

September 20, 2001

Mr. Oliver D. Kingsley, President
Exelon Nuclear
Exelon Generation Company, LLC
1400 Opus Place, Suite 500
Downers Grove, IL 60515

SUBJECT: ISSUANCE OF AMENDMENTS - BYRON STATION, UNITS 1 AND 2 AND
BRAIDWOOD STATION, UNITS 1 AND 2 - REQUEST FOR TECHNICAL
SPECIFICATIONS CHANGE - REVISION TO THE REACTOR COOLANT
PUMP FLYWHEEL INSPECTION PROGRAM (TAC NOS. MB1825, MB1826,
MB1827, AND MB1828)

Dear Mr. Kingsley:

The U.S. Nuclear Regulatory Commission (Commission) has issued the enclosed Amendment No. 123 to Facility Operating License No. NPF-37 and Amendment No. 123 to Facility Operating License No. NPF-66 for the Byron Station, Unit Nos. 1 and 2, respectively, and Amendment No. 118 to Facility Operating License No. NPF-72 and Amendment No. 118 to Facility Operating License No. NPF-77 for the Braidwood Station, Unit Nos. 1 and 2, respectively. The amendments are in response to your application dated April 27, 2001.

The amendments revise Technical Specifications Section 5.5.7 to provide an exception to the recommendations of Regulatory Position C.4.b which would allow either a qualified in-place ultrasonic volumetric examination over the volume from the inner bore of the flywheel to the circle of one-half the outer radius or a surface examination (magnetic particle testing and/or liquid penetrant testing) of exposed surfaces of the removed flywheel to be conducted at approximately 10-year intervals. The proposed change is in accordance with the NRC approved Improved Standard TS Generic Change Traveler TSTF-237, Revision 1.

A copy of the Safety Evaluation is also enclosed. The Notice of Issuance will be included in the Commission's biweekly Federal Register notice.

Sincerely,

/RA/

Mahesh Chawla, Project Manager, Section 2
Project Directorate III
Division of Licensing Project Management
Office of Nuclear Reactor Regulation

Docket Nos. STN 50-454, STN 50-455,
STN 50-456 and STN 50-457

Enclosures: 1. Amendment No. 123 to NPF-37
2. Amendment No. 123 to NPF-66
3. Amendment No. 118 to NPF-72
4. Amendment No. 118 to NPF-77
5. Safety Evaluation

cc w/encls: See next page

O. Kingsley
Exelon Generation Company

Byron/Braidwood Stations

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Byron/Braidwood Stations

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Mr. Oliver D. Kingsley, President September 20, 2001
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AND MB1828)

Dear Mr. Kingsley:

The U.S. Nuclear Regulatory Commission (Commission) has issued the enclosed Amendment No. 123 to Facility Operating License No. NPF-37 and Amendment No. 123 to Facility Operating License No. NPF-66 for the Byron Station, Unit Nos. 1 and 2, respectively, and Amendment No. 118 to Facility Operating License No. NPF-72 and Amendment No. 118 to Facility Operating License No. NPF-77 for the Braidwood Station, Unit Nos. 1 and 2, respectively. The amendments are in response to your application dated April 27, 2001.

The amendments revise Technical Specifications Section 5.5.7 to provide an exception to the recommendations of Regulatory Position C.4.b which would allow either a qualified in-place ultrasonic volumetric examination over the volume from the inner bore of the flywheel to the circle of one-half the outer radius or a surface examination (magnetic particle testing and/or liquid penetrant testing) of exposed surfaces of the removed flywheel to be conducted at approximately 10-year intervals. The proposed change is in accordance with the NRC approved Improved Standard TS Generic Change Traveler TSTF-237, Revision 1.

A copy of the Safety Evaluation is also enclosed. The Notice of Issuance will be included in the Commission's biweekly Federal Register notice.

Sincerely,

/RA/

Mahesh Chawla, Project Manager, Section 2
Project Directorate III
Division of Licensing Project Management
Office of Nuclear Reactor Regulation

Docket Nos. STN 50-454, STN 50-455,
STN 50-456 and STN 50-457

Enclosures: 1. Amendment No. 123 to NPF-37
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3. Amendment No. 118 to NPF-72
4. Amendment No. 118 to NPF-77
5. Safety Evaluation

cc w/encls: See next page

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***See previous concurrence**

OFFICE	PM:LPD3	PM:LPD3	LA:LPD3	SC: EMCB	SC:LPD3	OGC	SC:LPD3
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DATE	09/06/01	09/10/01	09/20/01	09/06/01	09/07/01	09/14/01	09/20/01

