

November 16, 2001

Mr. David A. Christian
Senior Vice President and
Chief Nuclear Officer
Innsbrook Technical Center-2SW
5000 Dominion Blvd.
Glen Allen, Virginia 23060-6711

SUBJECT: NORTH ANNA POWER STATION, UNITS 1 AND 2: REQUEST FOR
ADDITIONAL INFORMATION (RAI) REGARDING LICENSE AMENDMENT
REQUEST FOR ELIMINATION OF SEISMIC EFFECTS FROM CONTROL ROD
DROP TIME (TAC NOS. MB2504 AND MB2505)

Dear Mr. Christian:

By letter dated June 22, 2000, as supplemented by letters dated January 19 and July 26, 2001, VEPCO proposed technical specification changes to eliminate the seismic effects from the control rod drop times at North Anna Power Station. The staff has reviewed your submittals and has determined that additional information is needed to complete our review.

Our questions are provided in the Enclosure. This inquiry was discussed with Mr. Tom Shaub of your licensing staff on November 1, 2001, who agreed to provide the staff with a response within 60 days of the date of this letter.

Sincerely,

/RA/

Stephen R. Monarque, Project Manager, Section 1
Project Directorate II
Division of Licensing Project Management
Office of Nuclear Reactor Regulation

Docket Nos. 50-338 and 50-339

Enclosure: Request for Additional Information

cc w/encl: See next page

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Virginia Electric and Power Company

North Anna Power Station
Units 1 and 2

cc:

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REQUEST FOR ADDITIONAL INFORMATION
NORTH ANNA POWER STATION, UNITS 1 AND 2
LICENSE AMENDMENT REQUEST ON ELIMINATION OF SEISMIC EFFECTS
FROM CONTROL ROD DROP TIME

1. Assuming (1) control rods are subject to the postulated seismic-related control rod drop time delay for applicable seismic events, and (2) required reactor trip signals are successful, would reactor coolant system (RCS) pressure open the pressurizer power-operated relief valve(s) and/or the safety relief valves? If so, would the valves be required to function in a steam or water environment? Provide a discussion on the associated risk of these considerations in terms of Regulatory Guide (RG) 1.174, "An Approach for Using Probabilistic Risk Assessment in Risk-Informed Decisions on Plant-Specific Changes to the Licensing Basis" guidance.
2. Please provide for review your seismic risk analysis of the failure of a turbine trip/reactor trip scenario mentioned in your July 26, 2001, submittal. Discuss the risk significance of this consideration using the guidance in RG 1.174.
3. The submittal dated June 22, 2000, indicated that if accident analyses are re-performed, the rod drop time of 2.7 seconds would need to be increased. Such a change would involve concurrent protection system changes (e.g., reductions in high pressurizer pressure and/or low RCS flow reactor trip setpoints). This submittal indicated that this would have the potential to reduce normal operating margin and increase the potential for reactor trip events and associated plant equipment transients. Please discuss the potential magnitude of the decreases in the setpoints.

Enclosure