

December 21, 2001

Mr. J. A. Price  
Vice President - Nuclear Technical Services - Millstone  
c/o Mr. David A. Smith  
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Waterford, CT 06385

SUBJECT: MILLSTONE NUCLEAR POWER STATION, UNIT NO. 3 - REVISION TO  
TECHNICAL SPECIFICATIONS TO DELETE POST-MAINTENANCE TESTING  
SURVEILLANCE REQUIREMENTS OF CONTAINMENT ISOLATION VALVES  
(TAC NO. MB2319)

Dear Mr. Price:

The Commission has issued the enclosed Amendment No. 200 to Facility Operating License No. NPF-49 for the Millstone Nuclear Power Station, Unit No. 3 (MP3), in response to your application dated June 28, 2001.

The amendment modifies the Technical Specifications (TSs) to delete the post-maintenance testing Surveillance Requirement 4.6.3.1 of containment isolation valves.

A copy of the related Safety Evaluation is also enclosed. Notice of Issuance will be included in the Commission's biweekly Federal Register notice. If you have any questions, please contact me at (301) 415-1484.

Sincerely,

**/RA/**

Victor Nerses, Sr. Project Manager, Section 2  
Project Directorate I  
Division of Licensing Project Management  
Office of Nuclear Reactor Regulation

Docket No. 50-423

Enclosures: 1. Amendment No. 200 to NPF-49  
2. Safety Evaluation

cc w/encls: See next page

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Unit 3

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Millstone Nuclear Power Station  
Unit 3

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c/o Mr. David A. Smith  
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DOMINION NUCLEAR CONNECTICUT, INC., ET AL.

DOCKET NO. 50-423

MILLSTONE NUCLEAR POWER STATION, UNIT NO. 3

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 200  
License No. NPF-49

1. The Nuclear Regulatory Commission (the Commission) has found that:
  - A. The application for amendment by the applicant dated June 28, 2001, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;
  - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
  - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
  - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
  - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 2.C.(2) of Facility Operating License No. NPF-49 is hereby amended to read as follows:

(2) Technical Specifications

The Technical Specifications contained in Appendix A, as revised through Amendment No. 200, and the Environmental Protection Plan contained in Appendix B, both of which are attached hereto, are hereby incorporated in the license. Dominion Nuclear Connecticut, Inc. shall operate the facility in accordance with the Technical Specifications and the Environmental Protection Plan.

3. This license amendment is effective as of the date of issuance, and shall be implemented within 30 days.

FOR THE NUCLEAR REGULATORY COMMISSION

**/RA/**

James W. Clifford, Chief, Section 2  
Project Directorate I  
Division of Licensing Project Management  
Office of Nuclear Reactor Regulation

Attachment: Changes to the Technical  
Specifications

Date of Issuance: December 21, 2001

ATTACHMENT TO LICENSE AMENDMENT NO. 200

FACILITY OPERATING LICENSE NO. NPF-49

DOCKET NO. 50-423

Replace the following page of the Appendix A Technical Specifications, with the attached revised page. The revised page is identified by amendment number and contains a marginal line indicating the area of change.

Remove

3/4 6-15

Insert

3/4 6-15

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION

RELATED TO AMENDMENT NO. 200

TO FACILITY OPERATING LICENSE NO. NPF-49

DOMINION NUCLEAR CONNECTICUT, INC.

MILLSTONE NUCLEAR POWER STATION, UNIT NO. 3

DOCKET NO. 50-423

1.0 INTRODUCTION

By letter dated June 28, 2001, Dominion Nuclear Connecticut, Inc. (the licensee), for Millstone Nuclear Power Station, Unit No. 3, (MP3) requested the Nuclear Regulatory Commission's (NRC's) approval to amend its operating license NPF-49 by revising the plant's Technical Specifications (TSs). The amendment deletes the post-maintenance testing surveillance requirements (SRs) of containment isolation valves.