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EXELON GENERATION COMPANY, LLC

DOCKET NO. STN 50-454

BYRON STATION, UNIT NO. 1

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 123
License No. NPF-37

1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment by Exelon Generation Company, LLC (the licensee) dated April 27, 2001, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act) and the Commission's rules and regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.
2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 2.C.(2) of Facility Operating License No. NPF-37 is hereby amended to read as follows:

(2) Technical Specifications

The Technical Specifications contained in Appendix A as revised through Amendment No. 123 and the Environmental Protection Plan contained in Appendix B, both of which are attached hereto, are hereby incorporated into this license. The licensee shall operate the facility in accordance with the Technical Specifications and the Environmental Protection Plan.

3. This license amendment is effective as of the date of its issuance and shall be implemented within 30 days.

FOR THE NUCLEAR REGULATORY COMMISSION

/RA/

Anthony J. Mendiola, Chief, Section 2
Project Directorate III
Division of Licensing Project Management
Office of Nuclear Reactor Regulation

Attachment:
Changes to the Technical
Specifications

Date of Issuance: September 20, 2001

EXELON GENERATION COMPANY, LLC

DOCKET NO. STN 50-455

BYRON STATION, UNIT NO. 2

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 123
License No. NPF-66

1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment by Exelon Generation Company, LLC (the licensee) dated April 27, 2001, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act) and the Commission's rules and regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.
2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 2.C.(2) of Facility Operating License No. NPF-66 is hereby amended to read as follows:

(2) Technical Specifications

The Technical Specifications contained in Appendix A (NUREG-1113), as revised through Amendment No. 123 and the Environmental Protection Plan contained in Appendix B, both of which were attached to License No. NPF-37, dated February 14, 1985, are hereby incorporated into this license. The licensee shall operate the facility in accordance with the Technical Specifications and the Environmental Protection Plan.

3. This license amendment is effective as of the date of its issuance and shall be implemented within 30 days.

FOR THE NUCLEAR REGULATORY COMMISSION

/RA/

Anthony J. Mendiola, Chief, Section 2
Project Directorate III
Division of Licensing Project Management
Office of Nuclear Reactor Regulation

Attachment:
Changes to the Technical
Specifications

Date of Issuance: September 20, 2001

ATTACHMENT TO LICENSE AMENDMENT NOS. 123 AND 123

FACILITY OPERATING LICENSE NOS. NPF-37 AND NPF-66

DOCKET NOS. STN 50-454 AND STN 50-455

Revise the Appendix A Technical Specifications by removing the pages identified below and inserting the attached pages. The revised pages are identified by the captioned amendment number and contain marginal lines indicating the area of change.

Remove Pages

5.5-5

Insert Pages

5.5-5

EXELON GENERATION COMPANY, LLC

DOCKET NO. STN 50-456

BRAIDWOOD STATION, UNIT NO. 1

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 118
License No. NPF-72

1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment by Exelon Generation Company, LLC (the licensee) dated April 27, 2001, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act) and the Commission's rules and regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.
2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 2.C.(2) of Facility Operating License No. NPF-72 is hereby amended to read as follows:

(2) Technical Specifications

The Technical Specifications contained in Appendix A as revised through Amendment No. 118 and the Environmental Protection Plan contained in Appendix B, both of which are attached hereto, are hereby incorporated into this license. The licensee shall operate the facility in accordance with the Technical Specifications and the Environmental Protection Plan.

3. This license amendment is effective as of the date of its issuance and shall be implemented within 30 days.

FOR THE NUCLEAR REGULATORY COMMISSION

/RA/

Anthony J. Mendiola, Chief, Section 2
Project Directorate III
Division of Licensing Project Management
Office of Nuclear Reactor Regulation

Attachment:
Changes to the Technical
Specifications

Date of Issuance: September 20, 2001

EXELON GENERATION COMPANY, LLC

DOCKET NO. STN 50-457

BRAIDWOOD STATION, UNIT NO. 2

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 118
License No. NPF-77

1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment by Exelon Generation Company, LLC (the licensee) dated April 27, 2001, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act) and the Commission's rules and regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.
2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 2.C.(2) of Facility Operating License No. NPF-77 is hereby amended to read as follows:

(2) Technical Specifications

The Technical Specifications contained in Appendix A as revised through Amendment No. 118 and the Environmental Protection Plan contained in Appendix B, both of which were attached to License No. NPF-72, dated July 2, 1987, are hereby incorporated into this license. The licensee shall operate the facility in accordance with the Technical Specifications and the Environmental Protection Plan.

