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RS-01-056

March 26, 2001

U. S. Nuclear Regulatory Commission
ATTN: Document Control Desk
Washington, D.C. 20555-0001

Quad Cities Nuclear Power Station, Units 1 and 2
Facility Operating License Nos. DPR-29 and DPR-30
NRC Docket Nos. 50-254 and 50-265

Subject: Revision E to Request for Technical Specifications Change for Quad Cities Nuclear Power Station, Units 1 and 2, to Implement Improved Standard Technical Specifications

- References:
- 1) Letter from R. M. Krich (Commonwealth Edison Company) to U.S. NRC, "Request for Technical Specifications Changes for Dresden Nuclear Power Station, Units 2 and 3, LaSalle County Station, Units 1 and 2, and Quad Cities Nuclear Power Station, Units 1 and 2, to Implement Improved Standard Technical Specifications," dated March 3, 2000
 - 2) Letter from R. M. Krich (Commonwealth Edison Company) to U.S. NRC, "Revision A to Request for Technical Specifications Changes for Dresden Nuclear Power Station, Units 2 and 3, LaSalle County Station, Units 1 and 2, and Quad Cities Nuclear Power Station, Units 1 and 2, to Implement Improved Standard Technical Specifications," dated June 5, 2000
 - 3) Letter from R. M. Krich (Commonwealth Edison Company) to U.S. NRC, "Revision B to Request for Technical Specifications Changes for Dresden Nuclear Power Station, Units 2 and 3, LaSalle County Station, Units 1 and 2, and Quad Cities Nuclear Power Station, Units 1 and 2, to Implement Improved Standard Technical Specifications," dated September 1, 2000
 - 4) Letter from R. M. Krich (Commonwealth Edison Company) to U.S. NRC, "Revision C to Request for Technical Specifications Changes for Dresden Nuclear Power Station, Units 2 and 3, LaSalle County Station, Units 1 and 2, and Quad Cities Nuclear Power Station, Units 1 and 2, to Implement Improved Standard Technical Specifications," dated December 18, 2000

A001

- 5) Letter from R. M. Krich (Commonwealth Edison Company) to U.S. NRC, "Revision D to Request for Technical Specifications Changes for Dresden Nuclear Power Station, Units 2 and 3, LaSalle County Station, Units 1 and 2, and Quad Cities Nuclear Power Station, Units 1 and 2, to Implement Improved Standard Technical Specifications," dated February 15, 2001
- 6) Letter from R. M. Krich (Exelon Generation Company, LLC) to U.S. NRC, "Revision E to Request for Technical Specifications Changes and Proposed License Conditions Supporting the Implementation of Improved Standard Technical Specifications for Dresden Nuclear Power Station, Units 2 and 3, LaSalle County Station, Units 1 and 2, and Quad Cities Nuclear Power Station, Units 1 and 2," dated February 28, 2001

In Reference 1, in accordance with 10 CFR 50.90, "Application for amendment of license or construction permit," Commonwealth Edison (ComEd) Company, now Exelon Generation Company (EGC), LLC, proposed to amend Appendix A, Technical Specifications (TS) of Facility Operating License Nos. DPR-19, DPR-25, NPF-11, NPF-18, DPR-29 and DPR-30 for Dresden Nuclear Power Station, Units 2 and 3, LaSalle County Station, Units 1 and 2, and Quad Cities Nuclear Power Station, Units 1 and 2, respectively. The proposed changes revise the Dresden Nuclear Power Station, Units 2 and 3, LaSalle County Station, Units 1 and 2, and Quad Cities Nuclear Power Station, Units 1 and 2, current Technical Specifications (CTS) to a format and content consistent with NUREG-1433, Revision 1, "Standard Technical Specifications for General Electric Plants, BWR 4," and NUREG-1434, Revision 1, "Standard Technical Specifications for General Electric Plants, BWR 6," as applicable. References 2, 3, 4, 5, and 6 subsequently supplemented the proposed amendment.

