

October 22, 2001

Mr. James F. Klapproth, Manager
Engineering & Technology
GE Nuclear Energy
175 Curtner Ave
San Jose, CA 95125

SUBJECT: SAFETY EVALUATION REPORT ON GENERAL ELECTRIC NUCLEAR
ENERGY TOPICAL REPORT NEDE-32906P, REVISION 0, "TRACG
APPLICATION FOR ANTICIPATED OPERATIONAL OCCURRENCES (AOO)
TRANSIENT ANALYSES," (TAC NO. MA7779)

Dear Mr. Klapproth:

By letter dated January 25, 2000, General Electric Nuclear Energy (GENE) submitted for review and approval topical report (TR), NEDE-32906P, Revision 0, "TRACG Application for Anticipated Operational Occurrences (AOO) Transient Analyses," for the use of the TRACG code for application to boiling water reactor (BWR) AOO's. The primary document describing the TRACG code is NEDE-32176P, Rev. 2, "TRACG Model Description," December 1999, provided by letter dated December 15, 1999. The TR describes the TRACG analysis code and the assessment of the code's capabilities based on application of the Code Scaling, Applicability, and Uncertainty evaluation methodology (CSAU).

The staff finds that the subject TR is acceptable for referencing in licensing applications to the extent specified and under the limitations delineated in the TR and in the associated NRC safety evaluation (SE). The SE, which is enclosed, defines the basis for acceptance of the TR.

The NRC requests that GENE publish an accepted version of the revised NEDE-32906P within 3 months of receipt of this letter. The accepted version shall incorporate this letter and the enclosed SE between the title page and the abstract, and add an "-A" (designating accepted) following the report identification number (i.e., NEDE-32906P-A).

If the NRC's criteria or regulations change so that its conclusion in this letter, that the TR is acceptable, is invalidated, GENE and/or the applicant referencing the TR will be expected to revise and resubmit its respective documentation, or submit justification for the continued applicability of the TR without revision of the respective documentation.

Pursuant to 10 CFR 2.790, we have determined that the enclosed SE does not contain proprietary information. However, we will delay placing this document in the public document room for a period of ten (10) working days from the date of the letter to provide you with the

Mr. James F. Klapproth

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opportunity to comment on the proprietary aspects only. If you believe that any information in the enclosure is proprietary, please identify such information line by line and define the basis pursuant to the criteria of 10 CFR 2.790.

The subject TR and supporting documentation has been reviewed by the Advisory Committee on Reactor Safeguards which has agreed with the staff recommendation for approval by their letter of September 17, 2001.

If you have any further questions regarding this review, please contact Robert M. Pulsifer (301-415-3016).

Sincerely,

/RA/

Stuart A. Richards, Director
Project Directorate I
Division of Licensing Project Management
Office of Nuclear Reactor Regulation

Project No. 710

Enclosure: Safety Evaluation

cc: See next page

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Stuart A. Richards, Director
Project Directorate IV
Division of Licensing Project Management
Office of Nuclear Reactor Regulation

Project No. 710

Enclosure: Safety Evaluation

cc: See next page

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GE Nuclear Energy

Project No. 710

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