

May 24, 2001

LICENSEE: Nuclear Management Company (NMC), LLC

FACILITY: Prairie Island Nuclear Generating Plant, Unit 1

SUBJECT: SUMMARY OF FEBRUARY 9, 2001, MEETING REGARDING THE  
PRAIRIE ISLAND, UNIT 1, STEAM GENERATOR INSERVICE INSPECTION  
RESULTS (TAC NO. MB1305)

On February 9, 2001, the Nuclear Regulatory Commission (NRC) staff met with representatives from NMC (the licensee) at the NRC's headquarters in Rockville, Maryland. The purpose of the meeting was to discuss the licensee's recent eddy current inspections of rows 1 and 2 of the steam generator tubes (also referred to as the "low row" tubes) at Prairie Island, Unit 1. In particular, the meeting was to address data quality issues relating to the low row U-bends and whether the data was of sufficient quality to ensure that structurally significant flaws could be reliably detected. Meeting attendees are listed in Enclosure 1.

During the meeting, the licensee provided the results of their eddy current inspections, focusing upon the low row U-bends in the two Prairie Island, Unit 1, steam generators. This information was presented to support the licensee's assertion that the eddy current test techniques applied during their most recent inspection could reliably detect significant tube flaws amidst the full range of eddy current noise signals found in the apex region of the low row U-bends in the Unit 1, steam generators. A copy of the licensee's handouts is enclosed (Enclosure 2).

At the meeting's conclusion, the NRC staff expressed a desire for additional information to clarify and elaborate on certain items in the licensee's presentation, and to address issues such as root cause analysis and application of the test results to upcoming operating cycles. By a letter dated February 12, 2001, the NRC staff forwarded the follow-up questions from the meeting to the licensee (ADAMS Accession No. ML010440022).

The licensee provided its response to the NRC staff's questions by letter dated February 28, 2001 (ADAMS Accession No. ML010660075). The NRC staff has reviewed the licensee's response. The intent was not to review the licensee's SG tube inspection program in detail or to endorse the licensee's operational assessment methodology or the findings. Rather, the NRC staff's primary objective was to contrast the conditions that exist in the apex region of the low row U-bends in the Prairie Island, Unit 1, steam generators with those conditions that existed in the apex region of the low row U-bends in the Indian Point, Unit 2 (IP2), steam generators at the time of the IP2 February 15, 2000, tube failure. The NRC staff also focused on the licensee's root cause evaluation of the circumferential indications found in the apex region of tube R1C52 in the #12 steam generator during the 2001 inspection.

It should be noted that the licensee, in its February 28, 2001, letter, did not provide a complete response to all of the questions forwarded to them on February 12, 2001. However, the NRC staff concluded that the licensee's response did provide sufficient information to address the concerns that motivated the meeting with the licensee. Specifically, the response is sufficient to

support the licensee's contention that the recent eddy current inspection of the apex region of the low row U-bends in the Prairie Island, Unit 1, steam generators was adequate to identify and remove from service tubes with structurally significant degradation. This conclusion is based primarily on the relatively low noise levels that exist in the apex region of the low row U-bends in the Prairie Island, Unit 1, steam generators, compared to the noise conditions that existed in the same regions of the IP2 steam generators. The NRC staff also did not identify any significant concerns with the licensee's root cause evaluation of the circumferential indications found in tube R1C52. However, the NRC staff has identified additional questions regarding the significance of noise levels in the tangential regions of the low row U-bends. The NRC staff plans to discuss these additional questions with the licensee and will document the discussions in a supplement to this meeting summary.

***/RA/***

Tae Kim, Senior Project Manager, Section 1  
Project Directorate III  
Division of Licensing Project Management  
Office of Nuclear Reactor Regulation

Docket No. 50-282

Enclosures: 1. List of Attendees  
2. Licensee Handout

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support the licensee's contention that the recent eddy current inspection of the apex region of the low row U-bends in the Prairie Island, Unit 1, steam generators was adequate to identify and remove from service tubes with structurally significant degradation. This conclusion is based primarily on the relatively low noise levels that exist in the apex region of the low row U-bends in the Prairie Island, Unit 1, steam generators, compared to the noise conditions that existed in the same regions of the IP2 steam generators. The NRC staff also did not identify any significant concerns with the licensee's root cause evaluation of the circumferential indications found in tube R1C52. However, the NRC staff has identified additional questions regarding the significance of noise levels in the tangential regions of the low row U-bends. The NRC staff plans to discuss these additional questions with the licensee and will document the discussions in a supplement to this meeting summary.

**/RA/**

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\*Previously Concurred

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ACCESSION NO. ML011440011

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October 2000

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FEBRUARY 9, 2001 MEETING  
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R. Pearson	NMC
D. Johnson	NMC
J. Sorenson	NMC
J. Begley	E-Mech (Consultant)