This letter provides the NRC with further modification to the proposed changes for the Quad Cities Nuclear Power Station, Units 1 and 2.

The modification is a change to the Allowable Value (AV) for the High Pressure Coolant Injection (HPCI) System Isolation, Function 3a., HPCI Steam Line Flow - High. The revised value is provided in the attached Improved Technical Specifications (ITS) page. The AV change is a result of instrument testing conducted for ITS implementation.

This letter also provides a request to delete a License Condition requested in Reference 6 for Quad Cities Nuclear Power Station Unit 2. In the License Conditions included in Reference 6, EGC requested that for ITS Table 3.3.1.1-1 Function 5, Main Steam Isolation Valve – Closure, the former setpoint (i.e., $\leq 10\%$ closed) be in effect until the unit startup after the first outage of sufficient duration following the ITS implementation date. During a recent planned maintenance outage on Unit 2, the setpoints were adjusted to meet the requirements of ITS. Therefore, this specific License Condition is no longer needed for Unit 2.

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The ITS modification and the proposed license condition change have been reviewed and approved by the Plant Operations Review Committee and the Nuclear Safety Review Board in accordance with the Quality Assurance Program. We have reviewed the information contained in this letter and its attachment and determined that it does not affect the information supporting a finding of no significant hazards consideration provided in References 1, 2, 3, 4, 5, and 6.

EGC is notifying the State of Illinois of this modification to the previously submitted license amendment requests by transmitting a copy of this letter, including attachments and enclosures, to the designated State Official.

Should you have any questions concerning this information, please contact Mr. J. V. Sipek at (630) 663-3741.

Respectfully,



R. M. Krich
Director-Licensing
Mid-West Regional Operating Group

Attachments: Affidavit
Attachment 1 – Revision E to Quad Cities Improved Technical Specifications Document

cc: Regional Administrator - NRC Region III
NRC Senior Resident Inspector - Dresden Nuclear Power Station
NRC Senior Resident Inspector - LaSalle County Station
NRC Senior Resident Inspector - Quad Cities Nuclear Power Station
Office of Nuclear Facility Safety - Illinois Department of Nuclear Safety

STATE OF ILLINOIS)
COUNTY OF DUPAGE)
IN THE MATTER OF)
COMMONWEALTH EDISON (COMED) COMPANY) Docket Nos.
QUAD CITIES NUCLEAR POWER STATION - UNITS 1 and 2) 50- 254 and 50-265

SUBJECT: Revision E to Request for Technical Specifications Changes for Quad
Cities Nuclear Power Station, Units 1 and 2, to Implement Improved
Standard Technical Specifications

AFFIDAVIT

I affirm that the content of this transmittal is true and correct to the best of my
knowledge, information and belief.

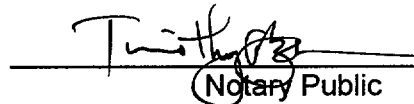
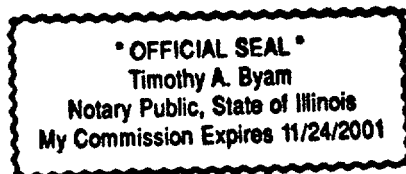


R. M. Krich
Director, Licensing
Mid-West Regional Operating Group

Subscribed and sworn to before me, a Notary Public in and

for the State above named, this 26th day of

March, 2001.



Notary Public

ATTACHMENT 1

**Revision E to Quad Cities Nuclear Power Station, Units 1 and 2
Proposed Improved Technical Specifications Submittal
Dated March 3, 2000**

**REVISION E TO QUAD CITIES NUCLEAR POWER STATION PROPOSED
IMPROVED TECHNICAL SPECIFICATIONS
SUMMARY OF CHANGES**

This attachment provides a brief summary of the changes in Revision E of the proposed Improved Technical Specifications (ITS) submittal for Quad Cities Nuclear Power Station. The original Technical Specifications amendment request (i.e., Revision 0) was submitted to the NRC by letter dated March 3, 2000.

