event and the required operator actions that should be performed first?

- A. **Loss of refueling cavity or spent fuel pool level.**  
**Install the weir gate and inflate the seals.**
- B. **Loss of refueling cavity or spent fuel pool level**  
**Move the fuel transfer cart to the spent fuel side (pit).**
- C. **Loss of spent fuel pool level only**  
**Move the fuel transfer cart to the reactor side.**
- D. **Loss of refueling cavity level only**  
**Close 1KF-122 (KF Fuel Transfer Canal Isolation).**

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**Distracter Analysis:**

- A. **Incorrect:** using the weir gate is optional.  
**Plausible:** this is one of the operator follow-up actions.
- B. **Correct:**
- C. **Incorrect:** the cart must be moved to the SFP side.  
**Plausible:** if the problem is on the SFP side it might be reasonable to not to add more fuel to that side.
- D. **Incorrect:** level is dropping on both sides, and you can't close valve with cart in the tube.  
**Plausible:** the candidate may choose this answer due to the EMF alarm and closing the valve is the next action.

Level: RO&SRO

KA: APE 036AA.2.02(3.2/3.9)

Lesson Plan Objective: KF LPRO 15



Source: Mod Ques\_033e

Level of knowledge: memory

References:

1. OP-CN-FH-FHS pages 7, 17
2. AP/1/A/5500/26 pages 1-2

**Bank Question: 080a****Answer: D**

1 Pt(s) Which one of the following accidents has the highest severity for pressurized thermal shock (PTS) in the NCS?

- A. Small break LOCA, NCPs running
- B. Large break LOCA, NCPs NOT running
- C. Large break LOCA, NCPs running
- D. Small break LOCA, NCPs NOT running

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**Distracter Analysis:**

- A. **Incorrect:** PTS is worse with NCPs not running.  
**Plausible:** small LOCAs have PTS potential.
- B. **Incorrect:** large break LOCA has little concern for PTS.  
**Plausible:** If candidate misunderstands significance of no pressure versus NCPs not running.
- C. **Incorrect:** large break LOCA has little concern for PTS  
**Plausible:** large break LOCAs with pumps running are more limiting in other respects.
- D. **Correct:**

Level: RO&SRO

KA: EPE 011EK3.10(3.7/3.9)

Lesson Plan Objective: PTS SEQ 13, 14

Source: Mod Ques\_080

Level of knowledge: comprehension

References:

1. OP-CN-TA-PTS page 12-18

**Bank Question: 104a****Answer: C**

1 Pt(s) Unit 1 is operating at 100% power. Given the following events and conditions:

- Pressurizer pressure and level controls are in the 1-2 position.
- NCS pressure is 2200 psig
- NCS temperature is 560 °F.
- Charging flow increases
- Pressurizer level increases
- PZR low level deviation alarms.
- All pressurizer heaters remain energized throughout the event

What is the cause of these indications?

- A. Pressurizer level master controller output fails high
- B. Pressurizer level channel I fails low
- C. NCS Loop C narrow range T<sub>c</sub> channel fails high
- D. Pressurizer level channel II fails low

---

**Distracter Analysis:** Tcold failing high causes reference level to fail high. This causes the master level controller output to go high on reference level. This causes increased charging and increased PZR level.

- A. **Incorrect:** would not cause DEV-LO alarm – not controlled by the master level output.  
**Plausible:** would cause all other indications
- B. **Incorrect:** charging flow decreases and heaters would trip off  
**Plausible:** will cause other symptoms.
- C. **Correct:** Heaters on due to current NCS pressure.
- D. **Incorrect:** heaters would trip off  
**Plausible:** will cause other symptoms

Level: RO&SRO

KA: APE 028AA1.02(3.4/3.4)

Lesson Plan Objective: ILE LPSO 6

Source: Mod Ques\_104

Level of knowledge: comprehension

References:

1. OP-CN-PS-ILE page 15-17, 21

**Bank Question: 124****Answer: B**

1 Pt(s) A team of workers must repack the seals on a pump in a 1500 mrem/hr high radiation area.

Which one of the following work teams and estimated repair times would maintain worker exposure ALARA?

- A. 10 people working for 20 minutes
- B. 6 people working for 30 minutes
- C. 4 people working for 1 hour
- D. 2 people working for 2 hours

---

**Distracter Analysis:**

- A. **Incorrect:** six people can accomplish the job with 4 1/2 Rem.  
**Plausible:** Each individual would have the least exposure.
- B. **Correct:**
- C. **Incorrect:** six people can accomplish the job with 4 1/2 Rem.  
**Plausible:** fewest individuals not exceeding the admin dose limit.
- D. **Incorrect:** six people can accomplish the job with 4 1/2 Rem.  
**Plausible:** Exposes the fewest individuals.

Level: RO&SRO

KA: G2.3.2 (2.5/2.9)

Lesson Plan Objective: HP LPRO 10

Source: Mod Ques\_124e

Level of knowledge: comprehension

References:

1. OP-CN-RAD-HP page 19

**Bank Question: 162a****Answer: B**

1 Pt(s) Unit 1 is purging containment while in mode 5.

Which one of the following instruments will prevent the release of radioactivity outside containment by completing the corresponding sequence of actions?

- A. EMF-36 (UNIT VENT GAS) will secure VP and VQ, and stops any waste gas release in progress.
- B. EMF-39 (CONTAINMENT GAS) will sound the containment evacuation alarm, secure VP and initiate containment ventilation isolation.
- C. EMF-40 (CONTAINMENT IODINE) will initiate containment ventilation isolation, and shutoff containment sump pump and ventilation drain headers.
- D. EMF-53A/B (CONTAINMENT TRN A(B) HI RANGE) will secure VP and VQ, and shutoff containment sump pump and ventilation drain headers.

---

**Distracter Analysis:**

- A. **Incorrect:** EMF-36 does not secure VP.  
**Plausible:** EMF-36 monitors the final VP release and secures the others.
- B. **Correct answer**
- C. **Incorrect:** EMF-40 does not isolate containment drains.  
**Plausible:** EMF-40 will isolate the VP release.
- D. **Incorrect:** EMF-53 will not secure VP.  
**Plausible:** EMF-53 is a containment radiation monitor, and isolates the containment drains.

Level: RO&SRO

KA: SYS 029A3.01(3.8/4.0)

Lesson Plan Objective: EMF LPRO 2

Source: Mod Ques\_162

Level of knowledge: memory

References:

1. OP-CN-WE-EMF pages 17-18

**Bank Question: 185b****Answer: C**

1 Pt(s) Unit 1 was operating at 100% power when a steam line rupture occurred inside containment. The operators responded by entering:

- E-0 (*Reactor Trip or Safety Injection*) followed by
- E-2 (*Faulted Steam Generator Isolation*) where they isolated the rupture and then transitioned to
- E-1 (*Loss of Reactor or Secondary Coolant*).

Given and the following conditions at the following times:

Time	0200	0205	0210	0215
Subcooling [°F]	+9	+6	+5	+2
S/G A (NR) [%]	5	7	9	11
S/G B (NR) [%]	8	10	16	20
S/G C (NR) [%]	0	0	0	0
S/G D (NR) [%]	9	15	21	30
Feed Flow A S/G [GPM]	135	135	145	130
Feed Flow B S/G [GPM]	150	130	160	160
Feed Flow C S/G [GPM]	0	0	0	0
Feed Flow D S/G [GPM]	170	180	150	155
NC pressure [psig]	1710	1725	1750	1765
Pzr Level [%]	15	18	21	25
Containment pressure [psig]	3.2	2.9	2.5	2.4

At 0200, the operators are at step 12 of E-1.

What is the earliest time that the operators can transition to ES-1.1 (*Safety Injection Termination*)?

**REFERENCES PROVIDED**

*E-1 page 5, Steam Tables*

- A. 0200
- B. 0205
- C. 0210
- D. 0215

---

**Distracter Analysis:**

- A. **Incorrect:** PZR level does not meet the ACC of 20%.



- Plausible:** If the candidate uses non-ACC value of 11%.
- B. Incorrect:** PZR and SG levels do not meet the ACC of 20% and 29%  
**Plausible:** If the candidate uses non-ACC values due to Cont Press <3.0
- C. Correct:** AFW flow >450gpm and PZR level and subcooling above limits.
- D. Incorrect:** Meets the criteria for termination later in time.  
**Plausible:** Based on miscalculating the AFW flow for C.

Level: RO&SRO

KA: WE 02AA2.1 (3.3/4.2)

Lesson Plan Objective: EP2 LPRO 8, 9

Source: Bank 185

Level of knowledge: analysis

References:

1. OP-CN-EP-EP2 page 8
2. E-1 page 5 - PROVIDED
3. OMP 1-7 page 7

**Bank Question: 189a****Answer: C**

- 1 Pt(s) Unit 1 was operating at 100% power when a toxic gas accident caused the operators to evacuate the control room and take control at the Auxiliary Shutdown Panel (ASP)

Given the following conditions:

- When the reactor tripped, 2 control rod bottom lights failed to light
- Pressurizer pressure stabilized at 2235 psig
- Safety injection has not occurred
- $T_{\text{cold}} = 500^{\circ}\text{F}$
- Core burn up = 90 EFPD
- Boron concentration = 1200 ppm
- The OAC is out of service

How much boric acid must be added in accordance with AP/1/A/5500/17 (Loss of Control Room) to maintain shutdown margin?

**REFERENCES PROVIDED**

*Reactor Operating Databook, Sections 4.1 & 4.2*  
*AP/1/A/5500/17 (Loss of Control Room)*

- A. 2850±28 gallons
- B. 15460±154 gallons
- C. 20550±205 gallons
- D. 24025±240 gallons

---

**Distracter Analysis:** AP/17 Enclosure 7, step 1.b requires boration to 2850 ppm. This will require 20553 gallons of boric acid.

- A. **Incorrect:** 20553 gallons of boric acid required  
**Plausible:** if the candidate misreads the need add 2850gal instead of getting to 2850 graph
- B. **Incorrect:** 20553 gallons of boric acid required  
**Plausible:** if the candidate does not calculate the addition properly (various plausible errors).
- C. **Correct Answer:** borate to 2850 ppm – which requires 20553 gallons to go from 1200 ppm to 2850 ppm
- D. **Incorrect:** 20553 gallons of boric acid required  
**Plausible:** if the candidate uses the wrong table, or other miscalculations.

Level: RO&SRO

KA: G2.2.34 (2.8/3.2)

Lesson Plan Objective: RT-RB LPRO 4

Source: Mod Ques\_189

Level of knowledge: analysis

References:

1. OP-CN-RT-RB page 9, 10
2. AP/1/A/5500/17 - PROVIDED
3. Reactor Operating Databook Sections 4.1 & 4.2 - PROVIDED

**Bank Question: 190a****Answer: A**

1 Pt(s) Unit 1 is in a refueling outage with the following plant conditions:

- NC system is in mid-loop
- ND is in operation
- Refueling has been completed

If there are no other exceptions to the containment closure verification, which one of the following statements correctly describes the required status of the containment equipment hatch?

- A. **Must be kept closed at all times while the NC system is in mid-loop.**
- B. **May be open provided a Watch is stationed to immediately close the hatch in the event of a loss of ND.**
- C. **Must be fastened by at least 4 bolts in mode 6.**
- D. **May be open provided contingency plans are in place and equipment is staged to close the hatch prior to core boiling in the event of a loss of ND.**

---

**Distracter Analysis:**

- A. **Correct:**
- B. **Incorrect:** it takes more than one person to close this hatch  
**Plausible:** based on misconstruing equipment vs. personnel hatches
- C. **Incorrect:** the equipment hatch may be open in mode 6 if core alterations are not in progress  
**Plausible:** this is the TS requirement for core alterations in mode 6.
- D. **Incorrect:** exceptions may NOT be authorized in mid-loop per SD 3.1.30.  
**Plausible:** if the candidate misunderstands the containment closure program.

Level: RO Only

KA: APE 069AK2.03(2.8 / 2.9)

Lesson Plan Objective: CNT-CNT SEQ 23

Source: Mod Ques\_190

Level of knowledge: memory

References:

1. OP-CN-CNT-CNT page 22
2. Site Directive 3.1.30 page 19 - 20

**Bank Question: 193a****Answer: A**

1 Pt(s) E-3 (Steam Generator Tube Rupture), enclosure 5 (NC Pressure and Makeup Control to Minimize Leakage) directs operators to energize pressurizer heaters if the ruptured S/G level is decreasing and pressurizer level is greater than 25%. What is the basis for this action in E-3?

- A. Maintain pressurizer saturation temperature equal to ruptured S/G pressure.
- B. Maintain pressurizer saturation temperature below the S/G PORV setpoint.
- C. Maintain pressurizer saturation temperature above ruptured S/G pressure.
- D. Maintain NCS pressure above ruptured S/G pressure.

---

**Distracter Analysis:** The purpose of this question is to determine if the candidate understands that thermal hydraulic equilibrium that is established between the NCS and the ruptured S/G. No references are provided because the candidate should be able to answer the question by simply comprehending the pressures and reasons for this equilibrium.

- A. **Correct Answer:**
- B. **Incorrect:** energizing heaters will not reduce (or hold down) pressure.  
**Plausible:** it is another requirement to maintain NCS pressure below the PORV setpoint.
- C. **Incorrect:** required to maintain NCS pressure equal to ruptured S/G pressure  
**Plausible:** condition will address decreasing S/G level but overcompensate.
- D. **Incorrect:** required to maintain NCS pressure equal to ruptured S/G pressure.  
**Plausible:** condition will address decreasing S/G level but overcompensate.

Level: RO&SRO

KA: EPE 038EA1.38(3.3/3.3)

Lesson Plan Objective: EP-EP4 Obj: 7

Source: Bank McGuire Exam 1997

Level of knowledge: comprehension

References:

1. OP-CN-EP-EP4 page 8
2. E-3 page 61
4. ERG Background document E-3 pages 48-49

**Bank Question: 263****Answer: A**

1 Pt(s)

Unit 1 is shutdown in mode 6 with fuel movement in progress. Given the following events and conditions:

- The new fuel elevator fails to operate in the up direction

Which one of the following statements describes the cause of this problem?

- A. **1EMF-15 (SPENT FUEL BLDG REFUEL BRIDGE) has failed high.**
- B. **1EMF-20 (NEW FUEL STOR 1A) has failed high.**
- C. **The load in the new fuel elevator weighs 1100 lbs.**
- D. **The spent fuel bridge crane is NOT indexed over the new fuel elevator.**

---

**Distracter Analysis:**

- A. **Correct answer**
- B. **Incorrect:** does not have an interlock with the new fuel elevator  
**Plausible:** new fuel vault monitor sounds like it "fits" with new fuel monitor if candidate does not know answer
- C. **Incorrect:** If load exceeds 1200 lbs., will prevent movement  
**Plausible:** this is a valid interlock but the weight is insufficient to actuate it
- D. **Incorrect:** there is no interlock to prevent moving the new fuel elevator  
**Plausible:** there is an interlock to prevent moving the spent fuel pool crane

Level: RO&SRO

KA: SYS 072G2.28 (2.6/3.5)

Lesson Plan Objective: FHS LPRO 8

Source: NRC Catawba Exam 1999 Ques\_263

Level of knowledge: memory

References:

1. OP-1/B/6100/010Z C/5



**Bank Question: 282a****Answer: B**

1 Pt(s) Which of the following describes the plant response to decreasing VI system pressure?

- A. • 86 psig - Standby Compressor starts
  - 80 psig - VI-78 (VS AUTO BACKUP TO VI) opens.
  - 76 psig - VI-500 (VI COMPRESSOR D TO VS HEADER BACKPRESSURE CONTROL) closes.
- B. • 86 psig - Standby Compressor starts
  - 80 psig - VI-500 closes.
  - 76 psig - VI-78 opens.
- C. • 86 psig- Backup Temporary/ Diesel VI Compressor starts
  - 80 psig - VI-500 closes.
  - 76 psig - VI-78 opens.
- D. • 86 psig- Backup Temporary/ Diesel VI Compressor starts
  - 80 psig - VI-78 opens.
  - 76 psig - VI-500 closes

---

**Distracter Analysis:**

Loss of VI (Obj. #5, 8)

Automatic actions:

- 86 psig - Standby Compressor starts
- 80 psig - 'LO VI PRESS' Alarm in Control Room.
- 80 psig – VI 670 'VI Dryer Auto Bypass' opens
- 80 psig - VI500 'VI supply to VS' closes. (Tag label in answers)
- 76 psig - VS78 'VS supply to VI' opens - VS provides instrument air via oil removal filters. (Tag label in answers)

A. **Incorrect:** VI 500 and VI 78 actions are in reverse order

**Plausible:** If candidate reverses the order

B. **Correct:**

C. **Incorrect:**

**Plausible:** backup compressor does not automatically start

D. **Incorrect:**

**Plausible:** backup compressor does not automatically start

Level: RO&SRO

KA: SYS 079A2.01(2.9/3.2)

Lesson Plan Objective: VI SEQ 5, 8

Source: NRC Catawba Exam 1999 Ques\_282

Level of knowledge: memory

References:

1. VI lesson plan page 19 of 36

**Bank Question: 300b****Answer: A**

1 Pt(s) Unit 1 was operating at 100% power when a 5 gpm S/G tube leak occurred in the B S/G. Given the following events and conditions:

- The operators implement AP/10 (REACTOR COOLANT LEAK).
- The steam supply to the turbine driven CA pump must be isolated

Which one of the following statements describes the correct method for isolating B S/G steam supply to the turbine drive CA pump?

- A. **Manually close the isolation valve (1SA-1) in the doghouse.**
- B. **Manually close the stop-check valve (1SA-3) in the doghouse.**
- C. **Manually close the isolation valve (1SA-4) in the mechanical penetration room.**
- D. **Manually close the stop-check valve (1SA-6) in the mechanical penetration room.**

---

**Distracter Analysis:**

- A. **Correct:**
- B. **Incorrect:** wrong location – 1SA-3 is located in the aux building mechanical penetration area - not the doghouse.  
**Plausible:** 1SA-3 is the RNO action if 1SA-1 does not close.
- C. **Incorrect:** 1SA-4 isolates the steam supply from the C S/G - not located in the mechanical penetration area.  
**Plausible:** if the candidate does not recognize the wrong valve
- D. **Incorrect:** will not isolate steam to the B S/G  
**Plausible:** 1SA-6 is the RNO action if 1SA-4 does not close. Valve is located in the mechanical penetration area.

Level: RO&SRO

KA: APE 037G2.4.34 (3.8/3.6)

Lesson Plan Objective: none

Source: Mod Catawba Exam 1997 Ques\_300

Level of knowledge: memory

References:

1. AP-10 page 21
1. OP-CN-STM-SM page 13, 18

**Bank Question: 311****Answer: A**

1 Pt(s) Unit 1 is operating at 50% power. Given the following conditions:

- Pressurizer pressure is 2235 psig
- Pressurizer Relief Tank (PRT) pressure is 21 psig
- PRT temperature is 125 °F
- PRT level is 81%
- A pressurizer code safety valve is suspected of leaking by it's seat

What temperature would be indicated on the associated safety valve discharge RTD if the code safety were leaking by?

**REFERENCES PROVIDED: Steam Tables**

- A. 258-262 °F
- B. 227-231 °F
- C. 161-165 °F
- D. 123 -127°F

---

**Distracter Analysis:**

- A. **Correct answer**
- B. **Incorrect:** Temp is too low - the correct temp is 260 °F  
**Plausible:** If the candidate makes the mistake of not correcting for atmospheric pressure by failing to adding 14.6 psi to the PRT pressure and uses 20 psia.
- C. **Incorrect:** Temp is too low - the correct temp is 260 °F  
**Plausible:** If the candidate reverses the correction for atmospheric pressure by subtracting 14.6 psi from PRT pressure of 20 psig to get 5 psia.
- D. **Incorrect:** Temp is too low - the correct temp is 260 °F  
**Plausible:** If the candidate thinks that the discharge temperature will be at the same temperature as the PRT fluid.

Level: RO&SRO

KA: APE 008AK1.01(3.2/3.7)

Lesson Plan Objective: FLO SEQ 8

Source: NRC Catawba Exam 1999 Ques\_311

Level of knowledge: analysis

References:

1. OP-CN-THF-FLO pages 17-18
2. Steam Tables - PROVIDED

**Bank Question: 339a****Answer: D**

1 Pt(s)

E-3, (Steam Generator Tube Rupture), step #21.b reads as follows:

**"IF AT ANY TIME ruptured S/G(s) pressure is decreasing...,  
THEN perform Step 21."**

Which one of the following statements is correct with regards to this step?

- A. The step is applicable continuously unless it is determined not to be pertinent to the recovery effort.
- B. The step is applicable while in E-3 and after transition to subsequent procedures until alternative guidance is provided.
- C. The step is only applicable until another continuous action step is reached in E-3.
- D. The step is only applicable while in E-3.

