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1.7 Deleted.

CONTAINMENT INTEGRITY

1.8 CONTAINMENT INTEGRITY shall exist when:

- a. All penetrations required to be closed during accident conditions are either:
 - 1. Capable of being closed by the Safety Features Actuation System, or
 - 2. Closed by manual valves, blind flanges, or deactivated automatic valves secured in their closed positions, except those approved to be open under administrative controls,
- b. The equipment hatch is closed,
- c. Each air lock is in compliance with the requirements of Specification 3.6.1.3,
- d. The containment leakage rates are within the limits specified in the Containment Leakage Rate Testing Program, and
- e. The sealing mechanism associated with each penetration (e.g., welds, bellows or O-rings) is OPERABLE.

CHANNEL CALIBRATION

1.9 A CHANNEL CALIBRATION shall be the adjustment, as necessary, of the channel output such that it responds with necessary range and accuracy to known values of the parameter which the channel monitors. The CHANNEL CALIBRATION shall encompass the entire channel including the sensor and alarm and/or trip functions, and shall include the CHANNEL FUNCTIONAL TEST. CHANNEL CALIBRATION may be performed by any series of sequential, overlapping or total channel steps such that the entire channel is calibrated.

CHANNEL CHECK

1.10 A CHANNEL CHECK shall be the qualitative assessment of channel behavior during operation by observation. This determination shall include, where possible, comparison of the channel indication and/or status with other indications and/or status derived from independent instrument channels measuring the same parameter.

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6.5.3 TECHNICAL REVIEW AND CONTROL

ACTIVITIES

6.5.3.1 Activities which affect nuclear safety shall be conducted as follows:

- a. Plant procedures required by Section 6.8.1 and changes thereto shall be prepared, reviewed and approved. Each such procedure or procedure change shall be reviewed by an individual/group other than the individual/group which prepared the procedure or procedure change, but who may be from the same organization as the individual/group which prepared the procedure or procedure change. Plant procedures, (including plant administrative procedures), Industrial Security Plan Implementing Procedures and Davis-Besse Emergency Plan Implementing Procedures will be approved by procedurally authorized individuals.
- b. Temporary approval of changes to plant procedures cited in Section 6.8.1 which clearly do not change the intent of the approved procedures, can be made by two members of the plant management staff, at least one of whom holds a Senior Reactor Operator's License. For changes to plant procedures, which may involve a change in intent of the approved

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procedures, the person authorized in Section 6.5.3.1a to approve the procedure shall approve the change.

- c. Proposed changes or modifications to plant structures, systems and components shall be reviewed as designated by procedurally authorized individuals. Each such modification shall be reviewed by an individual/group other than the individual/group which designed the modification, but who may be from the same organization as the individual/group which designed the modifications. Implementation of modifications to plant structures, systems and components shall be approved by procedurally authorized individuals.
- d. Proposed tests and experiments which affect plant nuclear safety and are not addressed in the Safety Analysis Report shall be reviewed by an individual/group other than the individual/group which prepared the proposed test or experiment and shall be approved by procedurally authorized individuals.
- e. Individuals responsible for reviews performed in accordance with Section 6.5.3.1 a, b, c and d above shall meet or exceed the appropriate qualification requirements of Section 4.2, 4.3.1, 4.4 or 4.6 of ANSI 18.1, 1971, and be previously designated by procedurally authorized individuals. Each such review shall include a determination of whether an additional, cross disciplinary, review is necessary. If deemed necessary, such review shall be performed by the review personnel of the appropriate discipline.
- f. Each review will include a determination of whether prior NRC approval is required pursuant to 10 CFR 50.59.

6.6 DELETED

APPLICABILITY

BASES

val was exceeded. Completion of the Surveillance Requirement within the allowable (equipment inoperability) outage time limits of the ACTION requirements restores compliance with the requirements of Specification 4.0.3. However, this does not negate the fact that the failure to have performed the surveillance within the allowed surveillance interval, defined by the provisions of Specification 4.0.2, was a violation of the OPERABILITY requirements of a Limiting Condition for Operation that may be subject to enforcement action. Further, the failure to perform a surveillance within the provisions of Specification 4.0.2 is a violation of a Technical Specification requirement and, therefore, the reportable event requirements of 10 CFR 50.73(a)(2)(i)(B) should be reviewed for applicability.

If the allowable (equipment inoperability) outage time limits of the ACTION requirements are less than 24 hours or a shutdown is required to comply with ACTION requirements, e.g., Specification 3.0.3, a 24-hour allowance is provided to permit a delay in implementing the ACTION requirements. This provides an adequate time limit to complete Surveillance Requirements that have not been performed. The purpose of this allowance is to permit the completion of a surveillance before a shutdown is required to comply with ACTION requirements or before other remedial measures would be required that may preclude completion of a surveillance. The basis for this allowance includes consideration for plant conditions, adequate planning, availability of personnel, the time required to perform the surveillance, and the safety significance of the delay in completing the required surveillance. If a surveillance is not completed within the 24-hour allowance, the time limits of the ACTION requirements are applicable at that time. When a surveillance is performed within the 24-hour allowance and the Surveillance Requirements are not met, the time limits of the ACTION requirements are applicable at the time that the surveillance is terminated.

Surveillance Requirements do not have to be performed on inoperable equipment because the ACTION requirements define the remedial measures that apply. However, the Surveillance Requirements have to be met to demonstrate that inoperable equipment has been restored to OPERABLE status.

4.0.4 This specification ensures that the surveillance activities associated with a Limiting Condition for Operation have been performed within the specified time interval prior to entry into an OPERATIONAL MODE or other applicable condition. The intent of this provision is to ensure that surveillance activities have been satisfactorily demonstrated on a current basis as required to meet the OPERABILITY requirements of the Limiting Condition for Operation.

Under the terms of this specification, for example, during initial plant startup or following extended plant outages, the applicable surveillance activities must be performed within the stated surveillance interval prior to placing or returning the system or equipment into OPERABLE status.