3. This license amendment is effective as of the date of its issuance and shall be implemented within 30 days.

FOR THE NUCLEAR REGULATORY COMMISSION

/RA/

Anthony J. Mendiola, Chief, Section 2
Project Directorate III
Division of Licensing Project Management
Office of Nuclear Reactor Regulation

Attachment:
Changes to the Technical
Specifications

Date of Issuance: September 20, 2001

ATTACHMENT TO LICENSE AMENDMENT NOS. 118 AND 118

FACILITY OPERATING LICENSE NOS. NPF-72 AND NPF-77

DOCKET NOS. STN 50-456 AND STN 50-457

Replace the following pages of the Appendix "A" Technical Specifications with the attached pages. The revised pages are identified by amendment number and contain vertical lines indicating the area of change.

Remove Pages

5.5-5

Insert Pages

5.5-5

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION
RELATED TO AMENDMENT NO. 123 TO FACILITY OPERATING LICENSE NO. NPF-37,
AMENDMENT NO. 123 TO FACILITY OPERATING LICENSE NO. NPF-66,
AMENDMENT NO. 118 TO FACILITY OPERATING LICENSE NO. NPF-72,
AND AMENDMENT NO. 118 TO FACILITY OPERATING LICENSE NO. NPF-77
EXELON GENERATION COMPANY, LLC
BYRON STATION, UNIT NOS. 1 AND 2
BRAIDWOOD STATION, UNIT NOS. 1 AND 2
DOCKET NOS. STN 50-454, STN 50-455, STN 50-456 AND STN 50-457

1.0 INTRODUCTION

By letter dated April 27, 2001, Exelon Generation Company, LLC, (the licensee), submitted a request for amendments to their Technical Specifications (TS) for the Reactor Coolant Pump Flywheel Inspection Program in Byron and Braidwood Stations, Units 1 and 2. The proposed amendments consist of revising TS 5.5.7 to provide an exception to the recommendations of Regulatory Position C.4.b of NRC Regulatory Guide (RG) 1.14, Revision 1, "Reactor Coolant Pump Flywheel Integrity," dated August 1975. The staff has reviewed the proposed changes to the TS and evaluated the licensee's justifications for these changes.

2.0 BACKGROUND

An integral part of the reactor coolant system (RCS) in a pressurized water reactor (PWR) is the reactor coolant pump (RCP). The RCP ensures an adequate cooling flow rate by circulating large volumes of the primary coolant water at high temperature and pressure through the RCS. Following an assumed loss of power to the RCP motor, the flywheel, in conjunction with the impeller and motor assembly, provide sufficient rotational inertia to assure adequate core cooling flow during RCP coastdown.

During normal power operation, the RCP flywheel possesses sufficient kinetic energy to produce high-energy missiles in the event of failure. Conditions which may result in overspeed of the RCP increase both the potential for failure and the kinetic energy of the flywheel. This led to the issuance of RG 1.14, which was published in 1971 and revised in 1975. RCP

flywheel inspections were implemented as a result of RG 1.14, which describes a range of actions to ensure flywheel integrity. One of the recommendations of RG 1.14 is regular inservice volumetric inspection of flywheels.

TS 5.5.7 states,

“This program shall provide for the inspection of each reactor coolant pump flywheel in general conformance with the recommendations of Regulatory Position c.4.b of Regulatory Guide 1.14, Revision 1, August 1975.”

Regulatory Position C.4.b of Regulatory Guide 1.14, Revision 1, recommends inservice inspections to be performed for each RCP flywheel as follows.

1. An in-place ultrasonic test (UT) of the areas of higher stress concentration at the bore and keyway at approximately 3-year intervals, during the refueling or maintenance shutdown coinciding with the inservice inspection schedule as required by American Society of Mechanical Engineers (ASME) Boiler and Pressure Vessel Code (B&PV) Section XI, "Rules for Inservice Inspection of Nuclear Power Plant Components."
2. A surface examination of all exposed surfaces and complete UT at approximately 10-year intervals, during the plant shutdown coinciding with the inservice inspection schedule as required by Section XI of the ASME B&PV Code.

3.0 EVALUATION

TS 5.5.7, "Reactor Coolant Pump Flywheel Inspection Program," requires that inspection of each RCP flywheel be in general conformance with the recommendations of Regulatory Position C.4.b of NRC RG 1.14, Revision 1, "Reactor Coolant Pump Flywheel Integrity," dated August 1975. The proposed change revises TS 5.5.7 to provide an exception to the recommendations of Regulatory Position C.4.b which would allow either a qualified in-place volumetric examination (i.e., ultrasonic examination) over the volume from the inner bore of the flywheel to the circle of one-half the outer radius, or a surface examination (i.e., magnetic particle testing and/or liquid penetrant testing) of exposed surfaces of the removed flywheel to be conducted at approximately 10-year intervals.

