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JOSEPH A. WIDAY  
VICE PRESIDENT & PLANT MANAGER  
GINNA STATION

June 14, 2001

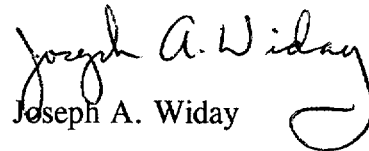
U.S. Nuclear Regulatory Commission  
Document Control Desk  
Attn: Guy S. Vissing  
Project Directorate I  
Washington, D.C. 20555

Subject: Emergency Operating Procedures  
R.E. Ginna Nuclear Power Plant  
Docket No. 50-244

Dear Mr. Vissing:

As requested, enclosed are Ginna Station Emergency Operating Procedures.

Very truly yours,

  
Joseph A. Widay

JAW/jdw

xc: U.S. Nuclear Regulatory Commission  
Region I  
475 Allendale Road  
King of Prussia, PA 19406-1415

Ginna USNRC Senior Resident Inspector

Enclosure(s):

AP Index  
FIG Index  
AP-RCS.3, Rev 8  
FIG-14.0, Rev 1

A645

REPORT NO. 01  
REPORT: NPSP0200  
DOC TYPE: PRAP

GINNA NUCLEAR POWER PLANT  
PROCEDURES INDEX  
ABNORMAL PROCEDURE

06/14/01 PAGE: 1

PARAMETERS: DOC TYPES - PROPS PRFIG PRAP

STATUS: EF QU 5 YEARS ONLY:

PROCEDURE NUMBER	PROCEDURE TITLE	REV	EFFECT DATE	LAST REVIEW	NEXT REVIEW	ST
AP-CCW.1	LEAKAGE INTO THE COMPONENT COOLING LOOP	014	01/09/01	05/01/98	05/01/03	EF
AP-CCW.2	LOSS OF CCW DURING POWER OPERATION	014	05/18/00	08/17/99	08/17/04	EF
AP-CCW.3	LOSS OF CCW - PLANT SHUTDOWN	012	05/18/00	08/17/99	08/17/04	EF
AP-CR.1	CONTROL ROOM INACCESSIBILITY	017	05/11/01	01/11/00	01/11/05	EF
AP-CVCS.1	CVCS LEAK	012	05/01/98	05/01/98	05/01/03	EF
AP-CVCS.3	LOSS OF ALL CHARGING FLOW	002	02/11/00	02/26/99	02/26/04	EF
AP-CW.1	LOSS OF A CIRC WATER PUMP	010	07/16/98	05/01/98	05/01/03	EF
AP-ELEC.1	LOSS OF 12A AND/OR 12B BUSES	020	09/08/00	05/01/98	05/01/03	EF
AP-ELEC.2	SAFEGUARD BUSES LOW VOLTAGE OR SYSTEM LOW FREQUENCY	009	03/22/99	03/22/99	03/22/04	EF
AP-ELEC.3	LOSS OF 12A AND/OR 12B TRANSFORMER (BELOW 350 F)	008	09/08/00	05/01/98	05/01/03	EF
AP-ELEC.14/16	LOSS OF SAFEGUARDS BUS 14/16	003	03/15/01	06/09/97	06/09/02	EF
AP-ELEC.17/18	LOSS OF SAFEGUARDS BUS 17/18	002	10/18/99	06/09/97	06/09/02	EF
AP-FW.1	PARTIAL OR COMPLETE LOSS OF MAIN FEEDWATER	012	02/11/00	02/27/98	02/27/03	EF
AP-IA.1	LOSS OF INSTRUMENT AIR	017	12/02/99	05/01/98	05/01/03	EF
AP-PRZR.1	ABNORMAL PRESSURIZER PRESSURE	012	03/26/01	12/02/99	12/02/04	EF
AP-RCC.1	CONTINUOUS CONTROL ROD WITHDRAWAL/INSERTION	007	05/22/01	05/14/98	05/14/03	EF
AP-RCC.2	RCC/RPI MALFUNCTION	008	11/16/98	02/06/97	02/06/02	EF
AP-RCC.3	DROPPED ROD RECOVERY	004	11/16/98	02/27/98	02/27/03	EF
AP-RCP.1	RCP SEAL MALFUNCTION	013	06/09/00	05/01/98	05/01/03	EF
AP-RCS.1	REACTOR COOLANT LEAK	015	09/08/00	05/01/98	05/01/03	EF
AP-RCS.2	LOSS OF REACTOR COOLANT FLOW	010	12/14/98	05/01/98	05/01/03	EF
AP-RCS.3	HIGH REACTOR COOLANT ACTIVITY	008	06/14/01	08/05/97	08/05/02	EF
AP-RCS.4	SHUTDOWN LOCA	011	12/02/99	05/01/98	05/01/03	EF
AP-RHR.1	LOSS OF RHR	015	02/08/01	05/01/98	05/01/03	EF

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GINNA NUCLEAR POWER PLANT  
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ABNORMAL PROCEDURE

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PARAMETERS: DOC TYPES - PROPS PRFIG PRAP STATUS: EF QU 5 YEARS ONLY:

