

November 23, 2001

LICENSEE : Duke Energy Corporation

FACILITIES: McGuire, Units 1 and 2, and Catawba, Units 1 and 2

SUBJECT: TELECOMMUNICATION WITH DUKE ENERGY CORPORATION TO DISCUSS
INFORMATION IN THEIR LICENSE RENEWAL APPLICATION ON THE
INSERVICE INSPECTION PLAN

On October 15, 2001, after the NRC (the staff) reviewed information provided in Appendix B of the license renewal application (LRA), a conference call was conducted between the staff and Duke Energy Corporation (the applicant) to clarify information presented in the application pertaining to the Inservice Inspection Plan. Participants in the conference call are provided in an attachment.

The questions asked by the staff, as well as the responses provided by the applicant, are as follows:

B.3.20 Inservice Inspection Plan

1. The LRA states that the inservice inspection plan includes the following inspections and activities: (1) ASME Section XI, Subsections IWB and IWC (secondary side of steam generators) inspections, (2) ASME Section XI, Subsection IWF inspections, (3) small bore piping, and (4) McGuire Unit 1 cold leg elbow. It is not clear if the inservice inspection plan also includes Class 2 components other than those associated with the secondary side of steam generators. Describe what inservice inspection and or augmented inspection programs are being credited to manage aging effects in the remaining Class 2 components and their integral attachments (scope).

The applicant indicated that the Inservice Inspection Plan was not credited for managing the aging of any Class 2 or Class 3 components. Other aging management programs (e.g. Chemistry Control Program, Inspection Program for Civil Engineering Structures and Components, and Fluid Leak Management Program) will be used to manage the aging of these components because they will be capable of detecting degradation (loss of material) such that corrective actions can be taken before failures occur. The applicant stated that the Inservice Inspection Plan would reveal only failures. As such, the credited aging management programs would allow for the detection of degradation and implementation of corrective actions before a failure is incurred. The staff is satisfied with this response and has no additional questions on this item.

2. The inservice inspection plan in the LRA does not address Class 3 pressure retaining components and associated integral attachments. Describe what inservice inspection and or augmented inspection programs are being credited to manage the aging effects of Class 3 components and their integral attachments.

The applicant provided the same rationale in response to this question that was provided for the first question. The staff is satisfied with this response and has no additional questions on this item.

3. In B.3.20.1 (parameters monitored or inspected) the LRA states that Class 1 component welds, integral attachments, piping welds, bolted closures and supports as well as the Class 2 pressure boundary portions of the steam generators (welds and welded attachments) are inspected for cracking and loss of material but fails to address the remaining Class 2 pressure boundary components and integral attachments. Describe what parameters or indicators will be monitored or inspected in the remaining Class 2 pressure retaining components and integral attachments in order to detect the presence of aging effects.

The applicant provided the same rationale in response to this question that was provided for the first question. The staff is satisfied with this response and has no additional questions on this item.

4. The LRA states that the risk-informed process used to select piping elements for inspection is consistent with all Class 1 piping (i.e., large bore, small bore and socket welds) with an internal diameter greater than 3/8-inch NPS. Describe how the results of the risk-informed evaluations will be integrated into the inservice inspection plan regarding parameters monitored or inspected, detection of aging effects, and monitoring and trending.

The applicant indicated that the risk-informed methodology was established in WCAP 14572 and described in a relief request submitted to the staff on June 26, 2001, for McGuire Units 1 and 2. The staff will review these documents and determine if additional information is needed to complete their review of this item.

A draft of this telecommunication summary was provided to the applicant to allow them the opportunity to comment prior to the summary being issued.

/RA/

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Office of Nuclear Reactor Regulation

Docket Nos. 50-369, 50-370, 50-413, and 50-414

Attachment: As stated

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