The proposed change is in accordance with the evaluation performed by the NRC staff of the Westinghouse Electric Corporation Topical Report, WCAP-14535A, "Topical Report on Reactor Coolant Pump Flywheel Inspection Elimination." The staff issued its safety evaluation (SE) of WCAP-14535A with the following provisions:

- Inspections need only be done on a 10-year interval instead of the current 40-month interval.
- Acceptable inspection methods are either ultrasonic examination or surface examination.

- Ultrasonic examination coverage is required only on the inner half of the flywheel radius.
- Surface examination coverage is the exposed surfaces of the flywheel when the pump is disassembled for maintenance.

Licensees can reference the SE for WCAP-14535A in license applications and detailed technical reviews of the submittals will not be required unless new technical information is presented. The SE for WCAP-14535A was incorporated into the staff approved Improved Standard TS Generic Change Traveler TSTF-237, Revision 1, Westinghouse Electric Corporation Topical Report WCAP-14535A, "Topical Report on Reactor Coolant Pump Flywheel Inspection Elimination."

The SE also stated that licensees who planned to submit a plant-specific application of the topical report for flywheels made of SA 533 Grade B material needed to confirm that their flywheels are made of SA 533 Grade B material. Further, licensees having Group-15 flywheels needed to demonstrate that material properties of their A516 material is equivalent to SA 533 Grade B material, and that its reference temperature, RT_{NDT} , is not less than 30 °F.

The licensee has confirmed that the flywheels for the Byron and Braidwood RCPs are of SA 533 Grade B material, and that they do not belong to either Group-10 or Group-15 flywheels. Therefore, the staff concludes that the flywheel material and grouping is consistent with the WCAP SE.

4.0 STATE CONSULTATION

In accordance with the Commission's regulations, the Illinois State official was notified of the proposed issuance of the amendments. The State official had no comments.

5.0 ENVIRONMENTAL CONSIDERATION

The amendments change a requirement with respect to the installation or use of a facility component located within the restricted area as defined in 10 CFR Part 20. The NRC staff has determined that the amendments involve no significant increase in the amounts, and no significant change in the types, of any effluents that may be released offsite, and that there is no significant increase in individual or cumulative occupational radiation exposure. The Commission has previously issued a proposed finding that the amendments involve no significant hazards consideration, and there has been no public comment on such finding (66 FR 31706). Accordingly, the amendments meet the eligibility criteria for categorical exclusion set forth in 10 CFR 51.22(c)(9). Pursuant to 10 CFR 51.22(b), no environmental impact statement or environmental assessment need be prepared in connection with the issuance of the amendments.

6.0 CONCLUSION

Based on the safety evaluation of WCAP-14535 by the NRC staff, and the evaluation above, the proposed change to the Technical Specifications Section 5.5.7 is acceptable for the Byron and Braidwood Stations, Units 1 and 2.

The Commission has concluded, based on the considerations discussed above, that: (1) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, (2) such activities will be conducted in compliance with the Commission's regulations, and (3) the issuance of the amendments will not be inimical to the common defense and security or to the health and safety of the public.

Principal Contributor: P. Patnaik

Date: September 20, 2001

DOCUMENT NAME: C:\Program Files\Adobe\Acrobat 4.0\PDF
Output\RCPFW.amd.wpd

ORIGINATOR NAME:MCHAWLA

SECRETARY NAME: Y. Edmonds/G. Hawkins

SUBJECT: Request for Technical Specifications Change- Revision to the
Reactor Coolant Pump Flywheel Inspection Program

NAME	DATE
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CONSIDERED OTHERS IMPACTED BY STAFF ACTION (2.206 PETITIONS, OPEN ALLEGATIONS, CONGRESSIONAL
OR PUBLIC INQUIRIES, SIGNIFICANT ENFORCEMENT ACTIONS, INSPECTION ACTIVITIES, COMMISSION
POLICIES, STAFF POSITIONS, OWNERS GROUP ACTIVITIES): YES ___ NO ___ INITIAL: PM ___ SC ___
COMMUNICATED W/IDENTIFIED STAKEHOLDER(S): YES ___ NO ___ INITIAL: PM ___ SC ___

2. C. Rosenberg	09/ /01
3. T.Chan	09/ /01
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SECRETARY FOR DISPATCHING	09/ /01

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