PROCEDURE NUMBER	PROCEDURE TITLE	REV	EFFECT DATE	LAST REVIEW	NEXT REVIEW	ST
AP-RHR.2	LOSS OF RHR WHILE OPERATING AT RCS REDUCED INVENTORY CONDITIONS	009	10/13/00	03/31/00	03/31/05	EF
AP-SG.1	STEAM GENERATOR TUBE LEAK	000	09/08/00	09/08/00	09/08/05	EF
AP-SW.1	SERVICE WATER LEAK	015	10/18/99	06/03/98	06/03/03	EF
AP-TURB.1	TURBINE TRIP WITHOUT RX TRIP REQUIRED	010	02/12/99	10/10/97	10/10/02	EF
AP-TURB.2	TURBINE LOAD REJECTION	017	02/11/00	05/13/98	05/13/03	EF
AP-TURB.3	TURBINE VIBRATION	010	02/11/00	02/10/98	02/10/03	EF
AP-TURB.4	LOSS OF CONDENSER VACUUM	014	05/01/98	05/01/98	05/01/03	EF
AP-TURB.5	RAPID LOAD REDUCTION	005	06/09/00	06/09/00	06/09/05	EF
TOTAL FOR PRAP	32					

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REPORT: NPSP0200  
DOC TYPE: PRFIG

GINNA NUCLEAR POWER PLANT  
PROCEDURES INDEX  
EOP FIGURE PROCEDURES

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PARAMETERS: DOC TYPES - PROPS PRFIG PRAP

STATUS: EF QU 5 YEARS ONLY:

PROCEDURE NUMBER	PROCEDURE TITLE	REV	EFFECT DATE	LAST REVIEW	NEXT REVIEW	ST
FIG-1.0	FIGURE MIN SUBCOOLING	000	05/01/98	05/01/98	05/01/03	EF
FIG-2.0	FIGURE SDM	002	10/13/00	05/01/98	05/01/03	EF
FIG-3.0	FIGURE NAT CIRC C/D WITH SHROUD FANS	000	05/01/98	05/01/98	05/01/03	EF
FIG-3.1	FIGURE NAT CIRC C/D WITHOUT SHROUD FANS	001	02/08/01	05/01/98	05/01/03	EF
FIG-3.2	FIGURE NC C/D WITH VOID IN UPPER HEAD	000	05/01/98	05/01/98	05/01/03	EF
FIG-4.0	FIGURE RCP SEAL LEAKOFF	002	02/28/01	05/01/98	05/01/03	EF
FIG-5.0	FIGURE RHR INJECTION	000	05/01/98	05/01/98	05/01/03	EF
FIG-6.0	FIGURE MIN RCS INJECTION	000	05/01/98	05/01/98	05/01/03	EF
FIG-7.0	FIGURE INTACT S/G PRESSURE	001	05/18/98	05/01/98	05/01/03	EF
FIG-8.0	FIGURE TSAT	000	05/01/98	05/01/98	05/01/03	EF
FIG-9.0	FIGURE TECH SPEC C/D	001	02/15/01	05/01/98	05/01/03	EF
FIG-9.1	FIGURE C/D LIMITS	000	05/01/98	05/01/98	05/01/03	EF
FIG-10.0	FIGURE LIMIT A	000	05/01/98	05/01/98	05/01/03	EF
FIG-11.0	FIGURE SOAK LIMITS	000	05/01/98	05/01/98	05/01/03	EF
FIG-12.0	FIGURE CNMT HYDROGEN	000	05/01/98	05/01/98	05/01/03	EF
FIG-13.0	FIGURE BACK PRESSURE	000	05/01/98	05/01/98	05/01/03	EF
FIG-14.0	FIGURE IA ISOL	001	06/14/01	05/01/98	05/01/03	EF
TOTAL FOR PRFIG	17					

EOP: AP-RCS.3	TITLE: HIGH REACTOR COOLANT ACTIVITY	REV: 8 PAGE 1 of 6
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ROCHESTER GAS AND ELECTRIC CORPORATION

GINNA STATION

CONTROLLED COPY NUMBER 23

  
RESPONSIBLE MANAGER

6-14-2001  
EFFECTIVE DATE

CATEGORY 1.0

REVIEWED BY: \_\_\_\_\_

EOP: AP-RCS.3	TITLE: HIGH REACTOR COOLANT ACTIVITY	REV: 8 PAGE 2 of 6
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A. PURPOSE - This procedure provides guidance necessary to operate the plant with indication of high reactor coolant activity.

B. ENTRY CONDITIONS/SYMPTOMS

1. SYMPTOMS - The symptoms of HIGH REACTOR COOLANT ACTIVITY are;

- a. Unexplained increase in letdown line monitor, R-9,  
or
- b. Sampling indicates I-131 equivalent GREATER THAN  
.64  $\mu\text{Ci/gm}$  at 40 gpm letdown or GREATER THAN  
.46  $\mu\text{Ci/gm}$  at 60 gpm letdown  
or
- c. Sampling indicates gross degassed activity GREATER  
THAN 20  $\mu\text{Ci/gm}$ , or
- d. Sampling indicates that total specific activity  
exceeds 100/E.

