



**UNITED STATES  
NUCLEAR REGULATORY COMMISSION  
REGION II**

**SAM NUNN ATLANTA FEDERAL CENTER  
61 FORSYTH STREET SW SUITE 23T85  
ATLANTA, GEORGIA 30303-8931**

**April 10, 2001**

Southern Nuclear Operating Company, Inc.  
ATTN: Mr. H. L. Sumner, Jr.  
Vice President - Hatch Project  
P. O. Box 1295  
Birmingham, AL 35201-1295

**SUBJECT: EDWIN I. HATCH NUCLEAR POWER PLANT - NRC INSPECTION REPORT  
50-321/01-04, 50-366/01-04**

Dear Mr. Summer:

The purpose of this letter is to notify you that the U.S. Nuclear Regulatory Commission (NRC) Region II staff will conduct a Safety System Design and Performance Capability inspection at your Hatch facility during May 29-June 15, 2001. A team of five inspectors will perform the inspection. The inspection team will be lead by Mr. Frank Jape, Senior Project Manager, from the NRC Region II Office. The inspection will be conducted in accordance with baseline Inspection Procedure (IP) 71111.21, Safety System Design and Performance Capability.

The inspection objective will be to verify that design bases have been correctly implemented for the plant service water and the residual heat removal service water systems, and related systems, to ensure that the systems can be relied upon to meet functional requirements. It should be noted that a similarly focused 50.59 inspection (per IP 71111.02, Evaluations of Changes, Tests, or Experiments) will be conducted concurrently with the first week of the Safety System Design and Performance Capability Inspection.

During a telephone conversation on April 2, 2001, Mr. Jape of my staff, and Mr. Steve Tipps of your staff, confirmed arrangements for an information gathering site visit and the two-week onsite inspection. The schedule is as follows:

- Information gathering visit: May 7-11, 2001
- Onsite inspection: May 29-June 1, 2001, and June 11-15, 2001

The purpose of the information gathering visit is to obtain information and documentation outlined in the enclosure needed to support the inspection. Please contact Mr. Jape prior to preparing copies of the materials listed in the enclosure. The inspectors will try to minimize your administrative burden by specifically identifying only those documents required for inspection preparation.

During the information gathering visit, the team leader will also discuss the following inspection support administrative details: office space; specific documents requested to be made available to the team in their office space at the site; arrangements for plant site access; and the availability of knowledgeable plant engineering and licensing personnel to serve as points of contact during the inspection.



Thank you for your cooperation in this matter. If you have any questions regarding the information requested or the inspection, please contact me at (404) 562-4605, or Mr. Jape at (404) 562-4541.

Sincerely,

**/RA/**

Charles R. Ogle, Chief  
Engineering Branch  
Division of Reactor Safety

Docket Nos.: 50-321,50-366  
License Nos.: DPR-57, NPF-5

Enclosure: Information Request for the Safety System Design and  
Performance Capability Inspection

cc w/encl:  
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SNOPCO

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DATE	4/10/01	4/10/01					
E-MAIL COPY?	YES NO	YES NO	YES NO	YES NO	YES NO	YES NO	YES NO

OFFICIAL RECORD COPY

DOCUMENT NAME: S:\DRS\EB\hatchinf.wpd

**INFORMATION REQUEST FOR THE SAFETY SYSTEM DESIGN AND  
PERFORMANCE CAPABILITY INSPECTION:  
Report No. 50-321,366/01-04**

**PLANT SERVICE WATER & RHR SERVICE WATER**

Note: Electronic media is preferred if readily available (i.e., on computer disc).

- Site specific administrative procedures related to normal operation, abnormal operation, and emergency operation of the plant service water and residual heat removal service water systems, (PSW & RHRSW), including support systems, and other related systems. Other related systems include air, electrical, prelube, cooling water, ventilation, and system suction and discharge interfaces.
- Design criteria (i.e., design basis documents) for the PSW & RHRSW, in addition to support systems, including power sources for the intake structure cooling fans.
- PSW & RHRSW Technical Specification requirements and a list of associated surveillance test/calibration procedures for the components within the system. Include a list of instruments used in the EOPs that are Technical Specification related.
- Copies of applicable sections of the UFSAR for the PSW & RHRSW, and other related systems and copies of applicable sections of changes to the UFSAR which have yet to be docketed.
- PSW & RHRSW and other related systems piping and instrumentation drawings, one-line diagrams, electrical schematics, and wiring and logic diagrams, including a list of instruments and equipment that would mitigate the loss of PSW & RHRSW.
- A list of engineering calculations applicable to the PSW & RHRSW
- A list of plant modifications to the PSW & RHRSW systems implemented since 1994, including the minor modification that opens the concrete ports on top of the intake structure for equipment cooling and the associated 50.59 evaluation.
- List of existing temporary modifications and operator work arounds involving operation of the PSW & RHRSW systems.
- List of Condition Reports (CRs) and non-routine work requests initiated since 1994 affecting the systems. Include equipment damaged by river silt or debris over the past 10 years such as check valves and heat exchangers.
- System Health Report and Performance Trends for the PSW & RHRSW.
- Maintenance Rule Performance criteria for the PSW & RHRSW.

Enclosure

- Summary of corrective maintenance activities, including the maintenance rule event log, performed on the PSW & RHRSW in the past 12 months. Include a history of inadvertent SW pump breaker trips and related root cause determinations.
- Self-assessment performed on PSW & RHRSW systems in the last 24 months.
- Engineering evaluations and improvement plans for the PSW & RHRSW systems including the river water intake building.
- System description and operator training modules for the PSW & RHRSW systems.
- List of Operating Experience Program evaluations of industry, vendor, or NRC generic issues related to the PSW & RHRSW for the past 3 years.
- Provide a list of equipment in the PSW & RHRSW that changes state or is manually manipulated during a LOCA, LOOP, or SBO. Provide equipment failure rates over the past 10 years for these components.
- List of current instrumentation in PSW & RHRSW system with a Risk Achievement Worth (RAW) greater than 2.
- List of current age of installed instrumentation and expected service life.
- Contingency plans for handling high and low river water levels.
- List of the top 300 cutsets for the nominal maintenance unavailability and for the no maintenance unavailability cases on your most recent plant safety analysis.