---

**Distracter Analysis:**

- A. **Incorrect:** step applicable only in E-3  
**Plausible:** this is the construct for When ... then actions
- B. **Incorrect:** step applicable only in E-3  
**Plausible:** this is an alternate construct for If at any time actions
- C. **Incorrect:** step applicable only in E-3  
**Plausible:** this is the construct for generalized continuous action steps.
- D. **Correct:**

Level: RO&amp;SRO

KA: G2.4.19 (2.7 / 3.7)

Lesson Plan Objective: ADM-OP SEQ 21

Source: NRC Catawba Exam 97 Ques\_339

Level of knowledge: memory

**References:**

- 1. OP-CN-ADM-OP page 10
- 2. OMP 1-7 page 7
- 3. EP/1/A/5000/E-3 page 22

**Bank Question: 343****Answer: B**

1 Pt(s) Unit 1 is responding to a LOCA inside containment. Which one of the following situations describes the case where the operators should use ES-1.2 (Post LOCA Cooldown and Depressurization) to bring the plant into cold shutdown?

- A. A small break LOCA with a loss of containment sump level due to a ruptured FWST.
- B. A small break LOCA where NC system pressure is above shutoff head for the ND pumps.
- C. A large break LOCA where NC system pressure is below shutoff head for the ND pumps.
- D. A large break LOCA with a loss of both NI pumps.

---

**Distracter Analysis:**

- A. **Incorrect:** would use ECA-1.1 (Loss of Emergency Coolant Recirc)  
**Plausible:** if the candidates do not recognize that a loss of containment sump level causes a loss of recirc capability
- B. **Correct Answer:** ES-1.2 is used for small or intermediate break LOCAs where the plant can be repressurized
- C. **Incorrect:** remain in E-1  
**Plausible:** if the candidates do not understand the major actions categories for E-1
- D. **Incorrect:** ES-1.2 is entered from ES-1.1 when NC pressure is less than shutoff head for NI pumps – requires NI pumps to be injecting  
**Plausible:** if the candidates do not recall that they remain in E-1 for large break LOCAs

Level: RO Only

KA: WE 03EK3.4(3.5 / 3.9)

Lesson Plan Objective: EP-EP2 SEQ 3

Source: NRC Catawba Exam 97, Ques\_343

Level of knowledge: memory

References:

1. OP-CN-EP-EP2 page 10



**Bank Question: 371****Answer: C**

- 1 Pt(s) In the event of a steamline rupture that cannot be isolated, FR-P.2 (Response to Anticipated Pressurized Thermal Shock Condition), could be implemented due to an excessive cooldown. What is the limiting component for this PTS event and what is the best indication of the temperature of this component?

	<u>Limiting Component</u>	<u>Best Indication</u>
A.	Steam generator tube sheet	$T_{\text{cold}}$
B.	Pressurizer spray nozzle	$T_{\text{PZR}} - T_{\text{cold}}$
C.	Reactor vessel wall	$T_{\text{cold}}$
D.	Reactor vessel downcomer	$T_{\text{hot}}$

**Distracter Analysis:**

- A. **Incorrect:** - the steam generator tube sheet is not the limiting component for a steam leak event  
**Plausible:** - if the candidate corresponds the steam leak to the steam generator and thinks that this is limiting due to the large pressure difference across the steam generator from the rupture (PTS)
- B. **Incorrect:** - the pressurizer spray nozzle is not limiting for this event as there is no spray flow  
**Plausible:** - if the candidate confuses this event with other events where the pressurizer delta Temp is limiting - or does not recognize that NC pumps have been stopped
- C. **Correct answer**
- D. **Incorrect:** - the reactor vessel downcomer is not generally limiting  
**Plausible:** - if the candidate does not recognize that  $T_{\text{hot}}$  is not the most limiting temperature. The Rx vessel wall and the Rx vessel downcomer are essentially synonymous for the same region.

Level: RO Only

KA: WE 08EK1.1(3.5 / 3.8)

Lesson Plan Objective: TA-PTS SEQ 2, 13

Source: NRC; McGuire Exam 99, Ques\_371

Level of knowledge: memory

References:

1. OP-CN-TA-PTS pages 7, 12-13, 20

**Bank Question: 380****Answer: C**

1 Pt(s)

Which one of the following statements is a correct description of the capabilities of EMF-48 (*Reactor Coolant*) at 100% power?

- A. Detects beta flux from the NC system coolant. This prevents the detector falsely responding to N16 gamma radiation, which would mask a failed fuel event.
- B. Detects the N16 gamma flux from the NC system coolant, which is proportional to the amount of failed fuel cladding.
- C. Detects total gamma flux from NC system coolant after a one minute sample delay time to allow N16 gamma radiation to decay away.
- D. Detects total gamma flux from NC system coolant. The gamma source term from a clad failure would be much greater than the N16 gamma flux at power.

---

**Distracter Analysis:**

- A. **Incorrect:** - EMF-48 does not detect beta radiation  
**Plausible:** - the detector could function if designed this way because N16 gamma would mask the failed fuel problems.
- B. **Incorrect:** - N16 gamma is proportional to reactor power level and does not correlate to failed cladding.  
**Plausible:** - if the candidate was confused over the correlation between N16 gamma levels and power levels.
- C. **Correct answer**
- D. **Incorrect:** - The gamma flux from N16 is >> failed fuel at power  
**Plausible:** - if the candidate did not know that N16 gamma was >> than failed fuel source term levels.

Level: RO&amp;SRO

KA: APE 076AK2.01(2.6/3.0)

Lesson Plan Objective: none

Source: NRC McGuire Exam 1999 Ques\_380

Level of knowledge: memory

References:

1. OP-CN-WE-EMF page 11

**Bank Question: 387a****Answer: B**

1 Pt(s)

Enclosure 1 to E-1 (Loss of Reactor or Secondary Coolant) provides foldout page actions to close NV-202B and NV-203A (NV PUMPS A&B RECIRC ISOL) when NC pressure is less than 1500 psig.

Which one of the following statements correctly describes the operator response and reason for this response when pressurizer pressure is 1495 psig?

- A. Notify the SRO of the need to close the valves to prevent NV pump runout at low pressures.
- B. Close the valves to prevent a reduction of full SI flow to the core.
- C. Close the valves to prevent overheating the NV pumps.
- D. Notify the SRO of the need to close the valves to prevent a reduction of full SI flow to the core.

---

**Distracter Analysis:**

- A. **Incorrect:** - pump runout is not a concern and the actions are automatic.  
**Plausible:** if the candidate does not know that foldout actions are independent - pump runout can be a concern for situations where the pump discharge pressure is very low.
- B. **Correct answer** - maximizes flow into the core.
- C. **Incorrect:** - pump runout is not a concern.  
**Plausible:** pump runout can be a concern for situations where the pump discharge pressure is very low, approximately 600 psig for NV pumps.
- D. **Incorrect:** - the actions are automatic.  
**Plausible:** - if the candidate does not know that foldout actions are independent.

Level: RO&amp;SRO

KA: G2.4.15 (3.0/3.5)

Lesson Plan Objective: EP-INTRO LPRO 12

Source: Mod McGuire Exam 1999 Ques\_387

Level of knowledge: memory

References:

1. OP-CN-EP-INTRO page 8
2. E-1 Foldout page 2
3. OMP 1-7 page 20-21
4. Background Document E-1 page 30

**Bank Question: 388****Answer: A**

1 Pt(s) Unit 1 is responding to a LOCA in the auxiliary building. The operators have implemented ECA-1.2, (*LOCA Outside of Containment*). Step 2 guides the operators to attempt to find and isolate the leak. Step 2C requires the following sequence:

- 1) Verify following NI pump miniflow valves – OPEN
  - 1NI-115A (*NI Pump 1A Miniflow Isol*)
  - 1NI-144A (*NI Pump 1B Miniflow Isol*)
  - 1NI-147B (*NI Pump 1C Miniflow Isol*)
- 2) Place the “PWR DISCON FOR 1NI-162A” in “ENABLE”
- 3) Close 1NI-162A (*NI To C- Legs Inj Hdr Isol*)

What is the reason for verifying the mini flow valves are open?

- A. **Protect the NI pumps from operating against shutoff head.**
- B. **Isolation of a potential LOCA path to the FWST.**
- C. **Protect the NI pumps from runout conditions upon restart.**
- D. **Provide a diversion path to prevent high pressure water from over-pressurizing the ND system during leak isolation procedures.**

---

**Distracter Analysis:**

- A. **Correct answer**
- B. **Incorrect:** - the mini flow valves will not isolate any potential path in the injection mode.  
**Plausible:** - in ES-1.3 (and some other EOPs), the NI mini flow valves are closed to prevent pumps from recirculating radioactive water back to the FWST - right reason, wrong procedure
- C. **Incorrect:** - Opening mini flow valves will not protect against runout  
**Plausible:** - if the candidate is confused over the difference between runout conditions and shutoff head conditions
- D. **Incorrect:** - this will not create a diversion path - it will align the NI recirc line to the FWST. The ND system is protected by check valves and in this case by the closed 1NI-162A.  
**Plausible:** - the ND system has a design pressure of 600 psig and if NC system pressure was applied, it would rupture. Setting up a

diversion path would be a reasonable thing to do. This alignment does not accomplish this goal.

Level: RO&SRO

KA: WE04EK2.1(3.5/3.9)

Lesson Plan Objective: EP-E2 SEQ 14

Source: NRC McGuire Exam 1999 Ques\_388

Level of knowledge: comprehension

**References:**

1. OP-CN-EP-E2 pages 13-14
2. ECA-1.2 page 6
3. OP-CN-CN-NI pages 7-8, 18

**Bank Question: 393****Answer: C**

1 Pt(s)

Unit 1 was in mode 3 with the shutdown banks fully withdrawn, preparing to conduct a reactor startup when source range channel N-31 failed. Given the following conditions and events:

- No reactor trip has occurred prior to this point
- AP/16 (*Malfunction of Nuclear Instrumentation System*) has been completed
- N-31 repairs have been made
- N-31 is being returned to service
- N-32 =  $10^1$  CPS
- Immediately upon taking the "level trip" switch to the "normal" position a reactor trip occurred

Which of the following operator errors would explain this event?

- A. The "Operation selector" switch was left in "level adj." position with level potentiometer set at a level of  $10^6$  CPS after retesting
- B. The "High-flux at shutdown" switch left in the "normal" position
- C. The instrument power fuse blew when the N-31 channel was reenergized.
- D. The source range detector instrument discriminator voltage was set too high.

---

**Distracter Analysis:**

- A. **Incorrect:** - the *operation selector* switch is taken out of the circuit when the *level trip* switch is taken to *normal*  
**Plausible:** - if the candidate thinks that a test signal can be inserted with *level trip* switch in the *normal* position
- B. **Incorrect:** - no effect – this is a normal switch alignment - only blocks out high flux alarm at  $10^5$  CPS - trip is separate from level trip function  
**Plausible:** - if the candidate does not understand that the *level trip* switch in *bypass* does not effect the high flux at shutdown trip – the high flux at shutdown switch will be in the blocked position under these circumstances.
- C. **Correct answer**
- D. **Incorrect:** - if discriminator voltage is too high, less neutrons will be passed, signal will be lower.



**Plausible:** - if the candidate does not understand how the pulse height discriminator circuit operates.

Level: RO&SRO

KA: APE 032AA.101(3.1/3.4)

Lesson Plan Objective: ENB LPRO 9

Source: NRC McGuire Exam 1999 Ques\_393

Level of knowledge: comprehension

References:

1. OP-CN-IC-ENB page 5, 9-10, 26

**Bank Question: 437a****Answer: B**

1 Pt(s)

The operator is investigating a suspected ground on the negative leg of a 125VDC bus. The Battery to Ground Volt Meter Selector Switch is in the "NEG" position.

Which one of the following indications is correct for the existence of a substantial ground on the negative leg of the 125 VDC electrical system?

- A. **Battery ground negative leg light burns dimly  
Battery to Ground Volt Meter reads bus voltage minus the ground voltage.**
- B. **Battery ground negative leg light burns brightly  
Battery to Ground Volt Meter reads the ground voltage.**
- C. **Battery ground negative leg light burns dimly.  
Battery to Ground Volt Meter reads the ground voltage.**
- D. **Battery ground negative leg light burns brightly  
Battery to Ground Volt Meter reads bus voltage minus the ground voltage.**

---

**Distracter Analysis:**

- A. **Incorrect:** - a brightly burning lamp = grounded condition  
**Plausible:** - believes dim light = ground
- B. **Correct:** - the negative leg light is brightly lit.  
If a ground exists, the Volt Meter will indicate the amount of volts to ground when this switch is taken to the NEG (measures volts to ground on the negative leg)
- C. **Incorrect Answer:**  
**Plausible** if candidate reverses the correct answer in his/her mind - a grounded condition often leads to a reduction in voltage and lamps glow dimly
- D. **Incorrect:**  
**Plausible:** - candidate believes voltmeter reads bus voltage, less the ground.

Level: RO&amp;SRO

KA: SYS 063A3.01 (2.7/3.1)

Lesson Plan Objective: EPL SEQ 15, 16

Source: Mod - McGuire Exam 1999 Ques\_437

Level of knowledge: memory

References:

1. Lesson plan pages 16 and 17

**Bank Question: 439****Answer: B**

1 Pt(s) What automatic actions will occur if radioactive particulate levels exceed the 1EMF-35(L) (*Unit Vent PART HI RAD*) trip 2 alarm set points in the unit vent exhaust flow stream?

- A. Stops containment purge (VP) supply fans
- B. Stops unit-related unfiltered exhaust (VA) fans
- C. Stops spent fuel pool ventilation exhaust (VF) fans
- D. Stops containment annulus ventilation (VE) fans

---

**Distracter Analysis:**

- A. **Incorrect:** - does not stop VP supply fans  
**Plausible:** - trip 2 alarm on EMF-39 (L) (containment gas monitor) causes this action
- B. **Correct answer** - 1EMF-35(L) monitors this exhaust stream
- C. **Incorrect:** - does not stop these fans – shifts VF to filter mode – VF exhaust fans continue to run but filter train is shifted into the exhaust line.  
**Plausible:** - EMF-35 monitors exhaust from Spent Fuel Pool area
- D. **Incorrect:** - does not stop VE fans  
**Plausible:** - monitors VE exhaust

Level: RO Only

KA: SYS 073A1.01(3.2/3.5)

Lesson Plan Objective: WL-EMF LPRO 2

Source: NRC McGuire Exam 1999 Ques\_439

Level of knowledge: memory

References:

1. OP-CN-WL-EMF pages 9

**Bank Question: 444a****Answer: B**

1 Pt(s)

Which one of the following statements describes the operation of the Containment Annulus Ventilation System (VE) during a large break LOCA into containment?

- A. **VE fans start in the exhaust mode at +3.0 psig in containment  
VE fans stop running when annulus pressure reaches -1.5 in. H<sub>2</sub>O  
VE fans cycle on and off between -1.5 in H<sub>2</sub>O and +3 psig**
- B. **VE fans start in the exhaust mode at +1.2 psig in containment  
VE shifts into recirc mode when annulus pressure reaches -1.5 in. H<sub>2</sub>O  
VE modulates dampers between recirc and exhaust modes to maintain -1.5 in. H<sub>2</sub>O in the annulus**
- C. **VE fans start at on an EMF-38, 39, 40 (containment monitors)  
VE fans stop running when annulus pressure reaches -1.5 in. H<sub>2</sub>O  
VE fans cycle on and off between -1.5 in Hg and +3.0 psig**
- D. **VE fans start at on an EMF-38, 39, 40 (containment monitors)  
VE shifts into recirc mode when annulus pressure reaches -1.5 in. H<sub>2</sub>O  
VE modulates dampers between recirc and exhaust modes to maintain -1.5 in. H<sub>2</sub>O in the annulus**

---

**Distracter Analysis:**

- A. **Incorrect:** - VE Fans are not designed to cycle on and off to maintain annulus pressure. They would not recirc through the filter trains if they tripped off to maintain pressure and Iodine removal would not be as effective  
**Plausible:** - tripping at 3 psig would be consistent with the initiation point. This is a plausible safety trip to ensure annulus pressure is not dropped too low
- B. **Correct answer**
- C. **Incorrect:** - EMF monitors do not start VE fans; VE Fans are not designed to cycle on and off to maintain annulus  
**Plausible:** - VP and VQ systems are controlled by EMF 38, 39, 40
- D. **Incorrect:** - EMF monitors do not start VE fans  
**Plausible:** - EMF 38, 39, 40, controls VP and VQ systems

Level: RO&amp;SRO

KA: SYS 027A4.03(3.3/3.2)

Lesson Plan Objective: VE LPRO 9

Source: NRC McGuire Exam 1999 Ques\_444

Level of knowledge: memory

References:

1. OP-CN-CNT-VE pages 8-9

**Bank Question: 453a****Answer: C**

1 Pt(s)

Unit 1 was operating at 70% power when a loss of condenser vacuum occurred. Given the following events and conditions:

- Reactor power, 68%
- Turbine load, 66% based on turbine impulse pressure
- Turbine exhaust hood temperature is 225 °F
- The operators are rapidly decreasing turbine load
- The operator reports that condenser vacuum is 23.2 in Hg and is continuing to decrease.

Which one of the following statements correctly describes the required action(s)?

- A. **Immediately manually trip the reactor.**
- B. **Immediately manually trip the reactor and then manually trip the turbine in anticipation of reaching the trip setpoint.**
- C. **Continue to monitor condenser vacuum, if vacuum decreases to 21.8 inches Hg, manually trip turbine.**
- D. **Continue to monitor condenser vacuum, if vacuum decreases to 21.8 inches Hg, first manually trip the reactor then manually trip the turbine.**

---

**Distracter Analysis:**

Turbine Trip Criteria: IF condenser vacuum decreases to less than 21.8 in. Hg, THEN:

- a. IF reactor power is greater than or equal to 69%, THEN manually trip reactor.
- b. Ensure turbine - TRIPPED.
- c. IF reactor is tripped, THEN GO TO EP/1/A/5000/E-0 (Reactor Trip Or Safety Injection). –
- d. REFER TO AP/1/A/5500/02 (Turbine Generator Trip).

IF turbine exhaust hood temperature is greater than 2250 F AND turbine load is less than 60% ~based on impulse pressure equal to 370 PSIG), THEN:

- a. Ensure turbine - TRIPPED.

- A. **Incorrect:** Tripping the reactor is not a required action below 69%  
**Plausible:** If unfamiliar with the 69% reactor trip criteria.
- B. **Incorrect:** Do not trip the reactor before tripping the turbine

**Plausible:** tripping the turbine is not **required** until the set point is reached – but is **allowed** by OMP 1-8 - if the candidate does not recognize that power is below 65%

**C. Correct:**

**D. Incorrect:** should not immediately trip the reactor.

**Plausible:** If the candidate does not recognize that power is below 65%

Level: RO&SRO

KA: APE 051 AA2.02 (3.9/4.1)

Lesson Plan Objective: ZM SEQ 17

Source: mod Catawba Exam 1999 Ques\_453

Level of knowledge: memory

References:

1. AP/1/5500/23 Loss of Condenser Vacuum
2. OMP 1-8 page 3



**Bank Question: 480a****Answer: D**

1 Pt(s)

Unit 1 is operating at 100% power preparing for a reactor startup. Given the following events and conditions:

- Isotopic analysis of S/G samples at 1200 hours on 3/19 indicated the following primary to secondary leak rates:
  - S/G A = 0.097 gpm
  - S/G B = 0.11 gpm
  - S/G C = 0.08 gpm
  - S/G D = 0.103 gpm
- 1EMF-71-74 (S/G A-D Leakage HI RAD) has not changed over the last 72 hours.

Which one of the following statements correctly describes the actions required (if any) based on these values?