Reference 2, Revision A to our ITS submittal, provided the Allowable Values (AVs) for the instrumentation in ITS Section 3.3. Revision A did not include mechanical or time delay relay devices. The AVs are a result of the application of the Exelon Setpoint Methodology (i.e., Nuclear Engineering Standard NES-EIC-20.4, "Analysis of Instrument Channel Setpoint Error and Instrument Loop Accuracy," or General Electric Company Report NEDC-31336P-A, "General Electric Instrument Setpoint Methodology," for nuclear instrumentation system functions only.

During ITS implementation efforts, it was discovered that the AV for the High Pressure Coolant Injection (HPCI) high steam flow isolation function results in an unacceptable field setpoint on Unit 1. Due to the existing system configuration the proposed AV (i.e., $\leq 151\%$ rated steam flow) results in a field setpoint that is susceptible to exiting flow-induced process noise levels thereby increasing the potential for spurious HPCI isolations.

For this reason, we are proposing to modify the AV for Unit 1 from $\leq 151\%$ rated steam flow to $\leq 286\%$ rated steam flow. This change impacts ITS Table 3.3.6.1-1, "Primary Containment Isolation Instrumentation," Function 3.a. A revised AV for Unit 1 has been generated in accordance with the Exelon setpoint methodology described above. The Unit 2 AV for this function remains unchanged.

Changes to our submittal are attached including the discard and insertion instructions.