2.0 BACKGROUND INFORMATION

The licensee is requesting approval to remove from the MP3 TS, SR 4.6.3.1 and add the word "DELETED" in its place. This SR requires that containment isolation valve operability be verified by performing a timed valve stroke test prior to returning a valve to service following any maintenance or repair. The licensee believes that a timed stroke test after valve maintenance is not always warranted because maintenance is sometimes performed on valves which would not adversely affect stroke time. The proposed change would provide the licensee flexibility in determining the appropriate post-maintenance test, based on the work performed.

3.0 EVALUATION

SR 4.6.3.1 pertains to post-maintenance testing of containment isolation valves and states, "Each isolation valve shall be demonstrated OPERABLE prior to returning the valve to service after maintenance, repair, or replacement work is performed on the valve or its associated actuator, control, or power circuit by performance of a cycling test and verification of isolation time." The licensee stated that deleting the requirement will provide them flexibility in determining the appropriate post-maintenance test based on the work performed, and that when appropriate, a timed valve stroke test will be performed to ensure component operability following maintenance activities. The licensee has further stated that post-maintenance testing is controlled by plant procedures that specify the appropriate testing after completion of valve work. Remaining MP3 TS SRs 4.6.3.2 and 4.6.3.3 require that each isolation valve be demonstrated OPERABLE at specified intervals and that the isolation time of each power-operated or automatic valve be determined to be within its limit at intervals specified by the Inservice Testing program as described in TS 4.0.5. The staff notes that no SR similar to



the current MP3 SR 4.6.3.1, requiring a timed valve stroke test after any form of maintenance to containment isolation valves, appears in the Westinghouse Standard Technical Specifications (STS). The Westinghouse STS SR that treats the verification of isolation time of containment isolation valves (CIVs) is STS SR 3.6.3.5. This SR states, "Verify the isolation time of each automatic power operated CIV is within limits." There is no reference to the conduct of maintenance. This is strictly a periodic surveillance. The stated frequency for this SR is written within brackets in the STS as, "[In accordance with the Inservice Testing Program or 92 days]." The brackets indicate that a licensee may choose to adopt either of the options when converting to STS. In this case, the licensee is not requesting a total conversion of all of its TSs to STS format. However, approval of this one specific change can be based on adherence to the STS. By deleting MP3 SR 4.6.3.1, yet retaining current SRs 4.6.3.2 and 4.6.3.3, MP3 is retaining requirements within its TSs for the verification of OPERABILITY of CIVs that are equivalent to the requirements presented in the STS.

#### 4.0 SUMMARY

Based on our review, we find the deletion of MP3 TS SR 4.6.3.1 to be acceptable because remaining SRs 4.6.3.2 and 4.6.3.3 will ensure that verification of CIV OPERABILITY, including isolation time, is performed in accordance with the licensee's Inservice Testing Program. Furthermore, the licensee, upon performing maintenance to CIVs will evaluate the need for a post-maintenance, timed valve stroke test and perform this test if appropriate. It is understood that this determination will be made in accordance with the licensee's in-house maintenance procedures.

#### 5.0 STATE CONSULTATION

In accordance with the Commission's regulations, the Connecticut State official was notified of the proposed issuance of the amendment. The State official had no comments.

#### 6.0 ENVIRONMENTAL CONSIDERATION

The amendment changes a requirement with respect to installation or use of a facility component located within the restricted area as defined in 10 CFR Part 20 and changes surveillance requirements. The NRC staff has determined that the amendment involves no significant increase in the amounts, and no significant change in the types, of any effluents that may be released offsite, and that there is no significant increase in individual or cumulative occupational radiation exposure. The Commission has previously issued a proposed finding that the amendment involves no significant hazards consideration, and there has been no public comment on such finding (66 FR 52799). Accordingly, the amendment meets the eligibility criteria for categorical exclusion set forth in 10 CFR 51.22(c)(9). Pursuant to 10 CFR 51.22(b) no environmental impact statement or environmental assessment need be prepared in connection with the issuance of the amendment.

## 7.0 CONCLUSION

The Commission has concluded, based on the considerations discussed above, that: (1) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, (2) such activities will be conducted in compliance with the Commission's regulations, and (3) the issuance of the amendment will not be inimical to the common defense and security or to the health and safety of the public.

Principal Contributor: J. Golla

Date: December 21, 2001