**REFERENCES PROVIDED - Tech Spec 3.4.13**

- A. No action required.
- B. Isolate S/G B by 1600 on 3/19
- C. Be in mode 3 by 1800 on 3/19
- D. Be in mode 5 by 0400 on 3/21

**Distracter Analysis:**

Leakage from the B S/G =  $0.11 \text{ gpm} \times 60 \text{ min/hr} \times 24 \text{ hr/day} = 158.4 \text{ gpd}$

Combined leakage from all 4 S/Gs

S/G A =  $0.097 \text{ gpm} \times 60 \text{ min/hr} \times 24 \text{ hr/day} = 139.7 \text{ gpd}$

S/G B =  $0.11 \text{ gpm} \times 60 \text{ min/hr} \times 24 \text{ hr/day} = 158.4 \text{ gpd}$

S/G C =  $0.08 \text{ gpm} \times 60 \text{ min/hr} \times 24 \text{ hr/day} = 115.2 \text{ gpd}$

S/G D =  $0.103 \text{ gpm} \times 60 \text{ min/hr} \times 24 \text{ hr/day} = 148.3 \text{ gpd}$

**561.6 gpd total leakage**

Each S/G must be  $< 150 \text{ gpd}$  and (Tech Spec 3.4.13)

Combined leakage through all 4 S/Gs  $< 576 \text{ gpd}$  (Tech Spec 3.4.13)

- A. **Incorrect:** must shutdown within 10 hours  
**Plausible:** if candidate miscalculates S/G leakage
- B. **Incorrect:** isolation of S/G B in 4 hrs will not stop the leak – can't isolate a S/G U-tube leak.  
**Plausible:** action statement A allows 4 hours to isolate the leakage – if the candidate does not understand that the leak cannot be isolated.
- C. **Incorrect:** must be in mode 3 by 2200

**Plausible:** action statement B (6 hours) to mode 3 if disregard the 4 hrs available to stop the leak before reducing power.

**D. Correct answer:** 4 hours to isolate – then 36 hours to be in mode 5 – 40 hours from 1200 3/19 = 0400 on 3/21

Level: RO Only

KA: SYS 002A2.01 (4.3 / 4.6)

Lesson Plan Objective: PS-NC SEQ 10

Source: Mod McGuire Exam 1999 Ques\_480

Level of knowledge: comprehension

References:

1. Tech Spec 3.4.13 - PROVIDED

**Bank Question: 503a****Answer: A**

1 Pt(s)

Unit 2 was operating at 80% power. Given the following events and conditions:

- Turbine impulse pressure instrument Channel I failed low
- Operators perform all actions in AP-15 (*Rod Control Malfunctions*) Case II (*Continuous Rod Movement*).
- Tave is maintained by adjusting turbine load

Which one of the following statements correctly describes the consequences of the Reactor Operator returning the CRD Bank Select switch to the Automatic position 10 minutes later?

- A. Rods will move in because Tref is less than Tave.
- B. Rods will move in because impulse pressure is less than nuclear power.
- C. Rods will not move in because low impulse pressure blocks rod movement.
- D. Rods will not move in because the impulse pressure input to power mismatch is not changing.

---

**Distracter Analysis:**

- A. **Correct:**
- B. **Incorrect:** The derivative of the Imp Press change has timed out.  
**Plausible:** based on misunderstanding the derivative function of the power mismatch circuit.
- C. **Incorrect:** C-5 only blocks rod withdrawal.  
**Plausible:** based on misunderstanding the C-5 rod stop function.
- D. **Incorrect:** The temperature mismatch function has an error signal at this time.  
**Plausible:** If the candidate does not recognize the cumulative nature of both error signals.

Level: RO&SRO

KA: APE 001G2.1.32(3.4/3.8)

Lesson Plan Objective: IC-IRX LPRO 5

Source: Mod Catawba exam 1999

Level of knowledge: analysis

References:

1. OP-CN-IC-IRX pages 9-10, 13-15
2. OP-CN-IC-IPX page 20

**Bank Question: 518****Answer: C**

1 Pt(s)

Which one of the following design features prevents the containment isolation valves inside containment (KC, NC, NI and NM systems) from spuriously opening during a containment flooding event following a containment isolation phase A signal?

- A. The valves are all located above the equipment qualification (EQ) flooding level and should never be submerged by design.
- B. These valves were designed to use air operators, which are not subject to spurious actuation when submerged.
- C. These valves have motor operators that have their OPEN controls disabled and must be manually reset from the control room.
- D. These valves have all power removed from their motor operators and must be manually enabled from the control room.

---

**Distracter Analysis:**

- A. **Incorrect:** Valves located below flooding plane  
**Plausible:** This is one way of preventing the problem
- B. **Incorrect:** Valves have MOVs not AOVs  
**Plausible:** Some plants have mainly AOVs in containment for this reason
- C. **Correct:**
- D. **Incorrect:** Valves do not have closing power removed or this would prevent actuation in response to a valid ESF signal  
**Plausible:** This would prevent the valves from spuriously opening. Some ECCS valves are protected this way.:

Level: RO&amp;SRO

KA: WE15EK2.1(2.8/2.9)

Lesson Plan Objective: CNT LPRO 13

Source: NRC Catawba Exam 1999 Ques\_518

Level of knowledge: memory

References:

1. OP-CN-CNT-CNT page 16

2. OP-CN-EP-FRZ pages 5, 7

**Bank Question: 547****Answer: C**

1 Pt(s) Unit 1 was operating at 100% power when main condenser vacuum dropped from 25 in Hg to 23 in Hg.

Which one of the following statements correctly describes the cause of this problem?

- A. Condenser water boxes are full
- B. RC system flow has increased
- C. CM flow to CSAE inter-cooler has been obstructed
- D. Condensate depression has increased.

---

**Distracter Analysis:**

- A. **Incorrect:** Will not reduce condenser vacuum - full is a normal condition for the water boxes.  
**Plausible:** If the ZP water boxes are NOT full, air can accumulate in the upper tubes and interfere with the heat transfer across the tubes
- B. **Incorrect:** Increased RC flow will improve vacuum  
**Plausible:** Decreased RC flow will degrade vacuum
- C. **Correct answer** - reduces effectiveness of steam jets
- D. **Incorrect:** - if condensate depression increases, the condensate temperature becomes lower than saturation temperature for the condenser pressure - vacuum would increase  
**Plausible:** Condenser thermodynamic efficiency decreases

Level: RO&SRO

KA: SYS 055K3.01 (2.5 / 2.7)

Lesson Plan Objective: MT-ZM SEQ 9, 14

Source: NRC Catawba Exam 99 Ques\_547

Level of knowledge: comprehension

References:

1. OP-CN-MT-ZM pages 5-8

**Bank Question: 556****Answer: D**

1 Pt(s)

Unit 2 is operating at 100% power when containment parameters vary as shown below:

<b>Containment</b>	<b>1200</b>	<b>1500</b>	<b>1800</b>	<b>2100</b>	<b>2400</b>
Temperature (°F)					
Upper	85	86	87	86	85
Lower	105	107	106	105	106
Humidity (% rel)					
Upper	25	26	25	26	25
Lower	15	15	18	14	15
Containment pressure (psig)	0.11	0.13	0.15	0.18	0.19
Aux. Bldg. pressure (in Hg)	29.1	29.2	29.3	29.4	29.5

Which one of the following statements correctly describes the cause of the trends in the containment atmospheric parameters?

- A. Normal external heating from the sun.
- B. Auxiliary Building pressure has increased.
- C. A packing leak on the letdown backpressure control valve.
- D. An air leak on the AOV for PORV 2NC-36.

**Distracter Analysis:**

- A. **Incorrect:** pressure not following temperature  
**Plausible:** temperature increases then decreases
- B. **Incorrect:** An increase in Auxiliary Building pressure would cause a drop in containment pressure as the containment pressure instrument is referenced to external Aux. Bldg. pressure  
**Plausible:** pressure is increasing - if the candidate reverses the effect of barometric pressure on containment pressure
- C. **Incorrect:** temperature and humidity are not increasing  
**Plausible:** pressure is increasing but not following temperature - the difference between upper and lower containment is normal.
- D. **Correct answer:** cont. pressure increasing without other cont. parameters increasing

Level: RO&amp;SRO

KA: G2.1.7 (3.7/4.4)



Lesson Plan Objective: VQ LPRO 2

Source: NRC Catawba 1999 Ques\_556

Level of knowledge: analysis

References:

1. OP-CN-CNT-VQ page 6

**Bank Question: 580****Answer: D**

1 Pt(s)

Which one of the following statements correctly describes the biological hazard of Tritium in the liquid RadWaste system?

- A. Tritium emits a low-energy gamma that can cause whole body dose as well as an internal hazard with a 12-year biological half-life if ingested.**
- B. Tritium emits a low-energy alpha that is an internal hazard with a 12-day biological half-life if ingested.**
- C. Tritium emits a low-energy beta that can cause both a skin dose as well as an internal hazard with a 12-day biological half-life if ingested.**
- D. Tritium emits a low-energy beta that is an internal hazard with a 12-day biological half-life if ingested.**

---

**Distracter Analysis:**

- A. Incorrect:** emits low energy beta only, not gamma - radiological half life is 12 years  
**Plausible:** Tritium is an internal hazard with a 12-day biological half-life
- B. Incorrect:** emits low energy beta only - radiological half life is 12 years  
**Plausible:** Tritium is an internal hazard with a 12-day biological half-life
- C. Incorrect:** It is not hazardous to the skin as the beta energy is too low  
**Plausible:** Tritium is an internal hazard with a 12-day biological half-life
- D. Correct answer**

Level: RO Only

KA: SYS 068K5.04 (3.2/3.5)

Lesson Plan Objective: WL LPRO 13

Source: NRC Catawba Exam 1999 Ques\_580

Level of knowledge: memory

References:

1. OP-CN-CH-PC pages 11-13

**Bank Question: 596a****Answer: B**

1 Pt(s) Units 1 and 2 were operating at 100% power when a fire broke out in the back of the control room.

Given the following conditions:

- The fire has not yet affected or degraded any control systems
- Heavy black smoke is throughout the control room
- The SRO implements AP/17 (*Loss of Control Room*)

Which one of the following statements correctly describes the operator response to this event?

- A. **Swap control to the auxiliary shutdown panels, then trip both unit reactors and turbines, and evacuate the control room.**
- B. **Dispatch RO's to the auxiliary shutdown panels, trip both unit reactors, turbines and feed pumps, then evacuate the control room.**
- C. **Evacuate the control room; trip both unit turbines and reactors on the way to the auxiliary shutdown panel.**
- D. **Immediately trip both unit reactors and turbines and evacuate the control room to the standby shutdown facility.**

---

**Distracter Analysis:**

- A. **Incorrect:** control is not swapped until after the CR is evacuated.  
**Plausible:** this is a more controlled approach to CR evacuation?
- B. **Correct:**
- C. **Incorrect:** Reactor and turbines tripped from the CR.  
**Plausible:** This is a reasonable approach assuming operators can no longer function in the CR.
- D. **Incorrect:** Evacuate to the ASP not the SSF  
**Plausible:** if the candidate confuses the SSF with ASP.

Level: RO&SRO

KA: APE 068AK3.12(4.1 / 4.5)

Lesson Plan Objective: CP-RSS SEQ 9

Source: Mod McGuire Exam 2000 Ques\_596

Level of knowledge: memory

References:

1. OP-CN-CP-RSS page 13
2. AP/1/A/5500/17 pages 2-3

**Bank Question: 647a****Answer: D**

1 Pt(s) Unit 1 was operating at 100% power. The following trends were noted:

	<b>0200</b>	<b>0205</b>	<b>0210</b>
1A-1D NCP#1 seal outlet temp (°F)	100	130	145
1A-1D NCP #1 seal leakoff flow (gpm)	3.0	3.5	4.0
1A-1D NCP #1 seal d/p (psid)	>400	>400	>400
1A-1D NCP seal water inj filter d/p (psid)	10	2.0	1.0
1A-1D NCP seal injection (gpm)	8.0	1.5	<1.0
VCT Level	50%	49%	50%
Pressurizer level	55%	56%	55%
Charging Flow (gpm)	87	88	87
Letdown Flow (gpm)	75	75	75

Which one of the following conditions would cause these parameter trends?

- A. #1 seal injection filter clogged
- B. 1NV-294, (NV PMPS A&B DISCH FLOW CTRL), failed closed
- C. The VCT depressurized
- D. 1NV-309, (SEAL WATER INJECTION FLOW), failed open

**Distracter Analysis:** Seal injection filter d/p and seal injection flow are decreasing which indicates loss of seal injection. Seal leakoff is increasing as indicated by leakoff flow and leakoff temp increasing. Pressurizer level stays about the same as seal injection flow is diverted into the NC system – total charging flow remains stable as corrected by pressurizer level control.

- A. **Incorrect:** seal injection filter d/p would increase  
**Plausible:** most other parameters would trend as indicated except seal filter d/p would increase.
- B. **Incorrect:** closing 1NV-294 would stop charging and result in opposite trends in PZR & VCT  
**Plausible:** if the candidate focuses only on NCP seal parameters.
- C. **Incorrect:** VCT depressurization would result in increased seal injection flow.  
**Plausible:** if the candidate focuses only on seal leakoff flow and temperature.
- D. **Correct Answer:** seal injection flow and d/p decreasing

Level: RO&SRO

KA: APE 022AA1.09(3.2/3.3)

Lesson Plan Objective: NCP LPRO 3; NV LPRO 6, 7

Source: Mod McGuire Exam 2000 Ques\_647

Level of knowledge: analysis

References:

1. OP-CN-PS-NV pages 26-27, 32- 34, 59, 61
2. OP-CN-PS-NCP pages 13, 22

**Bank Question: 661a****Answer: B**

1 Pt(s) Units 1 and 2 are at 100% power. Given the following conditions:

- Unit 2 has experienced 2 fuel pin failures.
- The mechanical seal has failed on NI pump 2B.
- The NI-2B pump room general area is 400 mrem/hr.
- In order to reach the NI-2B pump room the worker must transit through a 6 Rem/hr high radiation area for 2 minutes and return.
- The worker has an accumulated annual dose of 400 mrem, respectively.

What is the maximum allowable time that the worker can participate in the seal repair on NI Pump 2B without exceeding the exclusion flag exposure limit for external exposure?

- A. No longer than 2 hours
- B. No longer than 2.5 hours
- C. No longer than 3 hours
- D. No longer than 3.5 hours

---

**Distracter Analysis:**

The candidate should determine that the exclusion flag exposure limit is 90% of 2000 mrem admin limit = 1800 mrem

Transient exposure is 400 mrem (6000mrem/hr x 4/60hr). (During transit to and from the job).

$$400 \text{ mrem} + 400 \text{ mrem} = 800 \text{ mrem}$$

$$1800 \text{ mrem} - 800 \text{ mrem} = 1000 \text{ mrem allowable before reaching exclusion flag exposure admin limit}$$

$$1000 \text{ mrem} / 400 \text{ mrem/hr} = 2.5 \text{ hours}$$

- A. **Incorrect:** The answer is 2.5 hours.  
**Plausible:** based on using alert flag limit (1600) versus exclude flag.
- B. **Correct:**
- C. **Incorrect:** The answer is 2.5 hours.  
**Plausible:** based on calculating a one-way transit dose.
- D. **Incorrect:** The answer is 2.5 hours.  
**Plausible:** based on using admin limit (2000) and a one-way transit dose.



Level: RO&SRO

KA: G2.3.4 (2.5/3.1)

Lesson Plan Objective: HP LPRO 2, 11

Source: Mod McGuire Exam 2000 Ques\_661

Level of knowledge: comprehension

References:

1. NSD 507.6 page 10
2. GET Rad Worker Training pages 33, 34, 45

**Bank Question: 671a****Answer: D**

1 Pt(s) Unit 1 was releasing the contents of a waste gas decay tank in accordance with an approved release permit. 1EMF-50(L) (*Waste Gas Disch*) failed high during the release, and will not be repaired for 3 days.

Which one of the following actions must be taken to release the waste gas decay tank today?

- A. The release cannot be restarted until the repairs on 1EMF-50(L) (*Waste Gas Disch*) are completed.
- B. Recalculate the trip set points using 1EMF-50(H) (*Waste Gas Disch*) as the release path monitor, then restart the release after a new GWR form has been approved.
- C. Continue the release using 1EMF-36(L) (*Unit Vent Gas*) as the backup release path monitor.
- D. Recalculate the trip set points using 1EMF-36(L) (*Unit Vent Gas*) as the release path monitor, then restart the release after a new GWR form has been approved.

---

**Distracter Analysis:**

- A. **Incorrect:** not required to use 1EMF-50(L) as the only qualified release path monitor  
**Plausible:** if the candidate does not recognize that 1EMF-36(L) can be used to monitor the release path
- B. **Incorrect:** 1EMF-50(H) does not automatically trip WG-160 and cannot be used as a waste gas release path monitor  
**Plausible:** if the candidate thinks that substituting the high range of 1EMF-50(L) provides the same automatic protection
- C. **Incorrect:** the release would be terminated when 1EMF-50(L) tripped  
**Plausible:** if the candidate did not recognize that 1EMF-50(L) provided an automatic trip of WG-160 to terminate the release.
- D. **Correct answer**

Level: RO&SRO

KA: G2.3.11 (2.7/3.2)

Lesson Plan Objective: WE-WG LPRO 4

Source: NRC McGuire Exam 2000 Ques\_671

Level of knowledge: memory

References:

1. OP-CN-WE-EMF pages 9, 11
2. OP-CN-WE-WG page 9

**Bank Question: 714****Answer: C**

1 Pt(s)

Unit 1 was in mode 2 preparing for a plant startup when an electrical transient occurred. Given the following events and conditions:

- Steam dumps are in steam pressure control
- Reactor power is 2%
- Tave decreases
- Rods have sequenced normally during the startup.
- Bank D group step counter indicates 200 steps
- DRPI indication for Bank D rods reads 198 steps
- Rod bottom light H-8 is illuminated
- DRPI indication for rod H-8 reads 0 steps
- DRPI indication for rod D-2 reads 192 steps

Which one of the following actions best describes the correct action that should be taken by the crew and the reason for this action?

- A. Trip the plant because a dropped rod below mode one is not an analyzed condition.
- B. Trip the plant because greater than 2 rods are misaligned.
- C. Ensure CRD BANK SELECT switch is selected to MANUAL to prevent rod motion.
- D. Ensure that CRD BANK SELECT switch is in AUTOMATIC to allow Tave to recover to Tref.

---

**Distracter Analysis:**

- A. **Incorrect:** Not required to trip the reactor unless > 2 rods are dropped or misaligned – one dropped rod is an analyzed condition  
**Plausible:** If the candidate thinks that the AP14 case II immediate actions for a single dropped rod only applies to mode 1 conditions
- B. **Incorrect:** Rod H-8 is not  
**Plausible:** Plant trip is the preferred method of shutting the reactor and ensuring the reactor is subcritical.
- C. **Correct:** Immediate action per AP/14 step 2
- D. **Incorrect:** Switch must be in manual  
**Plausible:** AP/14 step 3 requires the operator to match Tave to Tref

Level: RO&amp;SRO

KA: 001G4.22(3.0/4.0)

Lesson Plan Objective: IRE SEQ 20

Source: New

Level of knowledge: memory

References:

1. AP/14 Case II page 5

**Bank Question: 715****Answer: D**

1 Pt(s)

Unit 1 was operating at 30% power. Given the following events and conditions:

- “A” NC pump trips
- No operator action has been taken
- All safety systems operate as designed

While the plant is still at power, which one of the following parameters will initially **INCREASE**?

- A. “A” steam generator level.
- B. Loop cold leg temperatures in the B, C and D loops.
- C. Steam generator pressures in the B, C and D loops.
- D. Steam generator steam flows in the B, C and D loops.

---

**Distracter Analysis:** The reactor will not trip below P-8.

- A. **Incorrect:** A S/G level will decrease  
**Plausible:** when the pump trips, steam flow in the steam generator will decrease, level will “shrink” – the candidate may reverse this cause and effect.
- B. **Incorrect:** Unaffected Tcolds will decrease  
**Plausible:** Steam flow in the unaffected loops will increase, heat removal from the RCS will increase, and cold leg temperature will decrease – the candidate may reverse this cause and effect or become confused with the reverse flow in the A loop
- C. **Incorrect:** Unaffected S/G pressures will decrease  
**Plausible:** Unaffected steam generators will increase steaming, pressure will decrease – the candidate may reverse this cause and effect.
- D. **Correct:** Steam flow in the affected steam generator will decrease, load does not change, the remaining steam generators will increase their steam rates, steam flow in those steam generators will increase.

Level: RO&SRO

KA: SYS003K5.04(3.2/3.5)

Lesson Plan Objective: none

Source: New

Level of knowledge: analysis

References:

1. OP-CN- THF-FF page 15
2. OP-CN-CF-IFE page 6

**Bank Question: 716****Answer: A**

1 Pt(s)

Unit 1 is responding to a large break LOCA inside containment. Given the following condition:

- The "C-Leg Recirc FWST To CONT SUMP SWAP ENABLE TRN A" light is lit on MC-11.

This light will light when swapover is not defeated and which one of the following events occurs?