EOP: AP-RCS.3	TITLE: HIGH REACTOR COOLANT ACTIVITY	REV: 8 PAGE 3 of 6
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STEP	ACTION/EXPECTED RESPONSE	RESPONSE NOT OBTAINED
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CAUTION

- o IF LETDOWN FLOW EXCEEDS 60 GPM, THEN LOCALLY MONITOR D/P ACROSS THE CVCS DI(S) TO VERIFY THAT FLOW IS CONTINUING AND THAT RELIEF VALVE, V-209, HAS NOT LIFTED.
- o LETDOWN FLOW THROUGH THE DI'S SHOULD BE LIMITED TO 90 GPM.

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NOTE: Conditions should be evaluated for site contingency reporting (Refer to EPIP-1.0, GINNA STATION EVENT EVALUATION AND CLASSIFICATION.

1 Verify RCS Activity:

- a. Direct RP Tech to sample RCS for activity
- b. RCS activity - GREATER THAN NORMAL (Check with RP Department for normal activity)
- b. IF normal activity verified, THEN direct I&C to check operability of R-9, letdown line monitor, AND return to normal operations.

EOP:

AP-RCS.3

TITLE:

HIGH REACTOR COOLANT ACTIVITY

REV: 8

PAGE 4 of 6

STEP

ACTION/EXPECTED RESPONSE

RESPONSE NOT OBTAINED

2 Increase Letdown Flow To  
60 GPM:

- |  |                                      |
|--|--------------------------------------|
| a. Verify deborating DI isolated -<br>DIVERT VLV CATION DEBOR DI<br>AOV-244 IN BYPASS POSITION | a. Place AOV-244 in bypass position. |
|--|--------------------------------------|
- b. Place letdown controllers in  
MANUAL at 60% open
- TCV-130
  - PCV-135
- c. Increase letdown flow as follows:
- 1) Close letdown orifice valve  
(AOV-200A or AOV-200B)
  - 2) Immediately open 60 gpm  
letdown orifice valve, AOV-202
- d. Adjust low pressure letdown  
pressure to approximately  
250 psig
- e. Place TCV-130 in AUTO at 105°F
- f. Place PCV-135 in AUTO at 250 psig
- g. Adjust charging pump speed and  
HCV-142 as necessary



EOP: AP-RCS.3	TITLE: HIGH REACTOR COOLANT ACTIVITY	REV: 8 PAGE 5 of 6
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STEP	ACTION/EXPECTED RESPONSE	RESPONSE NOT OBTAINED
3	Check Letdown Line Monitor, R-9 - LESS THAN 200 MR/HR ABOVE BACKGROUND	Evaluate conditions to determine whether local radiation emergency exists (Refer to EPIP 1-13, LOCAL RADIATION EMERGENCY).
<p>*****</p> <p style="text-align: center;"><u>CAUTION</u></p> <p>PLACING A NEW DI IN SERVICE MAY RESULT IN A POSITIVE OR NEGATIVE REACTIVITY ADDITION DUE TO A BORON CHANGE.</p> <p>*****</p>		
4	Direct RP Tech To Sample Letdown DI Efficiency - DECONTAMINATION FACTOR GREATER THAN 10	<u>IF</u> DI efficiency is <u>NOT</u> acceptable, <u>THEN</u> place a new mixed bed in service (Refer to S-3.2B, PLACING A MIXED BED DEMINERALIZER IN SERVICE - BORON CONCENTRATION DIFFERENT THAN RCS).
5	Evaluate AUX BLDG Radiation Levels:	
	a. Direct RP Tech to survey AUX BLDG	
	b. Check AUX BLDG radiation monitors - NORMAL	b. Perform the following:
	<ul style="list-style-type: none"> <li>• R-4</li> <li>• R-9</li> <li>• R-10B</li> <li>• R-13</li> <li>• R-14</li> </ul>	1) Direct RP Tech to survey AUX BLDG areas as necessary.  2) Evaluate conditions to determine whether local radiation emergency exists (Refer to EPIP 1-13, LOCAL RADIATION EMERGENCY).

EOP: AP-RCS.3	TITLE: HIGH REACTOR COOLANT ACTIVITY	REV: 8 PAGE 6 of 6
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STEP	ACTION/EXPECTED RESPONSE	RESPONSE NOT OBTAINED
6	Determine If Plant Operation Can Continue (Consult Plant staff if necessary) - OPERATION CAN CONTINUE	<u>IF</u> plant shutdown is required, <u>THEN</u> refer to 0-2.1, NORMAL SHUTDOWN TO HOT SHUTDOWN.
<u>NOTE:</u> Refer to 0-9.3, NRC IMMEDIATE NOTIFICATION, for reporting requirements.		
7	Notify Higher Supervision	
-END-		

EOP: FIG-14.0	TITLE: FIGURE 1A ISOL	REV: 1 PAGE 1 of 2
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Responsible Manager *Richard J. ...* Date 6-14-2001

# Turbine Bldg