DISCARD AND INSERTION INSTRUCTIONS

VOLUME 3	
Section 3.3	
Discard	Insert
ITS Page 3.3.6.1-6	ITS Page 3.3.6.1-6

VOLUME 4	
Section 3.3	
Discard	Insert
ISTS Page 3.3-58	ISTS Page 3.3-58

Primary Containment Isolation Instrumentation
3.3.6.1

Table 3.3.6.1-1 (page 2 of 3)
Primary Containment Isolation Instrumentation

FUNCTION	APPLICABLE MODES OR OTHER SPECIFIED CONDITIONS	REQUIRED CHANNELS PER TRIP SYSTEM	CONDITIONS REFERENCED FROM REQUIRED ACTION C.1	SURVEILLANCE REQUIREMENTS	ALLOWABLE VALUE
3. High Pressure Coolant Injection (HPCI) System Isolation					
a. HPCI Steam Line Flow - High	1,2,3	1	F	SR 3.3.6.1.2 SR 3.3.6.1.3 SR 3.3.6.1.6 SR 3.3.6.1.7	<div style="text-align: right;"> $\leq 286\%$ RATED STEAM FLOW (UNIT 1) $\leq 151\%$ rated steam flow (UNIT 2) </div> <div style="display: flex; justify-content: space-around;"> <div> ≤ 3.2 seconds and ≤ 8.8 seconds </div> <div> ≥ 113.0 psig </div> <div> ≤ 2.43 psig </div> <div> $\leq 169^{\circ}\text{F}$ </div> </div> <div style="display: flex; justify-content: space-around;"> <div> <div style="border: 1px solid black; padding: 2px;">A</div> <div style="border: 1px solid black; padding: 2px;">C</div> </div> <div> <div style="border: 1px solid black; padding: 2px;">B</div> <div style="border: 1px solid black; padding: 2px;">C</div> </div> <div> <div style="border: 1px solid black; padding: 2px;">A</div> <div style="border: 1px solid black; padding: 2px;">C</div> </div> <div> <div style="border: 1px solid black; padding: 2px;">A</div> <div style="border: 1px solid black; padding: 2px;">C</div> </div> </div> <div style="text-align: right; margin-top: -20px;"> <div style="border: 1px solid black; padding: 2px;">E</div> </div>
b. HPCI Steam Line Flow - Timer	1,2,3	1	F	SR 3.3.6.1.2 SR 3.3.6.1.6 SR 3.3.6.1.7	<div style="display: flex; justify-content: space-around;"> <div> ≥ 3.2 seconds and ≤ 8.8 seconds </div> <div> ≥ 113.0 psig </div> </div> <div style="display: flex; justify-content: space-around;"> <div> <div style="border: 1px solid black; padding: 2px;">B</div> <div style="border: 1px solid black; padding: 2px;">C</div> </div> </div>
c. HPCI Steam Supply Line Pressure - Low	1,2,3	2	F	SR 3.3.6.1.2 SR 3.3.6.1.3 SR 3.3.6.1.6 SR 3.3.6.1.7	<div style="display: flex; justify-content: space-around;"> <div> ≥ 113.0 psig </div> </div> <div style="display: flex; justify-content: space-around;"> <div> <div style="border: 1px solid black; padding: 2px;">A</div> <div style="border: 1px solid black; padding: 2px;">C</div> </div> </div>
d. Drywell Pressure - High	1,2,3	2	F	SR 3.3.6.1.2 SR 3.3.6.1.4 SR 3.3.6.1.7	<div style="display: flex; justify-content: space-around;"> <div> ≤ 2.43 psig </div> </div> <div style="display: flex; justify-content: space-around;"> <div> <div style="border: 1px solid black; padding: 2px;">A</div> <div style="border: 1px solid black; padding: 2px;">C</div> </div> </div>
e. HPCI Turbine Area Temperature - High	1,2,3	2	F	SR 3.3.6.1.5 SR 3.3.6.1.6 SR 3.3.6.1.7	<div style="display: flex; justify-content: space-around;"> <div> $\leq 169^{\circ}\text{F}$ </div> </div> <div style="display: flex; justify-content: space-around;"> <div> <div style="border: 1px solid black; padding: 2px;">A</div> <div style="border: 1px solid black; padding: 2px;">C</div> </div> </div>
4. Reactor Core Isolation Cooling (RCIC) System Isolation					
a. RCIC Steam Line Flow - High	1,2,3	1	F	SR 3.3.6.1.2 SR 3.3.6.1.4 SR 3.3.6.1.7	<div style="display: flex; justify-content: space-around;"> <div> $\leq 175\%$ rated steam flow </div> </div> <div style="display: flex; justify-content: space-around;"> <div> <div style="border: 1px solid black; padding: 2px;">A</div> <div style="border: 1px solid black; padding: 2px;">C</div> </div> </div>
b. RCIC Steam Line Flow - Timer	1,2,3	1	F	SR 3.3.6.1.2 SR 3.3.6.1.6 SR 3.3.6.1.7	<div style="display: flex; justify-content: space-around;"> <div> ≥ 3.2 seconds and ≤ 8.8 seconds </div> </div> <div style="display: flex; justify-content: space-around;"> <div> <div style="border: 1px solid black; padding: 2px;">B</div> <div style="border: 1px solid black; padding: 2px;">C</div> </div> </div>
c. RCIC Steam Supply Line Pressure - Low	1,2,3	4(a)	F	SR 3.3.6.1.2 SR 3.3.6.1.4 SR 3.3.6.1.7	<div style="display: flex; justify-content: space-around;"> <div> ≥ 54 psig </div> </div> <div style="display: flex; justify-content: space-around;"> <div> <div style="border: 1px solid black; padding: 2px;">A</div> <div style="border: 1px solid black; padding: 2px;">C</div> </div> </div>
d. RCIC Turbine Area Temperature - High	1,2,3	2	F	SR 3.3.6.1.5 SR 3.3.6.1.6 SR 3.3.6.1.7	<div style="display: flex; justify-content: space-around;"> <div> $\leq 169^{\circ}\text{F}$ </div> </div> <div style="display: flex; justify-content: space-around;"> <div> <div style="border: 1px solid black; padding: 2px;">A</div> <div style="border: 1px solid black; padding: 2px;">C</div> </div> </div>

(continued)

(a) Only inputs into one trip system.