- A. Safety Injection.**
- B. FWST level reaches the swapover setpoint.**
- C. Safety Injection occurs and FWST level reaches the swapover setpoint.**
- D. 1 NI-185A (ND PUMP 1A CONT SUMP SUCT) opens.**

---

**Distracter Analysis:**

- A. Correct:** The light is lit IF Safety Injection has occurred and the DEFEAT pushbutton has not been depressed
- B. Incorrect:** FWST level does not input into the logic for the light  
**Plausible:** FWST level does input into the logic for opening the sump isolation valves.
- C. Incorrect:** Does not indicate that FWST level has reached the swapover setpoint  
**Plausible:** Both of these conditions must exist to open the sump isolation valves.
- D. Incorrect:** The valves open when SI has occurred, (even if reset) and FWST reaches 37%, they are not part of the light.  
**Plausible:** If the candidate confuses the enable feature with the actuation.

Level: RO&amp;SRO

KA: 013A3.02(4.1/4.2)

Lesson Plan Objective: ISE SEQ 4

Source: New

Level of knowledge: comprehension

References:

1. OP-CN-ECCS-ISE page 25



**Bank Question: 717****Answer: C**

1 Pt(s)

Which of the following describes the expected response of AFD over core life? Assume ARO conditions are maintained throughout core life.

- A. Becomes more negative throughout core life.
- B. Remains relatively unchanged throughout core life.
- C. Initially, (within the first month), moves negative, becomes less negative over the remainder of core life.
- D. Initially, (within the first month), moves less negative, becomes more negative over the remainder of core life.

---

**Distracter Analysis:**

- A. **Incorrect:** becomes less negative after first month  
**Plausible:** Operator fails to understand the effect over core life of fuel burnup
- B. **Incorrect:** changes substantially over core life  
**Plausible:** operator feels that the competing effects of boron, MTC and burnup offset each other to lead to no net effect.
- C. **Correct:** Boron depletion and MTC becoming negative will push flux to the bottom of the core early in core life. After that, because some fuel is already depleted in the bottom of the core, flux will tend to shift to the top and AFD will become less negative.
- D. **Incorrect:** opposite to the actual conditions  
**Plausible:** Operator confuses the effect of MTC on AFD.

Level: RO Only

KA: SYS 015A1.06(2.5/2.9)

Lesson Plan Objective: PD Obj 9

Source: Mod PD-18-D

Level of knowledge: comprehension

References:

1. OP-CN-CTH-PD pages 14-15

**Bank Question: 718****Answer: B**

1 Pt(s) Unit 2 is conducting a plant shutdown from 100% power. Given the following events and conditions:

- Reactor power is 6%
- All manual actions have been taken as required in the procedures
- Intermediate Range channel N-36 fails HIGH.

Which of the following statements correctly describes how this failure affects the reactor shutdown and subsequent operation of the Nuclear Instrumentation System?

- A. The reactor will trip; the source range detectors will reenergize when N-35 decreases to the proper setpoint.
- B. The reactor will trip; the source range detectors will have to be manually reenergized.
- C. The reactor will not trip; the source ranges will reenergize when N-35 decreases to the proper setpoint.
- D. The reactor will not trip; the source ranges will have to be manually reenergized.

---

**Distracter Analysis:**

- A. **Incorrect:** The source range instruments will not automatically reenergize  
**Plausible:** If the operator believes the remaining IR energizes the Source Ranges, but knows the reactor trips.
- B. **Correct:** The IR trip will occur when either IR channel increases to > 25% equivalent. However this trip is blocked manually when P-10 is satisfied. Since reactor power is given as 6%, when N-36 fails high, the IR high flux trip will occur.  
The Source Range instruments will automatically reenergize when:
  - 1. P-10 is not satisfied, 3/4 NIS PR < 10% and
  - 2. P-6, both IR <  $10^{-10}$  ampsOtherwise they will need to be MANUALLY reenergized. With a high failure of IR N-36, they will not automatically reenergize.
- C. **Incorrect:** The reactor will trip and the source range instruments will not automatically reenergize  
**Plausible:** Operator believes the IR trip is blocked and only one IR is necessary to energize the SR

- D.**     **Incorrect:** The reactor will trip and the source range instruments will not automatically reenergize  
          **Plausible:** Operator believes the IR trip is blocked but knows both IR are necessary to energize the SR.

Level: RO&SRO

KA: SYS 015KG2.2(4.0/3.5)

Lesson Plan Objective: ENB SEQ 9

Source: New

Level of knowledge: analysis

References:

1. OP-CN-IC-ENB page 10, 13

**Bank Question: 720****Answer: C**

1 Pt(s) Unit 2 was operating at 100% power. Given the following events and conditions:

- The transmitter for channel I of Containment pressure has failed.
- The appropriate bistables have been tripped.
- Subsequently, the vital 120VAC power supply for instrument bus channel IV, (ERPD) fails
- All instruments powered from the bus are de-energized.

Which one of the following correctly describes the status of the Containment Spray System if containment pressure subsequently increases to 5 psig?

- A. Bistable for channel IV has tripped; containment spray has actuated when power was lost.**
- B. Bistable for channel IV has tripped; containment spray will actuate when containment pressure exceeds 0.4 psig.**
- C. Bistable for channel IV has not tripped; containment spray will actuate if containment pressure exceeds 3.0 psig.**
- D. Bistable for channel IV has not tripped; containment spray auto actuation is prevented from occurring.**

---

**Distracter Analysis:**

Containment Spray actuation is normally 2 of 4 channel coincidence. One channel was tripped when it failed earlier making the coincidence 1/3. The loss of power to the second channel does not trip a second bistable because they are energize-to-actuate. Of the two remaining channels, one tripping will result in auto actuation at the setpoint of 1 psig.

- A. Incorrect:** The bistable for channel IV has not tripped since the bistables are energize-to-actuate  
**Plausible:** If the candidate thinks that the bistables are tripped when they are deenergized – like most other protection bistables – and forgets about the CCPS enable signal.
- B. Incorrect:** Bistables are energize-to-actuate.  
**Plausible:** If the candidate reverses the energize-to-actuate logic, the CCPS enables containment spray actuation at 0.4 psig.
- C. Correct:** One bistable is tripped; only one more needs to trip above the 0.4 psig CCPS interlock – trips at 3.0 psig in containment.

- D.**    **Incorrect:** The loss of channel IV does not preclude containment spray actuation.  
         **Plausible:** One bistable is tripped only one additional channel needs to trip.

Level: RO&SRO

KA: SYS 022A3.01(4.1/4.3)

Lesson Plan Objective: ISE Obj 4

Source: New

Level of knowledge: analysis

References:

1. OP-CN-ECCS-ISE pages 10 and 20

**Bank Question: 721****Answer: C**

1 Pt(s)

Unit 1 was shutdown in mode 5 following refueling operations preparing to commence a plant heat up to mode 4. Given the following events and conditions:

- Alarm 1AD-13 B/7 (ICE COND ACCESS DOOR OPEN) annunciates
- Containment divider barrier integrity has been established

Which one of the following conditions could cause this alarm?

**REFERENCE PROVIDED: Tech Spec 3.6.14 and bases page 1**

- A. Malfunction of the containment air return system.**
- B. Malfunction of the containment pressure control system.**
- C. The door's inflatable rubber boot develops a leak.**
- D. Increasing containment pressure.**

---

**Distracter Analysis:** containment divider barrier integrity would require that the NF access doors be closed and the seal inflated.

- A. Incorrect:** would not effect the access doors.  
**Plausible:** this is a cause for ice condenser lower inlet doors to open.
- B. Incorrect:** would not effect the access doors  
**Plausible:** this is a cause for ice condenser lower inlet doors to open.
- C. Correct:** For the door to indicate closed, it must be latched and the inflatable rubber boot must be inflated.
- D. Incorrect:** containment pressure would not cause the access door to open  
**Plausible:** increasing containment pressure causes other ice condenser doors to open if a D/P develops between upper and lower containment.

Level: RO Only

KA: SYS 025 K6.01 (3.4/3.6)

Lesson Plan Objective: none

Source: New

Level of knowledge: comprehension

References:

1. OP/1/B/6100/010N B/7
2. NF lesson plan page 10
3. Tech Spec 3.6.14 page 1-3
4. Tech Spec 3.6.14 bases page 1

**Bank Question: 722****Answer: B**

1 Pt(s)

Unit 2 is operating at 100% power. Given the following events and conditions:

- A turbine control valve closes
- Reactor power decreases to 65%.

What action should the crew take to prevent possibly deadheading the condensate booster pumps?

- A. Start the standby hotwell pump.**
- B. Stop the "C" heater drain tank pumps.**
- C. Open the "C" heater bypass valve.**
- D. Ensure 2CM-83 is modulating to 150 psig.**

---

**Distracter Analysis:**

An engineering evaluation of CNS Unit 1 and 2 reactor trips showed that the Heater Drain Tank pumps develop sufficient discharge pressure to deadhead the condensate system. This phenomenon lasts until the drain flow to the tanks decreases and level control closes places the pumps in recirculation. This was observed to last up to 2.5 minutes. (PIP 98-1726).

- A. Incorrect:** This action while possibly appropriate does not address the deadheading issue.  
**Plausible:** This will change the condensate flow and the candidate may think that this will prevent deadheading.
- B. Correct:** To avoid the problem of deadheading, heater drain tank pumps are shutdown by procedure anytime the unit operates below 70% power.
- C. Incorrect:** Opening the heater bypass valve would not prevent deadheading.  
**Plausible:** This action would be taken if there was a problem with the heater, it does not address deadheading the condensate system.
- D. Incorrect:** This has no effect on deadheading the condensate system  
**Plausible:** This was the correct answer to a previous NRC question on load rejection.

Level: RO&SRO



KA: SYS 056A2.04(2.6/2.8)

Lesson Plan Objective: CM Obj 4

Source: New

Level of knowledge: comprehension

References:

1. OP-CN-lesson plan CM page 14

**Bank Question: 723****Answer: C**

1 Pt(s)

Unit 2 was operating at 45% power. Given the following events and conditions:

- Problems with feedwater control result in overfeeding all steam generators
- S/G NR levels increased to 80%
- No operator action has been taken.

Which one of the following statements correctly describes the automatic actions (if any) that should have already occurred?

- A. No automatic actions should have occurred at this time.
- B. Turbine trip, feedwater isolation.
- C. Turbine trip, feedwater isolation, CF pumps tripped.
- D. Reactor trip, turbine trip, feedwater isolation, CF pumps tripped.

---

**Distracter Analysis:**

- A. **Incorrect:** turbine tripped, feedwater isolated and CF pumps tripped  
**Plausible:** possible if operator believes P14 setpoint is 84% (unit 1 setpoint is 84% - unit 2 setpoint is 77%)
- B. **Incorrect:** CF pumps are tripped  
**Plausible:** if operator realizes CF isolation occurs but does not realize the CF pumps also trip.
- C. **Correct:** P-14 causes CF isolation, main turbine trip and CF pump trip.
- D. **Incorrect:** reactor would not trip – power below 48% P8  
**Plausible:** operator believes turbine trip will cause a reactor trip.

Level: RO&SRO

KA: SYS 059A4.12(3.4/3.5)

Lesson Plan Objective: CF Obj 10

Source: New

Level of knowledge: analysis

References:

1. OP-CN-MC-CF page 24
2. OP-CN-IC-IPX page 19

**Bank Question: 724****Answer: D**

1 Pt(s)

The crew is performing a cooldown in accordance with EP/1/A/5000/ES-0.2 (*Natural Circulation Cooldown*).

Which of the following actions will cause both the cooldown rate and the NC system flow rate to **increase**?

- A. Starting more CRDM fans.
- B. Increasing the setpoint on the steam dumps, if in automatic.
- C. Decreasing the output of the steam dump controller, if in manual.
- D. Increasing auxiliary feedwater flow to the steam generators.

---

**Distracter Analysis:**

The NC system flow rate increases as the temperature difference between the S/G and reactor vessel increases – causing the thermal driving head to increase.

- A. **Incorrect:** while starting more fans enhances head cooling, it will reduce the temperature difference between the S/G and reactor vessel and thereby reduce the thermal driving head and decreasing the NC system flow rate.  
**Plausible:** Starting more CRDM fans will increase the cooldown rate.
- B. **Incorrect:** increasing the setpoint will raise the pressure setpoint, if in auto; the dumps will close, decreasing the cooldown rate and NC system flow rate.  
**Plausible:** if the candidate reverses the effects that steam pressure has on the thermal driving head.
- C. **Incorrect:** decreasing the output closes steam dumps and reduces the cooldown rate on the S/G, which causes the delta temp to reduce, and decreases NC system flow.  
**Plausible:** if the candidate reverses the effect of the reduction in S/G cooldown rate.
- D. **Correct:** increased CA flow will have the effect of increased steam flow, a decrease in NC temperature and will increase the cooldown rate.

Level: RO&amp;SRO

KA: SYS 061K5.01(3.6/3.9)

Lesson Plan Objective: EP1 Obj 21

Source: New

Level of knowledge: comprehension

References:

1. OP-CN-THF-HT pages 10 and 11

**Bank Question: 725****Answer: B**

1 Pt(s)

Unit 1 was operating at 100% power. Given the following events and conditions:

- The feeder breaker to 600VAC MCC-1EMXA opens and the MCC is deenergized.

Which of the following NV loads have been lost?

- A. Reactor makeup pump 1A and boric acid pump 1A.**
- B. Boric acid pump 1A only.**
- C. Reactor makeup pump 1A only.**
- D. Centrifugal charging pump A auxiliary lube oil pump.**

---

**Distracter Analysis:**

- A. Incorrect:** power supply is MXW to the reactor makeup pump  
**Plausible:** 600 V load
- B. Correct:** power supply is EMXA
- C. Incorrect:** power supply is MXW to the reactor makeup pump  
**Plausible:** 600 V load
- D. Incorrect:** power supply is MXK  
**Plausible:** 600 V load

Level: RO&SRO

KA: 004 K2.02 (2.9/3.1)

Lesson Plan Objective: NV SEQ 17

Source: New

Level of knowledge: memory

References:

1. OP-CN-0157-01.03 (Not provided - large drawing size)
2. NV lesson plan pages 28, 37 & 38

**Bank Question: 726****Answer: A**

1 Pt(s) Unit 2 is operating at 100%. Given the following events and conditions:

- A planned release is in progress from the waste gas system.
- Plant Vent monitor, EMF-35 (L) (Unit Vent Part) reaches the trip 2 setpoint.

Which one of the following automatic actions should occur?

- A. If open, 1WG-160, (WG Decay Tank Outlet to Unit Vent Control), will close.**
- B. Containment Ventilation Isolation signal will be generated.**
- C. If open, 1WL-124, (Waste Monit Tnk Pmps Disch), will close.**
- D. Fuel Pool Ventilation (VF) Filter Train will be tripped.**

---

**Distracter Analysis:**

EMF-35, 36, and 37

Automatic functions:

Trips unit related unfiltered exhaust fans.

Aligns VF filter train to the filtered mode.

Secures WG release (shuts 1WG-160).

Secures cont air release (shuts VQ-10).

- A. Correct:**
- B. Incorrect:** not generated by EMF-35(L)  
**Plausible:** this is generated by emf-38, 39 or 40
- C. Incorrect:** not generated by EMF-35(L)  
**Plausible:** closed by emf-49
- D. Incorrect:** not generated by EMF-35(L)  
**Plausible:** VF not tripped, it is aligned to filtered mode

Level: RO&SRO

KA: SYS 071K4.04(2.9/3.4)

Lesson Plan Objective: EMF SEQ 2

Source: New

Level of knowledge: memory

References:

1. OP-CN-MC-EMF page 9 and 11
2. OP/1/B/6100/010Y A/1
3. OP/1/B/6100/010X C/5



**Bank Question: 728****Answer: B**

1 Pt(s)

Unit 2 is in mode 4. The crew is preparing to start all the NCPs.

Which one of the following would satisfy the oil lift system interlock and allow the starting of the NCPs?

- A. Start both oil lift pumps, when oil lift pressure is greater than 500 psig, start the NCP.
- B. Start one oil lift pump, when oil lift pressure is greater than 500 psig, start the NCP.
- C. Start both oil lift pumps, when oil lift pressure is greater than 200 psig, start the NCP.
- D. Start one oil lift pump, when oil lift pressure is greater than 200 psig, start the NCP.

---

**Distracter Analysis:**

- A. **Incorrect:**  
**Plausible:** Only one lift pump can be started at a time.
- B. **Correct:**  
**Plausible:** One lift pump started, when pressure is greater than 500 psig, the NCP can be started.
- C. **Incorrect:**  
**Plausible:** Only one lift pump can be started and the pressure is 500 psig.
- D. **Incorrect:**  
**Plausible:** Pressure setpoint is 500 psig.

Level: RO&amp;SRO

KA: SYS 003A4.03(2.8/2.5)

Lesson Plan Objective: NCP SEQ 10

Source: New

Level of knowledge: memory

References:

1. OP-CN-PS-NCP page 11



**Bank Question: 729****Answer: D**

1 Pt(s)

Unit 1 was operating at 100% when a control rod M-4 became misaligned. Given the following events and conditions:

- The crew has entered AP/14, Case 1, (*Control Rod Misalignment*).
- Data B Failure lights are lit on the DRPI panel.
- The DRPI position for Rod M-4 in Control Bank D is 204 steps
- The group step counter for Control Bank D is at 218 steps
- There is a General Warning alarm for Rod M-4
- There are no Urgent Failure lights lit
- No operator action has been taken at this time

What is the position of the misaligned rod?

- A. Between 216 and 192 steps.
- B. Between 214 and 194 steps.
- C. Between 214 and 200 steps
- D. Between 208 and 194 steps

---

**Distracter Analysis:**

Due to the calculation method in half accuracy mode (one Data Cabinet not supplying accurate data) the system accuracy is reduced.

With a Data A Failure, the system accuracy is +10, -4 steps.

With a Data B Failure, the system accuracy is -10, +4 steps.

Note in AP states: If either Data A or Data B Failure lights are flashing and the Urgent Failure lights are dark, then the individual rod position indication will be in the "half accuracy" mode providing 12 step increment position indication instead of 6. Individual rod position indication may differ by as much as 10 steps from group step counter indication.

- A. **Incorrect:**  
**Plausible:** Accuracy assumed is +/- 12
- B. **Incorrect:**  
**Plausible:** Accuracy assumed is +/-10
- C. **Incorrect:**  
**Plausible:** Accuracy assumed is +10/-4
- D. **Correct:** Correct accuracy of -10/+4

Level: RO Only

KA: SYS 001A2.17(3.3/3.8)

Lesson Plan Objective: EDA SEQ 3

Source: New

Level of knowledge: comprehension

References:

1. OP-CN-MC-EDA page 10
2. AP/14 case I page 2-3

**Bank Question: 730****Answer: B**

1 Pt(s)

Unit 1 is operating at 100% power, with the makeup system in automatic control. Given the following events and conditions:

- A small (8 gpm) leak develops downstream of 1NV-294 (*NV PMPS A & B FLO CTRL*)

Which one of the following statements correctly describes the plant indications several hours after the leak starts?

- A. Increased VCT makeup and pressurizer level decreasing.
- B. Increased VCT makeup and pressurizer level constant.
- C. VCT level constant and pressurizer level decreasing.
- D. VCT level decreasing to the FWST swapover setpoint and pressurizer level constant.

---

**Distracter Analysis:**

- A. **Incorrect:** pressurizer level will not decrease  
**Plausible:** operator believes that due to the leak, level will continue to decrease.
- B. **Correct:** Initially pressurizer level will decrease, which will cause charging to increase and return level to program. Pressurizer level will remain on program since the leak is within the capacity of the charging pumps. Because charging is greater than letdown, but within the capability of the makeup system, VCT will decrease to the makeup setpoint and be returned to program (this will repeat for as long as the leak exists).
- C. **Incorrect:** VCT level does not remain constant  
**Plausible:** operator fails to realize that a constant mass loss out of the VCT exists.
- D. **Incorrect:** VCT level will not decrease to the swapover setpoint  
**Plausible:** operator believes that makeup will not keep pace with the leak.

Level: RO Only

KA: SYS 004K3.05(3.7/4.1)

Lesson Plan Objective: none

Source: New

Level of knowledge: comprehension

References:

1. OP-CN-PS-ILE pages 11, 15-17

**Bank Question: 731****Answer: A**

1 Pt(s)

Federal Regulations require the emergency core cooling system to be designed to maintain peak cladding temperature below 2200 °F.

Which one of the following statements correctly describes the basis for this design criterion?

