

Lewis Sumner
Vice President
Hatch Project Support

**Southern Nuclear
Operating Company, Inc.**
40 Inverness Parkway
Post Office Box 1295
Birmingham, Alabama 35201

Tel 205.992.7279
Fax 205.992.0341



May 21, 2001

Docket Nos. 50-321
50-366

HL-6090

U.S. Nuclear Regulatory Commission
ATTN: Document Control Desk
Washington, D.C. 20555

Edwin I. Hatch Nuclear Plant
Reporting of Changes and Errors in ECCS Evaluation Models

Ladies and Gentlemen:

The Code of Federal Regulations [10 CFR 50.46(a)(3)(i)] requires the licensee to estimate the impact of any change to or error in an emergency core cooling system (ECCS) evaluation model or in the application of such a model to determine whether the change or error is significant. Regulation 10 CFR 50.46 (a)(3)(ii) specifies reporting requirements based upon the sum of the absolute value of the changes or errors in calculated peak clad temperature (PCT). If the absolute sum of the changes or errors is significant (exceeds 50°F), a 30-day report is required. If not, the submittal of an annual report summarizing the effect of the changes or errors on the limiting ECCS analysis is required. This letter is intended to fulfill this reporting requirement.

Recently, two errors have been reported that affect the Plant Hatch loss-of-coolant accident (LOCA) analysis of record. One of these errors is "significant", in that it exceeds 50°F and requires a 30-day report. The absolute sum of both errors reported by General Electric (GE), the LOCA analysis vendor, totals 100°F. These two errors were reported to Southern Nuclear Operating Company (SNC) in references 1 and 2 and evaluated in Table 1.

The reported errors affect the licensing basis PCTs reported in the Plant Hatch Final Safety Analysis Reports (FSARs). However, margins to regulatory limits remain large and fuel thermal limits are unaffected. No changes in plant configuration or operation are required as a result of the letter. Southern Nuclear Company may not revise the Plant Hatch FSARs to reflect the PCT change reported in Table 1. This is because we plan to re-analyze our LOCA analysis of record later this year. The re-analysis is necessary because of our plan to load 10x10 GE fuel and would of course be performed with the latest version of the SAFER code.

Should you have any questions in this regard, please contact this office.

Respectfully submitted,

A handwritten signature in cursive script that reads "Lewis Sumner".

H. L. Sumner, Jr.

GKM/eb

A001

U. S. Nuclear Regulatory Commission

Page 2

May 21, 2001

References:

1. E-mail from Thomas Martin (Global Nuclear Fuel) to Ken Folk (SNC), "10 CFR 50.46 Notification - SAFER Condensation Error", dated May 8, 2001, with attached GE 10 CFR 50.46 Notification Letter 2001-01.
2. E-mail from Thomas Martin (GNF) to G. K. McElroy (SNC), "10 CFR 50.46 – Notification SAFER Pressure Rate", dated May, 10, 2001 with attached GE 10 CFR 50.46 Notification Letter 2001-02.

Enclosure: Table 1, Estimated PCT Change for Plant Hatch January 2001 through May 2001

cc: Southern Nuclear Operating Company
Mr. P. H. Wells, Nuclear Plant General Manager
SNC Document Management (R-Type A02.001)

U.S. Nuclear Regulatory Commission, Washington, D.C.
Mr. L. N. Olshan, Project Manager - Hatch

U.S. Nuclear Regulatory Commission, Region II
Mr. L. A. Reyes, Regional Administrator
Mr. J. T. Munday, Senior Resident Inspector - Hatch

Table 1 - Estimated PCT Change for Plant Hatch
January 2001 through May 2001

<u>Ref. No.</u>	<u>Description</u>	<u>PCT Change</u>	<u>Absolute Value of PCT Change</u>	<u>Comments</u>
1.	Coding error that over predicts condensate being added to lower plenum/jet pump region	+90°F	+90°F	Impact of change (>50°F) requires 30-day report.
2.	Use of inconsistent core exit steam flow in SAFER pressure equation	+10°F	+10°F	None
TOTAL	(Ref. 1, 2)	100°F	100°F	