

October 11, 2001

Mr. Dale E. Young, Vice President
Crystal River Nuclear Plant (NA1B)
ATTN: Supervisor, Licensing & Regulatory Programs
15760 W. Power Line Street
Crystal River, Florida 34428-6708

SUBJECT: CRYSTAL RIVER UNIT 3, UPCOMING STEAM GENERATOR TUBE
INSERVICE INSPECTION (TAC NO. MB3015)

Dear Mr. Young:

Inservice inspections of once through steam generator (OTSG) tubes play a vital role in assuring that adequate structural integrity of the tubes is maintained. As required by the plant Technical Specifications, reporting requirements range from notifying the Nuclear Regulatory Commission (NRC) of the following prior to ascension into MODE 4:

1. Number of tubes plugged and repaired;
2. Crack-like indications and assessment of growth for indications in the first span;
3. Results of in situ pressure testing, if performed; and
4. Number of tubes and axially oriented tube end cracks (TEC) indications left in-service, the projected accident leakage, and an assessment of growth for the TEC indications.

Florida Power Corporation is to report the results of the OTSG tube inspections that fall into Category C-3 in accordance with Title 10, *Code of Federal Regulations* (10CFR), Section 50.72, and submit the complete results of the OTSG in-service inspection to the NRC within 90 days after breaker closure following restart. The report shall include:

1. Number and extent of tubes inspected.
2. Location and percent of wall-thickness penetration for each indication of an imperfection.
3. Location, bobbin coil amplitude, and axial and circumferential extent (if determined) for each first span Pit-like Intergranular Attack indication, and
4. Identification of tubes plugged or repaired and specification of the repair methodology implemented for each tube.

Dale E. Young

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A phone conference has been arranged with members of your staff to discuss the ongoing results of the SG tube inspections to be conducted during the upcoming Crystal River Unit 3 refueling outage. This phone call will occur after the majority of the tubes have been inspected, but before the SG inspection activities have been completed. Enclosed is a list of discussion points to facilitate this phone conference.

The staff plans to document a brief summary of the conference call as well as any material that you may have provided to the staff in support of the call.

Sincerely,

/RA/

John M. Goshen, Project Manager, Section 2
Project Directorate II
Division of Licensing Project Management
Office of Nuclear Reactor Regulation

Docket No. 50-302

Enclosure: List of Discussion Points

cc w/enclosure: See next page

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Florida Power Corporation

**CRYSTAL RIVER UNIT NO. 3
GENERATING PLANT**

cc:

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STEAM GENERATOR TUBE INSPECTION DISCUSSION POINTS

PREPARED BY THE OFFICE OF NUCLEAR REACTOR REGULATION

FLORIDA POWER CORPORATION

DOCKET NO. 50-302

CRYSTAL RIVER UNIT 3 NUCLEAR GENERATING PLANT

The following discussion points have been prepared to facilitate the phone conference arranged with the Crystal River Unit 3 (CR-3) staff to discuss the results of the Steam Generator (SG) tube inspections to be conducted during the upcoming CR-3 refueling outage. This phone call is scheduled to occur towards the end of the planned SG tube inspection interval, but before the unit exits its refueling outage.

It is the Nuclear Regulatory Commission (NRC) staff's expectation that any significant results or relevant trends discussed during the phone conference, as well as any materials provided by your staff to assist us during the phone conference in the understanding of the SG tube results, will be included in one of the special reports required by the plant Technical Specifications.

1. Discuss whether any primary to secondary leakage existed in this unit prior to shutdown.
2. Discuss the results of secondary side hydrostatic tests.
3. For each steam generator, provide a general description of areas examined, including the expansion criteria utilized and type of probe used in each area.
4. For analyzed eddy current results, describe bobbin indications (those not examined with rotating pancake coil (RPC)) and RPC/Plus Point/Cecco indications. Include the following information in the discussion: location, number, degradation mode, disposition, and voltages/depths/lengths of significant indications.
5. Describe repair/plugging plans for the SG tubes that meet the repair/plugging criteria.
6. Discuss the previous history of SG tube inspection results, including any "look backs" performed.
7. Discuss, in general, the new inspection findings.

Enclosure

8. Describe in situ pressure test plans and results, if applicable and available, including tube selection criteria.
9. Describe tube pull plans and preliminary results, if applicable and available; include tube selection criteria.
10. Discuss the assessment of tube integrity for the previous operating cycle.
11. Discuss the assessment of tube integrity for next operating cycle .
12. Provide the schedule for steam generator-related activities during the remainder of the current outage.
13. Discuss what steps have been taken, or will be taken, in response to the lessons learned from the Indian Point Unit 2 tube failure. In addition, please be prepared to discuss the following:
 - a) Discuss the actions that are taken in response to identifying a new degradation mechanism, and
 - b) Discuss the actions taken to ensure that data noise levels are acceptable, and
 - c) Address data quality issues and the need for criteria to address data quality.