- A. To prevent acceleration of the zircalloy-water reaction.**
- B. To prevent exceeding the zircalloy clad melting point.**
- C. To prevent exceeding the fuel melting point.**
- D. To prevent the onset of full film boiling and DNB.**

---

**Distracter Analysis:**

- A. Correct:** The zirconium-water reaction is described by the following chemical equation:  $\text{Zr} + 2\text{H}_2\text{O} \rightarrow \text{ZrO}_2 + 2\text{H}_2 + \text{HEAT}$ . The rate of this reaction is highly dependent upon clad temperature, such that above approximately 1800°F the reaction becomes significant. It becomes accelerated at 2200°F and auto-catalytic (self-sustaining) at 4800°F.
- B. Incorrect:** Zircalloy will melt at approximately 3400°F.  
**Plausible:** its approximately 900 degrees below the melting point.
- C. Incorrect:** fuel melt is a much higher temperature. ~5000 °F.  
**Plausible:** a logical answer if the candidate doesn't know the answer
- D. Incorrect:** these are event specific mechanisms rather analysis criteria.  
**Plausible:** If the candidate confuses heat transfer mechanisms with ECCS criteria.

Level: RO Only

KA: SYS 013G2.1.32(3.4/3.8)

Lesson Plan Objective: TA-AM SEQ 10

Source: New

Level of knowledge: memory

References:

1. OP-CN-IC-ISE page 5

2. OP-CN-TA-AM pages 7-9
3. 10CFR50.46



**Bank Question: 732****Answer: B**

1 Pt(s) Unit 1 is performing a plant shutdown.

Given the following events and conditions in the following sequence:

- Turbine load is currently at 300Mwe.
- 1A CF pump is running.
- 1B CF pump has been shutdown per procedure.
- A problem with the feed system causes the "B" S/G CF control valve to close.
- The running CF pump trips.
- The operator manually trips the reactor.

Assuming systems operate as designed, when did the CA system receive a start signal?

- A. When the CF control valve was closed for more than 30 seconds.
- B. When the CF pump tripped.
- C. When the B S/G NR level decreased below 37%.
- D. When the manual reactor trip occurs.

---

**Distracter Analysis:**

- A. **Incorrect:** trips upon loss of CF pumps  
**Plausible:** This signal is in effect only when load is greater than 40%
- B. **Correct:** The AMSAC signal to start CA on loss of both MFPs is always in service
- C. **Incorrect:** trips upon loss of CF pumps  
**Plausible:** 37% is the unit 2 setpoint
- D. **Incorrect:** trips upon loss of CF pumps  
**Plausible:** a reactor trip causes a CA start signal.

Level: RO Only

KA: SYS 059A2.01(3.4\*/3.6\*)

Lesson Plan Objective: CF SEQ 13

Source: New

Level of knowledge: analysis

References:

1. OP-CN-MC-CF page 9-10, 27-28

**Bank Question: 733****Answer: A**

1 Pt(s)

Unit 2 is responding to a large break LOCA with a failure of safety injection. Given the following events and conditions:

- The reactor vessel level drops below the top of active fuel and continues to decrease in a uniform manner,

Which one of the following statements correctly describes the expected response of the Source Range instruments as core voiding occurs?

- A. Initially increase with increased voiding fraction and then decrease when the effects of loss of moderator override the effects of leakage.**
- B. Initially decrease due to the effect of the loss of moderator and then increase due to the increased voiding fraction.**
- C. Continually increase due to the increase in fast neutron leakage.**
- D. Continually decrease due to the effects of the loss of moderator.**

---

**Distracter Analysis:**

- A. Correct:**
- B. Incorrect:** The SR count rate increases then decreases  
**Plausible:** If the candidate reverses the effects
- C. Incorrect:** The SR count rate will decrease after loss of moderator overrides voiding fraction effects  
**Plausible:** Initially, the SR response will increase – half correct
- D. Incorrect:** The SR will initially increase  
**Plausible:** The SR response will decrease after a certain point – half correct

Level: RO Only

KA: SYS 015K1.04(3.5/3.5)

Lesson Plan Objective: ENB SEQ 13

Source: New

Level of knowledge: memory

References:

1. OP-CN-IC-ENB page 28-29

**Bank Question: 735****Answer: C**

1 Pt(s) Which of the following is a characteristic of both NI pumps running as compared to just one NI pump running?

- A. Pump head loss is approximately halved for each pump.**
- B. Pump running amps are approximately halved for each pump.**
- C. Injection flow is approximately doubled.**
- D. Discharge pressure is approximately doubled.**

---

**Distracter Analysis:**

- A. Incorrect:** Total system head loss is increased.  
**Plausible:** System head loss increases as total flow rate increases
- B. Incorrect:** Running amps are slightly reduced – but not by one half.  
**Plausible:** If the candidate does not know pump laws.
- C. Correct:** The flow is additive for parallel pumps.
- D. Incorrect:** Discharge pressure will be increased but not doubled  
**Plausible:** if the candidate reverses the pump laws for flow and pressure.

Level: RO Only

KA: SYS 006K5.08(2.9\*/3.1\*)

Lesson Plan Objective: FF Obj 16, 17

Source: New

Level of knowledge: memory

References:

1. OP-CN-THF-FF page 9-12

**Bank Question: 736****Answer: C**

1 Pt(s) Unit 2 is in the process of conducting a plant startup. Given the following events and conditions:

- Power range channels indicate the following:
  - PR N41 = 8%
  - PR N42 = 8%
  - PR N43 = 10%
  - PR N44 = 8%

Which of the following conditions would result in an Automatic Reactor Trip?

- A. All four RCPs trip.**
- B. Pressurizer level increases to 94%.**
- C. RCS pressure decreases to 1840 psig.**
- D. One turbine impulse pressure channel fails high.**

---

**Distracter Analysis:**

- A. Incorrect:** all loop flow trips are automatically blocked below P-7  
**Plausible:** if the candidate does not recall that the NCP trip is blocked by P-7
- B. Incorrect:** Pressurizer High Level, Pressurizer Low Pressure, blocked by P-7  
**Plausible:**
- C. Correct:** As pressure decreases to 1845 psig, SI is actuated. The SI signal generates a Reactor Trip Signal
- D. Incorrect:** P-7 would be enabled, but this does not cause a trip  
**Plausible:** if the candidate is confused over the effect of turbine impulse on main generator trip

Level: RO&SRO  
KA: SYS 010K1.02(3.9/4.1)

Lesson Plan Objective: ISE SEQ 4  
Source: New

Level of knowledge: comprehension

References:  
1. OP-CN-ECCS-ISE page 14

**Bank Question: 737****Answer: B**

- 1 Pt(s)      Unit 2 is at 100% power. PZR level and pressure control are selected to channels 1 and 2.
- Which of the following failures will result in the pressurizer backup heaters immediately de-energizing?
- A. Controlling pressurizer pressure channel fails low.**
  - B. Backup pressurizer level channel fails low.**
  - C. Controlling pressurizer level channel fails high.**
  - D. Backup pressurizer pressure channel fails high.**

---

**Distracter Analysis:**

- A. Incorrect:** result is the heaters energize  
**Plausible:** a low pressure condition may raise concerns of saturation margin
- B. Correct:** letdown isolates, heaters de-energize
- C. Incorrect:** Heaters energize (on deviation)  
**Plausible:** high level may elicit concern regarding over-pressurizing a solid pressurizer due to heat up.
- D. Incorrect:** only input to PORV  
**Plausible:** high level may elicit concern regarding over-pressurizing a solid pressurizer due to heat up.

Level: RO&amp;SRO

KA:SYS 011K3.03(3.2/3.7)

Lesson Plan Objective: ILE SEQ 6

Source: New

Level of knowledge: comprehension

References:

1. OP-CN-PS-ILE pages 15-16

**Bank Question: 738****Answer: A**

1 Pt(s) Unit 2 is at full power. Reactor Trip breakers A (RTA) and B (RTB) are closed, bypass breakers (RYA, RYB) are open.

Which one of the following statements correctly describes how a loss of EPA would effect the operation of Reactor Trip breaker A from the control room?

- A. RTA would still open from either a manual or automatic signal.
- B. RTA would not open in response to a manual reactor trip signal; an automatic trip would still open the breaker.
- C. RTA would not open in response to either an automatic or manual reactor trip signal.
- D. RTA would immediately trip open because the shunt trip coil would deenergize.

---

**Distracter Analysis:**

- A. **Correct:** EPA powers the shunt trip coil on RTA. This coil energizes to trip RTA. This function would be lost. However, a manual reactor trip signal will also trip the UV coils from SSPS.
- B. **Incorrect:** A manual reactor trip signal will trip the UV coils from SSPS.  
**Plausible:** if the candidate does not recognize that SSPS train A provides power to the UV coils – which deenergize form the manual trip signal to trip RTA. SSPA train A is powered from an auctioneered circuit from ERPA and ERPB.
- C. **Incorrect:** Both Rx trips would still function – the UV coil is unaffected.  
**Plausible:** if the candidate does not recognize that a manual Rx trip sends a trip signal to the UV coils – which are powered from SSPS train A – which receives power from an auctioneering circuit from ERPA and ERPB.
- D. **Incorrect:** The ST coil energizes to trip.  
**Plausible:** if the candidate confuses the UV coil and the ST functions – ST coil is normally energized

Level: RO&SRO

KA: SYS 012A2.07(3.2/3.7)

Lesson Plan Objective: IPX SEQ 2, 4, 6

Source: New

Level of knowledge: comprehension

References:

1. OP-CN-MC-IPX pages 7, 8 23, 24, 25, 27
2. OP-CN-EL-EPL page 27



**Bank Question: 739****Answer: C**

1 Pt(s)

During a reactor startup, when should the "RPI at Bottom Rod Drop" (annunciator D/9 on 1AD-2) clear?

- A. When the rod position startup push button is depressed.
- B. Once all the shutdown banks are fully withdrawn.
- C. Once shutdown banks are withdrawn and control bank A is greater than 6 steps off the bottom.
- D. Once control bank D rods are off the bottom

---

**Distracter Analysis:**

The "RPI at Bottom Rod Drop" annunciator will be activated if any of the following conditions are met:

- Any Shutdown or Control Bank A rod has a calculated position which gives a rod bottom LED.
- Any Control Bank B, C, or D has a calculated position which gives a rod bottom LED and the other rods in the same bank and group do not have rod bottom LED's.

Any Control Bank B, C and D rod has a calculated position, which gives a rod bottom LED, and rods that should be sequenced out after that rod, are off the bottom. (this is not the case during a startup, so the alarm should remain clear)

- A. **Incorrect:** the pushbutton resets all the alarms associated with rod control – but not alarms associated with DRPI  
**Plausible:** if the candidate does not recognize that DRPI alarms are not reset.
- B. **Incorrect:** Also requires control bank A to be withdrawn  
**Plausible:** If the candidate thinks that only the SD banks need to be withdrawn.
- C. **Correct:** Alarm should clear once Bank A is off the bottom.
- D. **Incorrect:** Annunciator will clear when Bank A lifts off the bottom.  
**Plausible:** At full accuracy, the rod bottom LED for control bank D rods will extinguish and the six-step position LED will light when the rods are approximately three steps from the bottom.

Level: RO&SRO

KA: SYS 014K4.02(2.5\*/2.7\*)

Lesson Plan Objective: EDA SEQ 4, 5

Source: New

Level of knowledge: memory

References:

1. OP-CN-IC-EDA page 12

**Bank Question: 740****Answer: D**

1 Pt(s) Unit 1 is in mode 3 and the operators are controlling the plant from the SSF.

What is the coincidence, setpoint and auto action for starting the CAPT?

- A. 1 of 4 S/G WR level at 45%, SA-2 fails open.
  - B. 2 of 4 S/G WR level at 45%, SA-2 fails open.
  - C. 1 of 4 S/G WR level at 45%, SA-5 fails open.
  - D. 2 of 4 S/G WR level at 45%, SA-5 fails open.
- 

**Distracter Analysis:**

- A. **Incorrect:** requires 2 of 4 – SA-2 does not open  
**Plausible:** knows one valve opens, not sure which one.
- B. **Incorrect:** SA-2 does not open  
**Plausible:** coincidence correct, auto action incorrect.
- C. **Incorrect:** requires 2 of 4 S/G channels  
**Plausible:** incorrect coincidence, correct action.
- D. **Correct:**

Level: RO&SRO

KA: SYS 016K4.01(2.8\*/2.9\*)

Lesson Plan Objective: CA obj 4

Source: New

Level of knowledge: memory

References:

1. OP-CN-CA page 9

**Bank Question: 741****Answer: D**

1 Pt(s) A LOCA has occurred on Unit 2. The crew has determined that train A of the ND system must be used to supply containment spray.

Which one of the following statements correctly describes the electrical interlock that must be satisfied to allow an operator to open 2NS-43A (*ND PMP 1A TO CONT SPRAY HDR*)?

- A. Both 2 ND-1B, and 2 ND-2A (*ND PUMP 1A SUCT FRM LOOP B*) must be closed.
- B. Either 2 ND-36B or 2ND-37A (*ND PUMP 1B SUCT FROM LOOP C*) must be closed.
- C. Train B of the ND system must be operating in the Cold Leg Recirculation mode.
- D. CPCS signal must be present.

---

**Distracter Analysis:**

Interlocks to open 2NS-43A

- 1. ND-1B or ND-2A closed
- 2. NI-185A open
- 3. CPCS > 0.4 psig

- A. **Incorrect:** Only one valve of these 2 valves must be closed.  
**Plausible:** if the operator does not recognize that only 1 of these 2 valves is necessary to satisfy the interlock.
- B. **Incorrect:** These are the train B interlocks for 2ND-36B/37A.  
**Plausible:** operator reverses the trains
- C. **Incorrect:** Does not electrically interlock containment spray. Either train may be aligned for cold leg recirc.  
**Plausible:** This is an administrative requirement prior to opening 2NS-43A.
- D. **Correct:**

Level: RO&SRO

KA: SYS 026K4.07(3.8\*/4.1\*)

Lesson Plan Objective: NS SEQ 8, 9 ND SEQ 8

Source: New

Level of knowledge: memory

References:

1. ND system page 14
2. FR-Z.1

**Bank Question: 743****Answer: B**

1 Pt(s) Unit 1 is in the process of offloading the core to the spent fuel pool. Spent fuel pool temperature is 120°F.

Which of the following could cause indicated spent fuel pool level to suddenly decrease?

- A. The containment purge exhaust flow rate is set less than supply flow rate.
- B. The spent fuel building running filter exhaust fan trips but the supply fan continues to run.
- C. After shutting down the containment purge system, the operator places the "MODE SELECTOR" switch on 1RB-CP-1 in the "NORM" position.
- D. The running spent fuel pool cooling pump trips.

---

**Distracter Analysis:** With core offload in progress, containment integrity must be established per Tech Specs. While refueling is in progress, the refueling canal is open and any changes in pressure between containment and the spent fuel pool will result in a manometer effect between the refueling cavity and the SFP.

- A. **Incorrect:** exhaust flow < supply flow will cause pressure in containment to increase, causing SFP level to increase.  
**Plausible:** if the candidate reverses the effects of supply and exhaust flows on level.
- B. **Correct:** Continuing to supply outside air will increase pressure in the SFP building, increasing pressure, forcing level to shift to containment.
- C. **Incorrect:** This switch realignment will have no effect on containment pressure after the containment purge system has been shut down,  
**Plausible:** If this switch was realigned as described when the purge system was in operation, it would cause pressure in upper containment to increase. There a precaution dealing with the mode selector switch, however, placing it in NORM at this time will have zero effect on containment pressure, and therefore no effect on levels.
- D. **Incorrect:** this will not suddenly change level in the spent fuel pool  
**Plausible:** It takes hours to reach boiling conditions in the SFP and then it would be a very gradual decrease.

Level: RO&SRO

KA: SYS 033A1.01(2.7/3.3)

Lesson Plan Objective: VF SEQ 3, 4 VP SEQ 3

Source: New

Level of knowledge: comprehension

References:

1. OP/1/6450/015 enclosure 4.1 and 4.2
2. Lesson plan VF page 5
3. Lesson plan VP page 15

**Bank Question: 744a      Answer: C**

---

1 Pt(s)      Unit 1 was operating at 100% power when a tube rupture occurred in the A S/G.

Current conditions:

- A S/G pressure is 1000 psig.
- A S/G NR level is 100%
- A S/G WR level is approximately 74% and steady
- NC pressure is 1500 psig

Which one of the following statements correctly explains why the narrow and wide range level indications are not the same?

- A. The narrow range upper level taps are at a higher elevation inside the S/G than the wide range taps; therefore the wide range instrument will always read lower than the narrow range instrument has reached the upper level tap.
- B. The wide range upper level taps are at a higher elevation inside the S/G than the narrow range taps; therefore the wide range instrument will continue to show level indication after the narrow range instrument has reached the upper level tap.
- C. The wide range and narrow range upper level taps are at the same elevation inside the S/G but the wide range level instrument is cold calibrated; therefore it will always read less than narrow range level until the steam generator is cooled down.
- D. The wide range and narrow range upper level taps are at the same elevation inside the S/G but the wide range level instrument has conservative instrument errors to prevent feeding a hot dry S/G; therefore the WR instrument will always read less than narrow range level until the steam generator is cooled down.

---

**Distracter Analysis:**

- A.      **Incorrect:** NR and WR upper level taps are at the same elevation  
         **Plausible:** The candidate may think that placing the upper level tap higher causes the WR S/G level to read lower than NR
- B.      **Incorrect:** NR and WR upper level taps are at the same elevation  
         **Plausible:** operator believes that the WR level instruments are at a different location.



- C. **Correct:** Since the WR and NR Level Instruments upper taps are located at the same elevation, a NR indication of greater than 100% indicates both the NR and WR upper taps are submerged.  
ACTUAL Wide Range Level must therefore be 100%, an indicated level of ~ 74% is expected.  
As ACTUAL Level in the ruptured S/G continues to increase, the Wide Range Level will continue to indicate ~ 74%. Wide Range Level indication will only increase further if the S/G is cooled down.
- D. **Incorrect:** There is no conservative instrument error inserted in the WR SG level instrument to prevent feeding a hot dry S/G  
**Plausible:** partially correct – the instrument taps are at the same elevation – using conservative instrument error offset would be a plausible way of preventing feeding a hot dry S/G

Level: RO&SRO

KA: SYS 035K6.03(2.6/3.0)

Lesson Plan Objective: SG SEQ 7

Source: New

Level of knowledge: memory

References:

1. data book
2. SG lesson page 12&13

**Bank Question: 745****Answer: B**

1 Pt(s) Unit 1 was operating at 100% power when a sudden loss of VI supply pressure to the MSIVs occurred.

Which one of the following statements correctly describes the MSIV response?

- A. MSIVs will close immediately without operator action.
- B. MSIVs will close without operator action in approximately 15 minutes.
- C. MSIVs will not close until after the operator manually presses the closed pushbutton.
- D. MSIVs will not close because they require VI pressure to actuate.

---

**Distracter Analysis:**

- A. **Incorrect:** there is a backup tank that supplies air pressure.  
**Plausible:** if the candidate does not recall that the back up tank will
- B. **Correct:**
- C. **Incorrect:** The valves will close without operator action  
**Plausible:** if the candidate thinks that operator action is required to initiate closure with spring pressure
- D. **Incorrect:** MSIVs will close after ~ 15 minutes  
**Plausible:** if the candidate thinks that they fail open and require VI pressure to close.

Level: RO Only

KA: SYS 039A4.01(2.9/2.8)

Lesson Plan Objective: SM Obj 12

Source: New

Level of knowledge: memory

References:

1. OP-CN-STM-SM page 12

**Bank Question: 747****Answer: D**

1 Pt(s) Unit 1 is operating at 10% power conducting a plant shutdown. Given the following events and conditions:

- No 6900V tiebreakers are closed.
- The main turbine is not synchronized to the grid
- The automatic fast transfer switch is in the DEFEAT position
- RC pumps A and C are running.
- A fault occurs on transformer 1T2B and the transformer is deenergized.

Which one of the following statements correctly describes the RC pump response?

- A. No RC pumps would trip.
- B. Only RC pump A would trip.
- C. Only RC pump C would trip.
- D. RC pumps A and C would trip.

---

**Distracter Analysis:**

The long sides of 1TA and 1TC both are fed from 1T2B. Loss of 1T2B would result in a loss of RC pumps A and C. The fast or hot transfer would not occur because the fast transfer is defeated – a slow or dead bus transfer would occur.

- A. **Incorrect:** RC pumps A and C would trip  
**Plausible:** - if candidate thinks that a slow bus transfer would prevent loss of RC pumps
- B. **Incorrect:** RC pump C will also trip  
**Plausible:** RC pump A will trip – if the candidate believes 1T2B feeds only 1TA long side and forgets about 1TC long side
- C. **Incorrect:** RC pump A will also trip  
**Plausible:** – if the candidate believes 1T2B feeds 1TC long side and forgets about 1TA long side.
- D. **Correct:**

Level: RO&SRO

KA: SYS 062K2.01(3.3/3.4)

Lesson Plan Objective: EP SEQ 11

Source: New

Level of knowledge: comprehension

References:

1. OP-CN-EL-EP pages 23, 27-30
2. OP-CN-MT-RC page 6

**Bank Question: 748****Answer: B**

1 Pt(s)

An Emergency Diesel Generator is running in parallel with offsite power. The operator is preparing to shutdown the diesel.

