



**UNITED STATES
NUCLEAR REGULATORY COMMISSION
REGION II
SAM NUNN ATLANTA FEDERAL CENTER
61 FORSYTH STREET SW SUITE 23T85
ATLANTA, GEORGIA 30303-8931**

November 28, 2001

Duke Energy Corporation
ATTN: Mr. H. B. Barron
Vice President
McGuire Nuclear Station
12700 Hagers Ferry Road
Huntersville, NC 28078-8985

SUBJECT: MCGUIRE NUCLEAR STATION - NRC INSPECTION REPORT NO. 50-369/
01-06 AND 50-370/01-06

Dear Mr. Barron:

The purpose of this letter is to notify you that the U.S. Nuclear Regulatory Commission (NRC) Region II staff will conduct a safety system design and performance capability inspection at your McGuire Nuclear Station during January and February, 2002. A team of five inspectors will perform the inspection. The inspection team will be led by Mr. F. Jape, a Senior Reactor Inspector from the NRC Region II Office. The inspection will be conducted in accordance with baseline Inspection Procedure 71111.21, Safety System Design and Performance Capability.

The inspection objective will be to evaluate the capability of the auxiliary feedwater system to mitigate selected design basis accidents and transients. The team will also review scenarios leading to partial or total loss of the auxiliary feedwater system and the interface with the standby shutdown facility.

During a telephone conversation on November 19, 2001, Mr. F. Jape of my staff, and Mr. L. Abernathy of your staff, confirmed arrangements for an information gathering site visit and the two-week onsite inspection. The schedule is as follows:

- Information gathering visit: Week of December 10-14, 2001
- Onsite inspection: January 14-18, 2002 and January 28-February 1, 2002

The purpose of the information gathering visit is to obtain information and documentation outlined in the Enclosure needed to support the inspection. Mr. R. Bernhard, a Region II Senior Reactor Analyst, may accompany Mr. F. Jape during the information gathering visit to review PRA data and identify risk significant components which will be examined during the inspection. Please contact Mr. F. Jape prior to preparing copies of the materials listed in the Enclosure. The inspectors will try to minimize your administrative burden by specifically identifying only those documents required for inspection preparation.

During the information gathering visit, the team leader will also discuss the following inspection support administrative details: office space; specific documents requested to be made available to the team in their office space; arrangements for site access; and the availability of

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knowledgeable plant engineering and licensing organization personnel to serve as points of contact during the inspection.

Thank you for your cooperation in this matter. If you have any questions regarding the information requested or the inspection, please contact me at (404) 562-4605, or Mr. F. Jape at (404) 562-4541.

In accordance with 10 CFR 2.790 of the NRC's "Rules of Practice," a copy of this letter and its enclosure will be available electronically for public inspection in the NRC Public Document Room or from the Publicly Available Records (PARS) component of NRC's document system (ADAMS). ADAMS is accessible from the NRC Web site at <http://www.nrc.gov/reading-rm/adams.html> (the Public Electronic Reading Room).

Sincerely,

/RA D. Charles Payne for:/

Charles R. Ogle, Chief
Engineering Branch 1
Division of Reactor Safety

Docket Nos. 50-369, 50-370
License Nos. NPF-9, NPF-17

Enclosure: Information Request for the Safety System Design and
Performance Capability Inspection

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DATE	11/21/2001	11/21/2001	11/26/2001	11/ /2001	11/ /2001	11/ /2001	11/ /2001
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**INFORMATION REQUEST FOR THE SAFETY SYSTEM DESIGN AND
PERFORMANCE CAPABILITY INSPECTION**

**AUXILIARY FEEDWATER (CA) SYSTEM, and INTERFACE WITH STANDBY SHUTDOWN
FACILITY (SSF)**

Note: Electronic media is preferred if readily available (i.e., on computer disc).

1. Site specific administrative procedures related to standard operation, abnormal operation, and emergency operation of the CA system and the SSF.
2. Design criteria (i.e., design basis documents) for the CA system and the SSF.
3. Technical Specification requirements and a list of associated surveillance test/calibration procedures for the CA system pumps, valves, instruments and the applicable AC and DC electrical power.
4. P&IDs that include suction sources for CA system, and a listing of all CA civil, instrumentation and mechanical drawings. Electrical drawings consisting of key one line diagrams of power supply for the motor driven auxiliary feedwater pumps, and one line diagrams of power from the switchyard to the feeder buses.
5. A list of engineering calculations (Electrical, Instrumentation and Controls and Mechanical/Nuclear) applicable to the CA system and the SSF.
6. Strategy for handling loss of CA and SSF events.
7. A list of plant modifications to the CA system and the SSF implemented since 1992.
8. List of current open temporary modifications and operator work arounds involving operation of the CA system and the SSF.
9. List of PIPs initiated since 1992 affecting the CA system and the SSF.
10. Summary of corrective maintenance activities, including the maintenance rule event log, performed on the CA system and the SSF in the past 12 months.
11. Self-assessment performed on CA and SSF systems in the last 24 months.
12. System description and operator training modules for the CA and SSF systems.
13. List of Operating Experience Program evaluations of industry, vendor, or NRC generic issues related to the CA and SSF systems for the past 3 years.
14. List of instrument setpoint changes affecting CA and SSF systems initiated since 1992. Include the number and title, date, brief description, and corresponding calculation number.

ENCLOSURE