In accordance with OP/1(2)/A/6350/002, ENC 4.10, when manually unloading the Emergency Diesel Generator, the output breaker should be opened after load is reduced to 200 KW while maintaining a lagging power factor.

Which one of the following statements correctly describes the basis for this requirement?

- A. Diesel overspeed when the breaker is opened.**
- B. Breaker trip on reverse power.**
- C. Loss of power to the bus.**
- D. Breaker trip on Generator Differential.**

---

**Distracter Analysis:**

- A. Incorrect:** While possible if breaker is opened under greater load, this is not the reason for minimum load and lagging pf  
**Plausible:** the DG will pick up speed if suddenly unloaded – overspeed is a valid concern to protect the diesel against.
- B. Correct:** leading pf and low load could motor the machine leading to reverse power condition.
- C. Incorrect:** Unlikely since a breaker failure would have to occur.  
**Plausible:** if there was no reverse power trip, this condition could cause a loss of power to the bus if the DG acted as a motor and tripped the offsite power breaker on over-current.
- D. Incorrect:** An unbalance does not exist, Gen Diff should not occur.  
**Plausible:** This is a valid DG trip

Level: RO&SRO

KA: SYS 064A1.08(3.1/3.4)

Lesson Plan Objective: DG3 SEQ 17

Source: bank DG 3-027-D

Level of knowledge: memory

References:

1. OP-/1/A/6350/002 Encl 4.10

**Bank Question: 749****Answer: C**

- 1 Pt(s) Which of the following is performed when starting the RC system to prevent pump runout?
- A. Limiting the number of pumps that can be started based on the number of cooling towers available.
  - B. Limiting the number of pumps that can be started based on the number of condensers available.
  - C. Not operating a single pump with the discharge valve fully open.
  - D. Not allowing the first pump to be started unless the main condenser is isolated.

---

**Distracter Analysis:**

- A. **Incorrect:**  
**Plausible:** confuses start interlocks with runout protection.
- B. **Incorrect:**  
**Plausible:** knows that this is a requirement for pump starts.
- C. **Correct:**  
**Plausible:** "Norm"- "T/V" key switch and valve selector switch is provided for the discharge valves of the RC pumps to limit discharge valve to 52 degrees open following the start of the pump, for the valve selected, to prevent runout of the first pump started  
System procedure cautions that the pump cannot operate with its discharge valve fully open to prevent runout.
- D. **Incorrect:**  
**Plausible:** confuses caution, (no pump can be started if the condenser is isolated).

Level: RO&amp;SRO

KA: SYS 075K4.01(3.2/3.5)

Lesson Plan Objective: RC SEQ 14, 18

Source: New

Level of knowledge: memory

References:

1. OP-CN-RC 6400/001A
2. RC system lesson plan, page 7

**Bank Question: 751****Answer: B**

1 Pt(s)

Unit 1 was operating at 100% power when a large break LOCA occurred.  
Given the following sequence of events:

- Time 1: Safety Injection occurs
- Time 2: Containment phase B isolation signal generated
- Time 3: FWST low level alarm occurs
- Time 4: Containment Sump Isolation valves opened

At what time did KC to the ND heat exchangers automatically open?

- A. Time 1
- B. Time 2
- C. Time 3
- D. Time 4

---

**Distracter Analysis:**

- A. **Incorrect:**  
**Plausible:** operator believes KC auto aligns on S signal
- B. **Correct:** KD will fail open on either phase B or S signal with low FWST level.
- C. **Incorrect:**  
**Plausible:** operator fails to realize the KC valves opened on Sp
- D. **Incorrect:**  
**Plausible:** operator believes valves are interlocked with aligning ND to the sumps

Level: RO&SRO

KA: 005 K1.10 (3.2/3.4)

Lesson Plan Objective: ND SEQ 8

Source: New

Level of knowledge: memory

References:

1. Lesson plan ND page 18

**Bank Question: 752****Answer: B**

1 Pt(s)

Unit 2 is operating at 100% power. Given the following events and conditions:

- KC cooling is lost to an NCP.
- No operator action is taken.

Which one of the following conditions will first occur and require the NCP to be tripped?

- A. High # 1 seal leakoff discharge temperature.**
- B. High motor bearing temperature.**
- C. High radial bearing temperature.**
- D. High pump shaft vibration.**

---

**Distracter Analysis:**

- A. Incorrect:** seal leakoff temperature is not cooled by KC  
**Plausible:** Loss of seal injection would cause the discharge temperature to increase.
- B. Correct:** A loss of KC cooling to the NC pumps results in a gradual approach to an overheated condition prior to a possible shaft seizure. The duration of the heatup phase, estimated to be approximately ten minutes, provides a sufficient time for operator recognition and response. Additional time exists between exceeding the **high bearing temperature** limit and the conditions required for shaft seizure.  
NCP is tripped at 195 degrees (motor bearing)
- C. Incorrect:** KC does not cool the radial bearing – this is cooled by seal injection flow.  
**Plausible:** the operator may confuse seal injection with KC cooling.
- D. Incorrect:** KC flow does not affect the pump vibration  
**Plausible:** high shaft vibration will occur if the motor bearing overheats – but this will happen long after the high temperature alarm is received.

Level: RO&amp;SRO

KA: SYS 008K3.03(4.1/4.2)

Lesson Plan Objective: KC SEQ 7, 14



Source: New

Level of knowledge: comprehension

References:

1. AP-21 page ?

**Bank Question: 754****Answer: A**

1 Pt(s) Unit 1 is conducting a reactor startup. Given the following conditions and events:

- Steam dumps are in Auto in the Steam Pressure mode.
- The Train "A" P-12 solenoids lose power.

Which one of the following statements correctly describes the operation of the steam dumps?

- A. All the steam dumps will close and cannot be re-opened.**
- B. All the steam dumps will close; the cooldown bank can be used by going to BYPASS.**
- C. No effect, P-12 does not input to steam dump operation in the Steam Pressure mode.**
- D. No effect, however, if Tave decreases below the P-12 setpoint, the dumps will not close.**

---

**Distracter Analysis:**

- A. Correct:** The P-12 solenoids are in series and energized above P-12. If Tave decreases below the setpoint or the solenoids lose power, the valves close. However, without power, the solenoids cannot be re-energized.
- B. Incorrect:** The solenoids cannot be bypassed.  
**Plausible:** Operator may believe BYPASS will allow the valves to be opened.
- C. Incorrect:** P-12 solenoids will prevent the valves from opening  
**Plausible:** Operator may believe P-12 only affects Tave operation.
- D. Incorrect:** P-12 solenoids will prevent the valves from opening  
**Plausible:** Operator may believe P-12 does not affect operation unless necessary below 553.

Level: RO&SRO

KA: 041 G2.1 (3.7/3.6)

Lesson Plan Objective: IDE SEQ 8

Source: New

Level of knowledge: comprehension

References:

1. Lesson plan IDE page 6 and 27

**Bank Question: 755****Answer: D**

1 Pt(s)

Which of the following conditions will cause 2RN47A (*RN SUPPLY X-OVER ISOL*) to automatically close?

- A. Phase B on either unit or Emergency low level in either pump house pit.
- B. Unit 2 Phase B or Emergency low level in pump house pit A.
- C. Emergency start of either Unit 2 emergency Diesel generator or Phase B on either unit.
- D. Unit 2 Phase B or Emergency low level in pump house pit B.

---

**Distracter Analysis:**

RN Supply Crossover Isolation Valves (1&2RN47A and 48B)

The RN supply crossover isolation valves are normally open to supply cooling water to the non-essential header. Each valve closes upon a "S<sub>p</sub>" Phase B Isolation signal from its respective unit.

The RN supply crossover isolation valves are also equipped with safety related interlocks to close upon emergency low level in the pump house pits.

Pit A Emergency Low Level will close: 1 and 2 RN48B

Pit B Emergency Low Level will close: 1 and 2 RN47A

- A. **Incorrect:** Phase B on Unit 1 will not cause isolation  
**Plausible:** believes either unit or pit level isolates the non-essential header.
- B. **Incorrect:** emergency low level on pump house pit A will not cause isolation  
**Plausible:** partially correct – if candidate believes 47A isolated due to pit A
- C. **Incorrect:** emergency DG start will not cause isolation  
**Plausible:** partially correct - if the candidate thinks that an emergency start of the DG should result in isolating the non-essential header
- D. **Correct:**

Level: RO Only

KA: 076 K4.06 (2.8/3.2)

Lesson Plan Objective: RN SEQ 12

Source: New

Level of knowledge: memory

References:

1. OP-CN-PSS-RN page 17

**Bank Question: 756****Answer: C**

1 Pt(s)

Unit 1 was operating at 100% power when a LOCA occurred. Given the following events and conditions:

- 0200 = LOCA occurs inside containment, reactor trip
- 0201 = loss of offsite power,
- 0202 = Emergency diesels start and energize emergency buses
- 0203 = Safety Injection occurs
- 0204 = Containment pressure exceeds 1.2 psig
- 0210 = Containment pressure exceeds 3.0 psig
- 0215 = Instrument air decreases to 75 psig

At what point did Instrument Air to containment isolate?

- A. 0200 - 0202
- B. 0203 - 0204
- C. 0205 - 0210
- D. 0211 - 0220

---

**Distracter Analysis:**

- A. **Incorrect:** instrument air isolation is not caused by the LOOP  
**Plausible:** operator believes the valves are closed on LOOP
- B. **Incorrect:** VI isolation is not caused by either the safety injection or the phase "A" isolation signal, St  
**Plausible:** Breathing air isolates on St
- C. **Correct:** VI isolates on a phase B containment isolation signal, Sp
- D. **Incorrect:** VI already isolated  
**Plausible:** If operator feels VI isolates on decreasing pressure (vice the cross tie opening)

Level: RO Only

KA: 078 K3.01 (3.1/3.4)

Lesson Plan Objective: VI LPRO 25

Source: New

Level of knowledge: comprehension

References:

1. Lesson plan VI page 9-10,34
2. OP-CN-ECCS-ISE page 18

**Bank Question: 757****Answer: B**

1 Pt(s)

Units 1 and 2 are operating at 100% power with a normal service water line-up and RN pump 2A running. Given the following conditions and indications:

- Alarm RN PIT SWAP TO SNSWP (1AD-12 E/2) - LIT
- Alarm RN PIT A SCREEN HI D/P (1AD-12 B/2) - LIT
- All RN pumps start.
- RN suction and discharge valves swap to the SNSWP.

Which of the following best describes the cause of this condition?

- A. The Lake Wylie dam failed.
- B. RN pump intake pit "A" screens are clogged.
- C. An RN pump intake pit level indicator failed low.
- D. There was a spurious safety injection actuation.

---

**Distracter Analysis:**

- A. **Incorrect:** low lake level would not actuate level differential alarm  
**Plausible:** event would cause the other conditions.
- B. **Correct:**
- C. **Incorrect:** It takes 2 of 3 instrument failures.  
**Plausible:** if the candidate does not know the coincidence logic.
- D. **Incorrect:** S/I would not cause level alarms.  
**Plausible:** S/I causes some of the other conditions.

Level: RO&SRO

KA: 000062AA2.02(2.9 / 3.6)

Lesson Plan Objective: PSS-RN SEQ12

Source: New

Level of knowledge: comprehension

References:

1. OP-CN-PSS-RN pages 14, 32
2. OP/1/Q/6100/010M 1AD-12 B/3
3. OP/1/A/6100/010M 1AD-12 E/2



**Bank Question: 758****Answer: C**

1 Pt(s) Unit 2 was operating at 50% power. During surveillance testing of CA pump 2A, a large fire started in the CA pump room. Which one of the following best characterizes the plant fire system response to this event?

- A. An alarm will sound in the affected pump cubicle to warn personnel of the electric shock hazard of the pending sprinkler actuation in 1.5 minutes.
- B. An alarm will sound in the control room to notify operators to actuate the affected pump CO<sub>2</sub> system after the personnel have been safely evacuated from the pump room.
- C. An alarm will sound in the affected pump cubicle to warn personnel of the asphyxiation hazard of the pending CO<sub>2</sub> actuation in 6.5 minutes.
- D. An alarm will sound in the control room to notify operators to actuate the affected pump deluge system after the personnel have been safely evacuated from the pump room.

---

**Distracter Analysis:**

- A. **Incorrect:** There is no time delay in sprinkler system actuation.  
**Plausible:** if the candidate does not understand the protection provided for this fire area.
- B. **Incorrect:** the system actuates automatically.  
**Plausible:** Some fire systems are manually actuated.
- C. **Correct:**
- D. **Incorrect:** There is no deluge protection for CA pumps.  
**Plausible:** MFPs have deluge protection.

Level: RO Only

KA: APE 067AK3.02(2.5 / 3.3)

Lesson Plan Objective: SS-RFY SEQ 36

Source: New

Level of knowledge: memory

References:

1. OP-CN-SS-RFY pages 13, 21-22

**Bank Question: 763****Answer: C**

- 1 Pt(s) Unit 2 is responding to a small break LOCA. The operators have entered FR-C.2 (*Response to Degraded Core Cooling*) due to failure of the NI and NV systems to inject. The note at the start of FR-C.2 states:

*Normal conditions for running NC pumps are desired, but NC pumps should not be tripped if normal conditions cannot be established or maintained.*

Which one of the following best describes the operator response and basis if the lower bearing temperature on all four NCP's is 300°F and increasing?

- A. All four NCP's should be tripped to prevent imminent failure of the NCP seals.
- B. All four NCP's should remain in operation to assure rapid depressurization to accumulator injection pressure.
- C. All four NCP's should remain in operation to facilitate two-phase flow through the core.
- D. All four NCP's should be tripped to minimize coolant loss through the break.

---

**Distracter Analysis:**

- A. **Incorrect:** The pumps should not be tripped.  
**Plausible:** This is the basis for NCP trip criteria (>235°F)
- B. **Incorrect:** wrong basis for action.  
**Plausible:** This is the logical consequence of this action, and solves the cause of the problem.
- C. **Correct:**
- D. **Incorrect:** The pumps should not be tripped.  
**Plausible:** This is the reason for tripping NCP's during a LOCA with NV or NI injecting.

Level: RO Only

KA: 000074EK3.08(4.1 / 4.2)

Lesson Plan Objective: EP-FRC SEQ 4

Source: New

Level of knowledge: memory

References:

1. OP-CN-EP-FRC page 5
2. ERG Basis Document FR-C.2 page 1
3. EP/1/A/5000/FR-C.2 page 2
4. Ep/1/A/5000/F-0 page 3

**Bank Question: 764****Answer: D**

1 Pt(s)

Unit 2 was operating at 100% power. At 0200, NCP 2C tripped on an electrical fault. Reactor power is 96% and the reactor trip breakers are closed. The operators implement FR-S.1, (*Response to Nuclear Power Generation/ATWS*).

Given the following plant indications:

<u>Indication</u>	<u>A</u>	<u>B</u>	<u>C</u>	<u>D</u>
2NV-312A	OPEN	OPEN	CLSD	OPEN
2NV-314B	OPEN	CLSD	OPEN	CLSD
2NV- 252A	CLSD	OPEN	CLSD	OPEN
2NV- 253B	CLSD	CLSD	OPEN	CLSD
2NV-236B	CLSD	CLSD	CLSD	CLSD
2NV-188A	OPEN	OPEN	CLSD	OPEN
2NV-189B	OPEN	OPEN	OPEN	CLSD
2NI-9A	CLSD	CLSD	OPEN	CLSD
2NI-10B	CLSD	CLSD	CLSD	OPEN
NV PMP 2A	ON	OFF	OFF	ON
NV PMP 2B	OFF	ON	ON	ON
CHG Flow	78	0	25	140

Which one of the above columns contains the correct plant indications from which it can be concluded that NV system is effectively combating the ATWS event?

- A. Column A indications
- B. Column B indications
- C. Column C indications
- D. Column D indications

---

**Distracter Analysis:**

- A. **Incorrect:** No emergency boration from BAT or FWST – 2NV-236B blocks suction from the BAT.  
**Plausible:** if the candidate is focused on normal NV line-up or doesn't know the emergency boration sources.
- B. **Incorrect:** No charging flow.  
**Plausible:** there's a flow path from the FWST and an NV pump running.

- C. Incorrect:** only 25 gpm (versus 30gpm).  
**Plausible:** there's a flow path from FWST through 2NI-9.
- D. Correct:** - uses B train valves instead of A

Level: RO&SRO

KA: APE 024G4.1(4.3/4.6)

Lesson Plan Objective: EP-FRS SEQ 5

Source: New

Level of knowledge: comprehension

References:

1. OP-CN-EP-FRS page 6
2. EP/1/A/5000/FR-S.1 pages 2-3

**Bank Question: 766****Answer: B**

1 Pt(s)

Unit 2 was operating at 75% power with rod control in automatic. A Rod Control Urgent Failure alarm is received on 2AD-2. Given the following events and conditions:

- |                                       |                            |
|---------------------------------------|----------------------------|
| • Control rod bank D                  | No motion                  |
| • PR instruments                      | decreasing                 |
| • Tave                                | decreasing                 |
| • Tref / Taut deviation               | increasing                 |
| • All control rod groups              | At programmed withdrawal   |
| • All shutdown rod groups             | Fully withdrawn except one |
| • 2AD-2, D/9 "RPI at Bottom Rod Drop" | LIT                        |

Which of the following statements correctly describes the required Reactor Operator actions?

- A. **Verify only one rod dropped and reactor power stabilizes; perform actions of AP/14 (*Control Rod Misalignment*) as directed by the SRO.**
- B. **Verify only one rod dropped or misaligned; switch rod control to manual; and perform actions of AP/14 as directed by the SRO.**
- C. **Verify only one rod dropped; ensure CRD Bank Select in Manual; and perform actions of AP/15 (*Rod Control Malfunction*) as directed by the SRO.**
- D. **Verify no rod movement; switch rod control to manual; and perform actions of AP/15 as directed by the SRO.**

---

**Distracter Analysis:**

- A. **Incorrect:** Does not switch rod control to manual.  
**Plausible:** Reasonable actions if the candidate forgets the immediate actions.
- B. **Correct:** Immediate actions of AP/14, Case II.
- C. **Incorrect:** Does not verify another misalignment, and wrong AP.  
**Plausible:** If the candidate misses the misalignment verification, and/or chooses the wrong AP based on its title.
- D. **Incorrect:** Correct answer for the wrong AP.  
**Plausible:** If the candidate chooses AP/15 based on its title.

Level: RO&SRO

KA: APE 003G2.4.1(4.3/4.6)

Lesson Plan Objective: IRE SEQ 20

Source: New

Level of knowledge: comprehension

References:

1. OP-CN-IC-IRE pages 10, 13, 21
2. AP/1/A/5500/14 page 6

**Bank Question: 767****Answer: D**

1 Pt(s)

Unit 1 is responding to a small-break loss of coolant accident inside containment. Given the following events and conditions:

- Operators have implemented E-0 (*REACTOR TRIP*), and E-1 (*LOSS OF REACTOR OR SECONDARY COOLANT*) through step 10.
- Bus 1ETB is de-energized
- NI pump 1A has failed.
- Containment hydrogen concentration is 7%
- The TSC has recommended purging containment to reduce hydrogen concentration to 3.5% before starting the recombiners.

Which one of the following statements correctly describes the method for performing this evolution to control the off-site dose?

- A. **Containment air is exhausted to the auxiliary building where it is filtered prior to release to the unit vent stack.**
- B. **Containment air is exhausted to the containment air release system where it is filtered prior to release to the unit vent stack.**
- C. **Containment air is exhausted to the annulus where it is continuously recirculated through the annulus ventilation filters.**
- D. **Containment air is exhausted to the annulus where it is continuously recirculated and filtered prior to release to the unit vent stack.**

---

**Distracter Analysis:**

- A. **Incorrect:** Air is released to the annulus area.  
**Plausible:** This could be a way to filter the release if the air was vented to the auxiliary building.
- B. **Incorrect:** Air is released to the annulus area.  
**Plausible:** If the candidate confuses the VQ and VY systems.
- C. **Incorrect:** The air is released to the vent stack.  
**Plausible:** If the candidate does not remember that VE releases to the stack to maintain negative pressure in the annulus.
- D. **Correct:**

Level: RO&amp;SRO

KA: EPE 009G2.3.9(2.5/3.9)



Lesson Plan Objective: VX SEQ 4

Source: New

Level of knowledge: comprehension

References:

1. OP-CN-CNT-VE pages 5- 7
2. OP-CN-CNT-VX pages 6, 9-10
3. OP-CN-CNT-VQ pages 7, 9
4. OP-CN-PSS-VA page 23

**Bank Question: 768****Answer: B**

1 Pt(s)

Unit 1 is at 25% power when a loss of main feedwater occurs. Given the following events and conditions:

- The main turbine trips.
- CA pumps 1A and 1B start automatically.
- CA flow to the S/Gs was throttled when the S/G A N/R level was greater than 11%.
- Control valve 1CA-62A to S/G A cannot be closed.

Which one of the following statements correctly describes the consequences of operator failure to respond to this malfunction?

- A. The reactor will be overcooled causing a criticality concern.
- B. S/G A will be overfilled causing a main steam pipe stress concern.
- C. The reactor will be overcooled causing a pressurized thermal shock concern.
- D. S/G A will be overfilled causing a feed ring thermal stress concern.

---

**Distracter Analysis:**

- A. **Incorrect:** The reactor remains critical and maintains programmed Tave.  
**Plausible:** could be a concern if the reactor tripped.
- B. **Correct:**
- C. **Incorrect:** The reactor does not see a significant cooldown.  
**Plausible:** could be a concern if the reactor tripped.
- D. **Incorrect:** The feed rings are designed to withstand the shock of CA flow.  
**Plausible:** A possible concern if the candidate does not understand S/G design.

Level: RO&amp;SRO

KA: APE 054AK3.03(3.8/4.1)

Lesson Plan Objective: SM LPRO 25

Source: New

Level of knowledge: comprehension

References:

1. OP-CN-STM-SM pages 16-17
2. AP/1/A/550/06 page 2
3. OP-CN-TA-PTS pages 18, 20, 22
4. OP-CN-CF-CA pages 10,11,15

**Bank Question: 769****Answer: C**

1 Pt(s)

Unit 1 is recovering from a loss of secondary coolant accident. Safety injection initiated properly. A total loss of feedwater has caused the operators to implement FR-H.1, (*Loss of Secondary Heat Sink*). Given the following plant conditions:

- NCS Pressure 2335psig
- NCS Temperature 565°F
- S/G 1A, 1B, 1C Pressure 1180psig
- S/G 1A, 1B, 1C Level (WR) 2%
- S/G 1D Pressure 100psig
- S/G 1D Level (WR) 35%
- VI system pressure 10psig
- Containment pressure 3.4psig

Which one of the following actions is initially required to assure the maintenance of adequate core cooling?

- A. **Depressurize S/G 1A, 1B, and 1C to allow feeding the S/G using the condensate system.**
- B. **Reset the CAPT and align it to feed S/G's 1A, 1B and 1C.**
- C. **Open 1NC-32B (PZR PORV) and 1NC-34A (PZR PORV) using nitrogen pressure.**
- D. **Reset safety injection and containment phase "A" isolation signals to re-establish instrument air pressure to open 1NC-32B and 1NC-34A.**

---

**Distracter Analysis:**

- A. **Incorrect:** Can't feed dry S/G's  
**Plausible:** if the candidate does not recognize dry S/G criteria met, this is one FR-Z.1 recovery method.
- B. **Incorrect:** Can't feed dry S/G's  
**Plausible:** if the candidate does not recognize dry S/G criteria met, this is one FR-Z.1 recovery method.
- C. **Correct:**
- D. **Incorrect:** Must reset S<sub>p</sub> to reopen VI valves.  
**Plausible:** if the candidate thinks that VI is a phase "A" isolated system.

Level: RO&SRO

KA: WE05AA1.1(4.1/4.0)

Lesson Plan Objective: FRH LPRO 5

Source: New

Level of knowledge: analysis

References:

1. OP-CN-EP-FRH page 6

**Bank Question: 770****Answer: B**

1 Pt(s) Unit 1 is starting-up in mode 1. Given the following conditions and events:

- Battery charger 1ECB is out of service.
- Battery charger 1ECS is supplying bus 1EDB.
- Annunciator 1AD-11, H/3 (125VDC ESS PWR CHANNEL C TROUBLE) alarms due to the loss of power from battery charger 1ECC.
- The battery charger 1ECC output breaker is faulted.

Which of the following describes operator action needed in response to these conditions?

**REFERENCES PROVIDED: - Tech Spec's 3.8.4, 3.8.7, 3.8.9**

- A. Switch inverter 1EIC to the regulated power supply, and be in mode 3 within 6 hours.**
- B. Cross-tie bus 1EDC to bus 1EDA, and fix one of the chargers within 10 days.**
- C. Switch inverter 1EIC to the regulated power supply, and fix one of the chargers within 24 hours.**
- D. Cross-tie bus 1EDC to bus 1EDA, and fix one of the chargers or be in mode 3 within 6 hours.**

---

**Distracter Analysis:**

- A. Incorrect:** they would have 14 hours if the buses were not cross-tied.  
**Plausible:** This is a possible action if the buses could not be cross-tied.
- B. Correct:**
- C. Incorrect:** They still have to cross-tie or shutdown within 14 hours..  
**Plausible:** If the candidate focuses on inverter vs DC source TS.
- D. Incorrect:** the busses can be cross-tied for 10 days.  
**Plausible:** If the candidate thinks more than one train is affected.

Level: RO&SRO

KA: APE 058AA1.01(3.4/3.5)

Lesson Plan Objective: EPL LPRO 9, 19

Source: New

Level of knowledge: memory

References:

1. OP-CN-EL-EPL page 19
2. Tech Spec's 3.8.4, 3.8.7, 3.8.9 - PROVIDED

**Bank Question: 771****Answer: D**

1 Pt(s) Unit 1 is operating at full power and Unit 2 is refueling. Unit 1 is releasing a waste gas decay tank with significant packing leak on isolation valve 1WG-160, (*WG Decay Tank Outlet to Unit Vent Control*).

Which one of the following statements correctly describes the automatic and manual actions required to assure that the release is contained and filtered?

- A. **1EMF-50 (*Waste Gas Disch*) automatically stops the auxiliary building ventilation unfiltered exhaust fans.**
- B. **1EMF-41 (*Aux Bldg Vent Hi Rad*) automatically closes valve 1WG-160. The operators must manually stop the auxiliary building ventilation unfiltered exhaust fans.**
- C. **1EMF-37 (*Unit Vent Iodine*) automatically stops the auxiliary building ventilation unfiltered exhaust fans. The operators must manually close 1WG-160.**
- D. **1EMF-35 (*Unit Vent Part Hi Rad*) automatically closes valve 1WG-160 and stops the auxiliary building ventilation unfiltered exhaust fans.**

---

**Distracter Analysis:**

- A. **Incorrect:** EMF-50 does not secure the VA fans.  
**Plausible:** A reasonable response to a leak in the Aux Bldg.
- B. **Incorrect:** WG-160 does not automatically close  
**Plausible:** if the candidates do not recognize that this is not an automatic action. EMF-41 is an ARM that will alarm under these conditions.
- C. **Incorrect:** EMF-37 auto trips WG-160 closed.  
**Plausible:** partially correct – EMF 37-stops the VA fans.
- D. **Incorrect:** EMF-35 can secure 1WG-160 and VA fans, but will not see an abnormal release in this case.

Level: RO&SRO

KA: APE 060AK3.02(3.3/3.3)

Lesson Plan Objective: VA LPRO 9

Source: New



Level of knowledge: analysis

References:

1. OP-CN-PSS-VA pages 8, 13, 14, 16, 23
2. OP-CN-WE-EMF pages 9, 10, 17-18

**Bank Question: 773****Answer: D**

1 Pt(s)

You are manually preparing a Red Tag Removal and Restoration (R&R). You notice a red stamp on the electrical one-line drawing you are using that reads 'Interim As Built' along with CNEE-05163 written in the space next to the stamp.

What actions are required prior to using this drawing for the R & R?

- A. Note the modification number on the R & R Record Sheet for tracking purposes.
- B. Go to the WCC to verify that the modification changes do not affect the R & R.
- C. Notify the NSM coordinator that the drawing needs to be updated prior to hanging the tags.
- D. Go to the Control Room to verify that the modification changes do not affect the R & R.

---

**Distracter Analysis:**

- A. **Incorrect:** the drawing accuracy must be verified  
**Plausible:** a reasonable answer if the notation were only administrative
- B. **Incorrect:** the WCC does not maintain as-built information for electrical drawings.  
**Plausible:** WCC does maintain flow diagrams.
- C. **Incorrect:** the tags are hung after verification of drawing accuracy  
**Plausible:** a reasonable action if the right answer is not known.
- D. **Correct:**

Level: RO&SRO

KA: G2.1.24(2.8/3.1)

Lesson Plan Objective: ADM-NSO2 SEQ 2

Source: New

Level of knowledge: memory

References:

1. OMP 2.10 pages 3, 5-6

2. OP-CN-AD-NSO2 page 8

**Bank Question: 774****Answer: C**

- 1 Pt(s)      Unit 2 is in a refueling outage with core alternations in progress. IAE is performing a calibration of source range detector N-31. Which of the following best describes the actions required when BDMS channel B fails low?

**REFERENCES PROVIDED: Tech Spec 3.9.2**

- A.      Suspend core alterations until the RMW pump capacity is verified to be less than COLR limits.
- B.      Core alterations may continue because one channel of BDMS and channel N-32 remain operable.
- C.      Suspend core alterations and verify that unborated water sources are isolated.
- D.      Core alterations may continue because channel N-32 remains operable and the RMW pump capacity was verified to be less than COLR limits during refueling prerequisites.

---

**Distracter Analysis:**

- A.      **Incorrect:** the unborated sources must be verified closed.  
**Plausible:** this would be true if both SR instruments were in service.
- B.      **Incorrect:** core alterations must be suspended, must have both SR's or both BDMS.  
**Plausible:** a reasonable approach considering many plants don't have to have BDMS.
- C.      **Correct:**
- D.      **Incorrect:** core alterations must be suspended.  
**Plausible:** an even more reasonable approach considering many plants don't have to have BDMS

Level: RO Only

KA: G2.2.22 (3.4 / 4.1)

Lesson Plan Objective: FH-FHS SEQ 6

Source: New

Level of knowledge: memory

References:

1. OP-CN-FH-FHS page 21
2. Tech Spec 3.9.2

**Bank Question: 775****Answer: B**

1 Pt(s)

An NCP motor has been removed from containment to the electrical maintenance shop for overhaul. Health Physics surveyed the motor with the following results:

- General radiation levels in the shop work area -- <0.1 mR/hr
- Radiation levels on contact with the motor -- 0.3 mR/hr
- Radiation level on contact with the pump shaft coupling -- 100 mR/hr
- External loose surface contamination -- 400 dpm/cm<sup>2</sup>
- Internal loose surface contamination -- 750 dpm/cm<sup>2</sup>

Which of the following statements correctly describes the controls, if any, that are required to minimize the exposure to the workers performing the overhaul?

- A. A standing RWP is required because this is a special operation requiring decontamination.
- B. An RWP is required because the operation requires temporary shielding.
- C. No RWP is required because the pump is not inside the RCA.
- D. A standing RWP is required because this is a special operation requiring temporary shielding.

---

**Distracter Analysis:**

- A. **Incorrect:** an RWP is required and decontamination is not an option.  
**Plausible:** if the candidate focuses on a blanket RWP for shop work or S for special.
- B. **Correct:**
- C. **Incorrect:** An RWP is required.  
**Plausible:** If the candidate assumes no controls are needed outside the radiological controlled area.
- D. **Incorrect:** An RWP is required.  
**Plausible:** if the candidate focuses on a blanket RWP for shop work or S for special.

Level: RO&SRO

KA: G2.3.10 (2.9/3.3)

Lesson Plan Objective: RAD-HP LPRO 9, 11

Source: New

Level of knowledge: comprehension

References:

1. OP-CN-RAD-HP pages 18, 25, 28

**Bank Question: 776****Answer: C**

1 Pt(s)

Unit 1 was operating at 100% when a LOCA with loss of offsite power occurs. One EDG fails to start. The operators are entering EP/E-1, (*Loss of Reactor or Secondary Coolant*).

Given the following critical safety function status indications:

- Core Cooling – ORANGE
- Subcriticality – GREEN
- Containment – RED
- Inventory - GREEN
- Heat Sink – RED
- Integrity – GREEN

Which one of the following describes the highest priority problem, and the appropriate operator action?

- A. Core cooling; Transition to FR-C.2, (*Response to Degraded Core Cooling*).
- B. Core cooling; Transition to FR-C.1, (*Response to Inadequate Core Cooling*).
- C. Heat Sink; Transition to FR-H.1, (*Response to Loss of Secondary Heat Sink*).
- D. Containment; Transition to FR-Z.1, (*Response to High Containment Pressure*).

---

**Distracter Analysis:**

- A. **Incorrect:** heat sink is the highest priority RED  
**Plausible:** if the candidate picks the first item requiring transition from E-1.
- B. **Incorrect:** heat sink is the highest priority RED  
**Plausible:** if the candidate picks the first RED condition and does not recognize that the wrong procedure is listed.
- C. **Correct:**
- D. **Incorrect:** heat sink is the highest priority RED  
**Plausible:** if the candidate picks the first RED condition.

Level: RO&SRO

KA: G2.4.17 (3.1/3.8)



Lesson Plan Objective: EP-INTRO LPRO 1

Source: Bank Ques\_776

Level of knowledge: memory

References:

1. OP-CN-EP-INTRO page 6, 8
2. OMP 1-7 pages 9-10

**Bank Question: 779****Answer: C**

1 Pt(s)

The crew is realigning a Control Bank "C", Group 1 rod in accordance with OP/1/A/6150/008, (Rod Control), Enclosure 4.6, (Rod Retrieval (>12 Steps Misaligned)).

The RO has been directed to operate the lift coil disconnect switches in accordance with the enclosure.

The RO reports that the lift coil disconnect switches for the unaffected rods in Control Bank "C" Group 1 are in the DISCONNECTED position and that the misaligned rod switch is in the CONNECTED position.

Which of the following additional actions, if any, must be performed to correctly realign the Bank C control rod?

- A. Return the unaffected group 1 rods to CONNECTED and place the misaligned rod disconnect switch to DISCONNECTED.
- B. Place all the Control Bank A, B, D and Control Bank C group 2 rods in DISCONNECTED.
- C. Place all Control Bank "C" Group 2 rods in DISCONNECTED.
- D. No additional actions necessary, continue with the misaligned rod alignment.

---

**Distracter Analysis:** The next step in the procedure reads:

2.10 Disconnect all lift coils in the affected bank, except for the affected rod, by placing the control rod disconnect switches in the "DISCONNECTED" position.

- A. **Incorrect:** This action would further exacerbate the misaligned rod  
**Plausible:** operator feels the action taken is backwards from what's required
- B. **Incorrect:** not correct to disconnect rods that are not part of bank C  
**Plausible:** operator feels all the control bank rods, except the misaligned one should be in disconnect.
- C. **Correct:** if group 2 is not disconnected – then they will move when the misaligned rod is realigned.  
**Plausible:** all the control rods in the bank, group 1 and 2, except the misaligned one should be in disconnect
- D. **Incorrect:** this will cause group 2 of bank C rods to move.  
**Plausible:** operator believes the actions taken are correct.

Level: RO&SRO

KA: APE 005 EA1.2 (3.7/3.5)

Lesson Plan Objective: IRE SEQ 10, 17

Source: New

Level of knowledge: comprehension

References:

1. OP-6150/008
2. Lesson IRE page 12

**Bank Question: 780****Answer: A**

1 Pt(s)

Unit 1 was operating at 100% power when a reactor trip occurred. The crew is verifying natural circulation in ES-0.1 (*Reactor Trip Response*).

Given the following events and conditions:

- NC WR Thot 580°F, stable
- NC WR Tcold 555°F, stable
- NC Pressure 1920 psig
- Core Exit Thermocouples 585°F, stable
- Steam Generator pressure 1090 psig, stable
- Steam Generator narrow range level off scale low
- Steam Generator wide range level 70%
- CA flow 400 gpm per steam generator

Which one of the following statements correctly describes the status of natural circulation?

**REFERENCES PROVIDED: ES-0.1 Encl 3 page 21**

- A. All conditions are met; natural circulation is established.
- B. Natural circulation is not established; subcooling is insufficient.
- C. Natural circulation is not established; heat sink is insufficient.
- D. Natural circulation is not established; NC delta T is insufficient.

---

**Distracter Analysis:**

**Objective:** Explain the bases of Enclosure 1 (Foldout Page) actions of EP/1/A/5000/ES-0.1 (*Reactor Trip Response*).

- A. **Correct:**
- B. **Incorrect:** subcooling is greater than 0 °F  
**Plausible:** operator error determining subcooling
- C. **Incorrect:** heat sink is satisfactory – have SG WR level = 70% with 400 gpm CA flow  
**Plausible:** operator believes NR level required for heat sink
- D. **Incorrect:**  
**Plausible:** operator looks for normal at power delta T

Level: RO Only

KA: WE 09 EK2.1 (3.2/3.4)

Lesson Plan Objective: EP1 SEQ 15

Source: New

Level of knowledge: analysis

References:

1. OP-CN-EP-EP1 pages 6-7
2. ES-0.1 page 21

**Bank Question: 782****Answer: B**

1 Pt(s)

Which one of the following statements correctly describes the purpose of the time delay associated with the "KC HX A RN Outlet Flow-Lo" Annunciator on 1AD-12 following a safety injection signal?

- A. To allow sufficient time for the 1A RN pump to develop sufficient flow following a sequencer start.
- B. To allow sufficient time for 1RN-291 (KC HX 1A OUTLET THROTTLE) to stroke open.
- C. To allow sufficient time for 1RN-287A (KC HX 1A INLET ISOL) to stroke open.
- D. To allow sufficient time for the 1KC-C37A (TRAIN A MINIFLOW ISOL) to stroke open.

---

**Distracter Analysis:**

- A. **Incorrect:** the RN pump starts without a time delay  
**Plausible:** pumps start on Ss signal.
- B. **Correct:** The "KC HX A(B) RN Outlet Flow-Lo" Annunciator (OBJ #15,) is interlocked with an Ss signal such that the alarm will only be enabled while a Ss signal is present. It also provides a 72 second time delay after the Ss signal to allow the RN valve to stroke full open.
- C. **Incorrect:** These valves do not move.  
**Plausible:** If the candidate confuses the inlet isolation valves with the throttle valves
- D. **Incorrect:** These valves do not affect RN flow to the heat exchanger.  
**Plausible:** if the candidate confuses the miniflow isolation valves with the heat exchanger isolation valves – some miniflow valves have automatic actions.

Level: RO&amp;SRO

KA: APE 026 AA1.07 (2.9/3.0)

Lesson Plan Objective: KC SEQ 15

Source: New

Level of knowledge: memory

References:

1. KC lesson page 8 of 26
2. OP/1/A/6100/010M 1AD-12 D/2

**Bank Question: 783****Answer: C**

1 Pt(s)

Unit 2 is operating at 100% power. Given the following events and conditions:

- Pressurizer pressure control is in automatic
- One set of backup Heaters is energized in "ON"
- Actual pressurizer pressure is 2270 psig

The pressurizer Master Pressure Controller malfunctions and the setpoint is step changed from 2235 psig to 2335 psig.

Which one of the following statements correctly describes the initial automatic responses in the pressurizer Pressure Control System as a result of this failure?

- A. PORV NC-34A opens, Spray Valves open, and pressurizer Heaters de-energize
- B. PORV NC-32 and NC-36 open, Spray Valves open, and pressurizer Heaters de-energize
- C. Spray Valves close and pressurizer Heaters energize
- D. All PORVs remain closed, Spray Valves open and pressurizer Heaters de-energize

---

**Distracter Analysis:**

- A. **Incorrect:**  
**Plausible:** operator believes that the setpoint of 2335 psig will open the PORV
- B. **Incorrect:**  
**Plausible:** operator believes setpoint increase causes spray valves to open, and the other two PORV respond to a "high" pressure condition.
- C. **Correct:**  
**Plausible:** system responds to "low" pressure. Heaters on, spray valves that were open, will close.
- D. **Incorrect:**  
**Plausible:**

Level: RO&SRO

KA: APE 027 AK2.03 (2.6/2.8)



Lesson Plan Objective: IPE SEQ 8, 10

Source: New

Level of knowledge: analysis

References:

1. IPE pages 8, 14, 15

**Bank Question: 786****Answer: B**

1 Pt(s)

Which of the following conditions would be consistent with excessive number 2 seal leakage on an operating NCP?

- A. 1AD-7, C/2, "NCP #1 SEAL LEAKOFF LO FLOW" and 1AD-7, B/3 "NCP #1 SEAL LO D/P" annunciators lit.
- B. 1AD-7, C/2, "NCP #1 SEAL LEAKOFF LO FLOW" annunciator lit.
- C. 1AD-7, C/1, "NCP #1 SEAL LEAKOFF HI FLOW" and 1AD-7, B/3, "NCP #1 SEAL LO D/P" annunciators lit.
- D. 1 AD-7, B/1 "NCP #1 SEAL OUTLET HI TEMP" annunciator lit.

---

**Distracter Analysis:**

- A. **Incorrect:** C/2 cause is #1 seal damage or #2 seal failure, B/3 cause is loss of #1 seal.  
**Plausible:** operator believes that the failure of the number 2 seal will cause seal d/p to decrease.
- B. **Correct:**  
**Plausible:** Number 2 seal leakoff increasing will rob flow from the number 1 seal, leakoff will decrease.
- C. **Incorrect:** cause for C/1 is damaged or cocked #1 seal.  
**Plausible:** as stated previously, operator feels the failure will decrease number 1 d/p.
- D. **Incorrect:** B/1 caused by insufficient cooling water.  
**Plausible:** operator feels the failure will cause a seal outlet temperature to increase.

Level: RO&amp;SRO

KA: APE 015 AK2.07 (2.9/2.9)

Lesson Plan Objective: NCP SEQ 12

Source: New

Level of knowledge: comprehension

References:

1. OP/1/B/6100/010H annunciators B/1, B/3, C/1, C,2



**Bank Question: 788****Answer: D**

1 Pt(s)

Unit 1 was operating at 100% power when a total loss of offsite and onsite AC electrical power occurred. Given the following events and conditions:

- The crew is performing the actions of ECA-0.0 (*LOSS OF ALL AC POWER*).
- Power has not been restored.
- The operator reports core exit thermocouples read 1200°F and increasing.

Which one of the following statements correctly describes the actions the crew should take?

- A. **Immediately go to FR-C.1 (*RESPONSE TO INADEQUATE CORE COOLING*).**
- B. **Remain in ECA-0.0 until after power is restored to at least one emergency bus then transition to FR-C.1**
- C. **Complete ECA-0.0 and when directed to implement monitoring CSF status trees in the appropriate recovery procedure, verify a valid RED path exists and transition to FR-C.1.**
- D. **Immediately transition to EG/1/A/CSAM/SACRG1 (*SEVERE ACCIDENT CONTROL ROOM GUIDELINE INITIAL RESPONSE*).**

---

**Distracter Analysis:**

Step 30, IF core exit temperatures greater than 1200 °F AND increasing, THEN GO TO EG/1/A/CSAM/SACRG1 (Severe Accident Control Room Guideline Initial Response).

- A. **Incorrect:** must immediately transition to SACRG1  
**Plausible:** operator knows an immediate transition is necessary but believes C.1 is appropriate.
- B. **Incorrect:** must immediately transition to SACRG1  
**Plausible:** operator believes one emergency bus must be restored prior to addressing the inadequate core cooling condition.
- C. **Incorrect:** must immediately transition to SACRG1  
**Plausible:** operator follows the normal rules of usage regarding Red and Orange paths in ECA-0.0
- D. **Correct:** efforts to adequately cool the core have failed, SAMG usage is appropriate

Level: RO&SRO

KA: APE 055 EA1.01 (3.7/3.9)

Lesson Plan Objective: CRG-1 SEQ 2

Source: New

Level of knowledge: memory

References:

1. ECA-0.0 page 23 step 30
2. OP-CN-EP-CRG-1 page 6

**Bank Question: 789****Answer: C**

1 Pt(s)

Unit 1 was shutdown for refueling in mode 6.

IAE was calibrating set points on various area radiation monitors when the following occurred:

- 1WL-867A, (*VUCDT CONT ISOL*) closed
- 1WL-825A, (*CONT SMP PMPS DISCH CONT ISOL*) closed

Which one of the following EMFs could have correctly caused this action?

- A. 1EMF-49(L) (*Liquid Waste Discharge Monitor*)
- B. 1EMF-52 (*Clean Area Floor Drain Monitor*)
- C. 1EMF-53(A) (*Containment Hi Range Monitor*)
- D. 1EMF-54 (*Unit Vent Hi Range Monitor*)

---

**Distracter Analysis:**

- A. **Incorrect:** does not automatically isolate 1WL-867A and 1WL-825A  
**Plausible:** 1EMF-49 provides automatic isolation signal to 1WL-124 to terminate a liquid release
- B. **Incorrect:** does not automatically isolate 1WL-867A and 1WL-825A  
**Plausible:** 1EMF-52 automatically diverts flow from turb bldg sump to ND and NS sump.
- C. **Correct:**
- D. **Incorrect:** does not automatically isolate 1WL-867A and 1WL-825A  
**Plausible:** Automatically trips sample pump supplying EMF-35, 36 and 37

Level: RO&amp;SRO

KA: SYS 072A4.01(3.0\*/3.3)

Lesson Plan Objective: WE-EMF LPRO 2

Source: New

Level of knowledge: memory

References:

1. OP-CN-WE-EMF page 11, 12
2. OAC alarm response

**Bank Question: 790****Answer: C**

1 Pt(s)

Unit 1 is operating at 100% power and is preparing to discharge a waste monitor tank (WMT). Given the following events and conditions:

- IAE has been unable to adjust the 1EMF-49 (*LIQUID WASTE DISCH*) to the required trip 2 setpoint specified on the discharge permit.
- 1EMF-49 operates properly in all other calibration tests
- 1EMF-57, (*MONITOR TANK BUILDING LIQUID DISCHARGE*) is operable
- Secondary coolant activity is 0.005 micro curies/gram dose equivalent Iodine 131.

Which one of the following statements correctly describes the complete set of actions that must be completed before releasing contents of the WMT?

**REFERENCES PROVIDED: - SLC 16.11-2****A.**

- The release may proceed for up to 30 days provided that the flow rate is estimated once every four hours.
- 1EMF-57 can be used to monitor the release path.

**B.**

- The release may proceed for up to 30 days provided that the flow rate is estimated once every four hours.
- A dedicated operator must be stationed at 1EMF-49 to monitor the release rates and ensure that the setpoint is not exceeded.

**C.**

- Two independent analyses of the WMT contents must be performed prior to the start of the release.
- The discharge permit calculations and the release path valve line-up must be independently verified.

**D.**

- The release may proceed for up to 30 days provided that grab samples are taken once per 24 hours.
- 1EMF-57 can be used to monitor the release path.

---

**Distracter Analysis:****A.**

**Incorrect:** need to IV the calculations and the release path.

**Plausible:** Partially correct – This is part of action statement D – the candidate may mistakenly refer to 3.d (monitor tank building waste liquid effluent line) -which is the discharge line in use. 1EMF-57 is used to monitor the release.



- B. Incorrect:** Cannot use a dedicated operator to monitor the release – need to IV the calculations and the release path and two analyses of RMT are required.  
**Plausible:** This is part of action statement D – the candidate may mistakenly refer to 3.d (monitor tank building waste liquid effluent line) -which is the discharge line in use.
- C. Correct:** action statement C in SLC 16.11-2
- D. Incorrect:** Grab sample are not required - need to IV the calculations and the release path and two analyses of WMT are required.  
**Plausible:** First action is the corrective action for loss of 1EMF-31 in the same SLC table. If the candidate refers to the wrong corrective action, this is possible. This is action statement E of the SLC –may select this statement if the candidate mistakenly references 1.b – 1EMF-57 is used to monitor the release.

Level: RO Only

KA: SYS 068A2.04 (3.3 / 3.3)

Lesson Plan Objective: WE-WL SEQ 16

Source: New

Level of knowledge: comprehension

References:

1. SLC 16.11-2 - PROVIDED

**Bank Question: 791****Answer: B**

1 Pt(s)

Unit 1 was operating at 100% power when a reactor trip occurred on low pressurizer pressure. Given the following events and conditions:

- Main steamlines isolated
- Safety injection actuated
- NC temperature initially decreased to 400°F then increased rapidly
- NC pressure initially decreased to 1700 psig then increased rapidly
- Pressurizer level initially decreased to 0% then increased rapidly
- Containment temperature is 100°F
- Pressurizer tailpipe temperatures 120°F
- Containment EMF trip 1 lights are dark
- All steam generator pressures approximately 700 psig and stable
- All steam generator narrow range levels are off scale low, wide range levels are increasing

Based on these indications, which of the following events has occurred?

- A. Pressurizer safety or PORV has failed open.
- B. Steam line break downstream of the MSIVs.
- C. Small break LOCA
- D. Steam line break upstream of the MSIVs.

---

**Distracter Analysis:**

- A. **Incorrect:** tailpipe temps are normal, as is containment parameters  
**Plausible:** candidate misinterprets the pressure and temperature response and holds to the increasing pressurizer level.
- B. **Correct:** when isolated, NC pressure and temperature will begin to increase.
- C. **Incorrect:** normal containment conditions  
**Plausible:** operator believes NI flow is causing the system to recover.
- D. **Incorrect:** system would not recover as described  
**Plausible:** candidate confuses upstream and downstream of the MSIVs

Level: RO&SRO

KA: APE 040 AA1.12 (4.2/4.2)

Lesson Plan Objective: EP1 SEQ 23

Source: New

Level of knowledge: analysis

References:

1. E-0, steps 24, 25, 26

**Bank Question: 792****Answer: C**

1 Pt(s)

Unit 1 is operating at 100% power. The spare battery charger (1ECS) is being aligned to replace charger 1ECA.

How would the operators know if 1ECS was being supplied by motor control center 1EMXJ in this lineup?

- A. **1AD-11 H/1 (125 VDC ESS PWR CHANNEL A TROUBLE) will clear when the 1EDA tie breaker to 1EDC is closed.**
- B. **1AD-11 H/1 (125 VDC ESS PWR CHANNEL A TROUBLE) will alarm when the 1EDS output breaker to 1EDA/C is closed.**
- C. **1AD-11 I/6 (STBY CHARGER 1ECS INPUT/OUTPUT TRAINS X-CONNECTED) will alarm when the 1EDS output breaker to 1EDA/C is closed.**
- D. **1 SI-14 STANDBY CHARGER ECS TRAIN A BKRS CLOSED status light will light when the 1EDA tie breaker to 1EDC is closed.**

---

**Distracter Analysis:**

- A. **Incorrect:** the 125VDC trouble annunciator will light.  
**Plausible:** this would eliminate the CR alarm caused when the spare charger is in service.
- B. **Incorrect:** the trouble annunciator will alarm when the 1EDA tie breaker is closed.  
**Plausible:** if the candidate confuses this annunciator with the cross-train alarm.
- C. **Correct:**
- D. **Incorrect:** the status light for B train will light when 1EMXJ power to 1ECS is aligned.  
**Plausible:** if the candidate does not understand the power supply status light monitoring.

Level: RO&SRO

KA: G2.1.31(4.2 / 3.9) aware of the control room

Lesson Plan Objective: EL-EPL SEQ 3

Source: New

Level of knowledge: comprehension

References:

1. OP-CN-EL-EPL pages 9, 15-16, 24
2. ARP 1AD-11; I/6
3. ARP 1AD-11; H/1

**Bank Question: 793****Answer: D**

1 Pt(s) Unit 2 is operating at 100% power with all rods out.

An operator notices that one core exit thermocouple for quadrant II on the plasma display indicates 2200°F.

Which of the following correctly describes a reason for this thermocouple to be much higher than the other thermocouples?

- A. The thermocouple reference junction temperature has increased.
- B. The thermocouple reference junction temperature has decreased.
- C. The thermocouple measuring junction has an open circuit.
- D. The thermocouple measuring junction has shorted.

---

**Distracter Analysis:** The voltage across a thermocouple junction increases as the temperature of that junction increases. A shorted measuring junction will cause the temperature to fail high.

- A. **Incorrect:** the temperature measured is based on the difference on voltage between the reference junction (at 165 °F) and the T/C. If the reference junction is heated above 165 °F, then the voltage difference will decrease and the temperature signal will decrease.  
**Plausible:** if the candidate reverses the effects
- B. **Incorrect:** the temperature measured is based on the difference on voltage between the reference junction (at 165 °F) and the T/C. If the reference junction is cooled below 165 °F, then the voltage difference will increase and the temperature signal will increase. However, the increase of 1600 °F is not possible because the reference junction temperature cannot be cooled enough.  
**Plausible:** the effect of a decrease in reference junction temperature will be to make the measured temperature increase.
- C. **Incorrect:** an open measuring junction causes the temperature indication to fail LOW not high.  
**Plausible:** if candidate believes that like an RTD and open causes a high reading
- D. **Correct:** a shorted thermocouple causes the removal of the difference in EMF, the TC will read high.

Level: RO&SRO

KA: 017 A2.01 (3.1/3.5)

Lesson Plan Objective: IG SEQ 2

Source: New

Level of knowledge: comprehension

References:

1. OP-CN-SS-IG page 10
2. OP-CN-TA-AM page 26
3. OP-CN-PS-CCM page 15

**Bank Question: 794****Answer: C**

1 Pt(s)

A large fire has enveloped Unit 1 B-train blackout switchgear room. As the fire brigade prepares to attack the fire with hoses, the control room isolates the B-train blackout bus electrically.

Which of the following correctly describes the impact of these events and conditions on the plant's ability to combat this fire?

- A. The fire brigade will be able to spray down the fire because fire hoses can be pressurized from the A-train fire header.
- B. The fire brigade will be able to spray down the fire but with reduced fire main capacity. Only one of the fire pumps will be available from an operable power supply.
- C. The fire brigade will be able to spray down the fire. Two of the fire pumps will be available from an operable power supply.
- D. The fire brigade will not be able to spray down the fire. The blackout bus provides power to the fire pumps.

---

**Distracter Analysis:**

- A. **Incorrect:** there is only one fire main with redundant pumps.  
**Plausible:** if the candidate thinks that the fire main has train separation.
- B. **Incorrect:** there will be 2 pumps running, not only one.  
**Plausible:** if the candidate thinks both Unit 1 fire pumps are powered from the same bus.
- C. **Correct:** 3 fire pumps are powered from 1TC, 1FTB and 2FTA – only 1FTB is deenergized.
- D. **Incorrect:** there will be water available.  
**Plausible:** if the candidate does not know that redundant blackout power supplies are available.

Level: RO Only

KA: SYS 086K3.01(2.7 / 3.2)

Lesson Plan Objective: SS-RFY SEQ 2

Source: New

Level of knowledge: comprehension



References:

1. OP-CN-SS-RFY page 8

**Bank Question: 795****Answer: D**

1 Pt(s)

Unit 1 is operating at 100% power with rod control in manual. Given the following events and conditions:

- Maintenance is conducting a valve stroke test on 1NI-147B (*NI PMP RECIRC TO FWST*) with the plant at full power.
- The SRO directs the balance of plant (BOP) operator to operate 1NI-147B in support of the maintenance procedure.

While the valve is stroking (both indicating lights on), an annunciator 1AD-7 F/3 (*LETDN HX OUTLET HI TEMP*) alarms.

Which of the following actions describes BOP's actions in response to this condition?

- A. The BOP should immediately ensure correct letdown and KC flow to the letdown heat exchanger.
- B. The BOP should ensure 1NI-147B is closed, and then ensure correct KC flow to the letdown heat exchanger.
- C. The BOP should immediately verify letdown heat exchanger outlet temperature and ensure correct letdown and KC flow.
- D. The BOP should ensure 1NI-147B is open and then verify letdown and KC flow to the letdown heat exchanger.

---

**Distracter Analysis:**

- A. **Incorrect:** BOP must first address NI inoperability due to 147 not open.  
**Plausible:** These are the ARP immediate actions.
- B. **Incorrect:** 1NI-147 needs to be open to restore operability.  
**Plausible:** If the operator incorrectly assesses operability.
- C. **Incorrect:** BOP must first address NI inoperability due to 1NI-147 not open.  
**Plausible:** a more correct description of BOP response than A.
- D. **Correct:**

Level: RO Only

KA: G2.2.23(2.6 / 3.8)

Lesson Plan Objective: ADM-TS SEQ 5

Source: New

Level of knowledge: comprehension

References:

1. OP-CN-ADM-TS page 22
2. OMP 1.7 page 9
3. ARP 1AD-7; F/3

**Bank Question: 796****Answer: C**

1 Pt(s)

Unit 1 was operating at 100% power. Given the following events and conditions:

- 1 AD-6 F/10, (*PRT HI TEMP*) in alarm
- 1 AD-6 F/11 (*PRT HI PRESS*) in alarm
- Lower containment temperature = 124 °F
- The NC system is at normal operating temperature
- Letdown is in service

Which one of the following statements correctly describes a condition that could cause these alarms?

- A. **The PRT has heated up due to ambient containment temperature.**
- B. **1 ND-14, (*ND SUCTION RELIEF VALVE*) has lifted.**
- C. **1 NV-15B, (*LETDN CONT ISOL*), has spuriously closed.**
- D. **The reactor vessel inner O-ring has leaked.**

---

**Distracter Analysis:**

- A. **Incorrect:** Containment temperature is less than PRT temperature.  
**Plausible:** Heat up from containment ambient conditions can cause this to occur if containment temperature is high enough.
- B. **Incorrect:** the ND system is too low in temperature and isolated from the NC system, to cause this to occur even if the ND suction relief were to lift.  
**Plausible:** the ND suction relief line goes to the PRT.
- C. **Correct:** if 1NV-15B lifts, the letdown relief valve 1NV-14 will lift and relieve to the PRT.
- D. **Incorrect:** the reactor vessel inner O-ring leaks to the NCDT.  
**Plausible:** if the candidate thinks that this leaks to the PRT

Level: RO Only

KA: SYS 007A1.03(2.6/2.7)

Lesson Plan Objective: PS-NC LPRO3

Source: New

Level of knowledge: comprehension

References:

1. OP-CN-PS-NC pages 12, 21-22, 25-26
2. OP-CN-PS-NV pages 11-13
3. OP-CN-PS-NC page 31
4. OP/1/B/6100/010G F/10, F/11

**Bank Question: 797****Answer: D**

1 Pt(s)

Unit 1 is in mode 6. Given the following events and conditions:

- Both trains of SSPS are in "TEST".
- Window A/2 annunciator is received on Panel 1RAD-1

*"1EMF-39 CONTAINMENT GAS HI RAD"*

- 1EMF-39 is the only monitor in alarm.

What one of the following statements correctly describes the automatic actions (if any) that should occur?

- A. 1EMF-39 has no auto actions with both trains of SSPS in test.
- B. 1EMF-39 will generate a containment evacuation alarm only.
- C. 1EMF-39 will generate a containment evacuation alarm and generate a containment ventilation isolation (S<sub>H</sub>) signal.
- D. 1EMF-39 will generate a containment evacuation alarm and isolate containment purge.

---

**Distracter Analysis:**

- A. **Incorrect:** EMF-39 actions are not blocked by SSPS  
**Plausible:** candidate may confuse EMF 39 with either 38 or 40, which are blocked by both trains of SSPS. Additionally, the containment evacuation alarm is only blocked by P-6
- B. **Incorrect:** will also isolate containment purge  
**Plausible:** This is true for EMF 38 and 40
- C. **Incorrect:** will not generate containment ventilation isolation with SSPS in test.  
**Plausible:** This is true if one train of SSPS is not in test.
- D. **Correct:** EMF-39 will directly close down VP if it goes into high alarm and both trains of SSPS are in test.

Level: RO&amp;SRO

KA: WE 016 EK2.1 (3.0/3.1)

Lesson Plan Objective: VP SEQ 7

Source: New

Level of knowledge: memory

References:

1. OP-CN-OP/1/B/6100/010X, annunciators A/1, A/2, A/3

**Bank Question: 798**

**Answer: A**

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1 Pt(s)      Which of the following is the power supply to the unit 2 “A” Hydrogen Recombiner?

- A.      **2EMXK**
- B.      **2SMXW**
- C.      **2MXW**
- D.      **2EMXL**

---

**Distracter Analysis:** This question tests the candidates’ knowledge of the thumb rules for labeling power supplies as well as the type of power provided to the recombiners. 2EMXK is the ONLY train A emergency power supply in the list.

- A.      **Correct:** A and B Hydrogen recombiners are from essential power supplies, (EMXK and EMXJ)
- B.      **Incorrect:** Hydrogen recombiners are powered from EMXK  
**Plausible:** 2SMXW is a shared power supply - candidate may believe it’s a shared power supply
- C.      **Incorrect:** Hydrogen recombiners are powered from EMXK  
**Plausible:** 2MXW is a blackout power supply - candidate may believe it’s a blackout power supply
- D.      **Incorrect:** Hydrogen recombiners are powered from EMXK  
**Plausible:** 2EMXK is a train B power supply - candidate may confuse the A vs. B trains.

Level: RO&SRO

KA: 028 K2.01 (2.5/2.8)

Lesson Plan Objective: none

Source: New

Level of knowledge: memory

References:

1. DBD Unit 2 System and Equipment Description - section 32.4  
page 55