<CTS>

Primary Containment Isolation Instrumentation 3.3.6.1

3.2.A-1
4.2.A-1
(DOC M.4)

Table 3.3.6.1-1 (page 2 of 6)
Primary Containment Isolation Instrumentation

FUNCTION	APPLICABLE MODES OR OTHER SPECIFIED CONDITIONS	REQUIRED CHANNELS PER TRIP SYSTEM	CONDITIONS REFERENCED FROM REQUIRED ACTION C.1	SURVEILLANCE REQUIREMENTS	ALLOWABLE VALUE
2. Primary Containment Isolation					
a. Reactor Vessel Water Level - Low	1,2,3	XZK		SR 3.3.6.1.1 SR 3.3.6.1.2 SR 3.3.6.1.3 SR 3.3.6.1.6 SR 3.3.6.1.7 SR 3.3.6.1.8	≥ 11.8 inches ≤ 2.93
b. Drywell Pressure - High	1,2,3	XZK		SR 3.3.6.1.1 SR 3.3.6.1.2 SR 3.3.6.1.3 SR 3.3.6.1.6 SR 3.3.6.1.7 SR 3.3.6.1.8	≤ 11.92 psig add SR 3.3.6.1.4 70
c. Drywell Radiation - High	1,2,3	X1K	F	SR 3.3.6.1.1 SR 3.3.6.1.2 SR 3.3.6.1.6 SR 3.3.6.1.7 SR 3.3.6.1.8	≤ 1000 R/hr
d. Reactor Building Exhaust Radiation - High	1,2,3	[2]	H	SR 3.3.6.1.1 SR 3.3.6.1.2 SR 3.3.6.1.6 SR 3.3.6.1.7 SR 3.3.6.1.8	≤ [60] mR/hr
e. Refueling Floor Exhaust Radiation - High	1,2,3	[2]	H	SR 3.3.6.1.1 SR 3.3.6.1.2 SR 3.3.6.1.6 SR 3.3.6.1.7 SR 3.3.6.1.8	≤ [20] mR/hr
f. Manual Initiation	1,2,3	[1 per group]	G	SR 3.3.6.1.7	NA
3. High Pressure Coolant Injection (HPCI) System Isolation					
a. HPCI Steam Line Flow - High	1,2,3	X1K	F	SR 3.3.6.1.1 SR 3.3.6.1.2 SR 3.3.6.1.3 SR 3.3.6.1.6 SR 3.3.6.1.7 SR 3.3.6.1.8	≤ 151 rated steam flow (UNIT 2) ≤ 286% RATED STEAM FLOW (UNIT 1)

Primary Containment Isolation Instrumentation
3.3.6.1

Table 3.3.6.1-1 (page 2 of 3)
Primary Containment Isolation Instrumentation

FUNCTION	APPLICABLE MODES OR OTHER SPECIFIED CONDITIONS	REQUIRED CHANNELS PER TRIP SYSTEM	CONDITIONS REFERENCED FROM REQUIRED ACTION C.1	SURVEILLANCE REQUIREMENTS	ALLOWABLE VALUE
3. High Pressure Coolant Injection (HPCI) System Isolation					
a. HPCI Steam Line Flow - High	1,2,3	1	F	SR 3.3.6.1.2 SR 3.3.6.1.3 SR 3.3.6.1.6 SR 3.3.6.1.7	≤ 286% rated steam flow (Unit 1) ≤ 151% rated steam flow (Unit 2)
b. HPCI Steam Line Flow - Timer	1,2,3	1	F	SR 3.3.6.1.2 SR 3.3.6.1.6 SR 3.3.6.1.7	≥ 3.2 seconds and ≤ 8.8 seconds
c. HPCI Steam Supply Line Pressure - Low	1,2,3	2	F	SR 3.3.6.1.2 SR 3.3.6.1.3 SR 3.3.6.1.6 SR 3.3.6.1.7	≥ 113.0 psig
d. Drywell Pressure - High	1,2,3	2	F	SR 3.3.6.1.2 SR 3.3.6.1.4 SR 3.3.6.1.7	≤ 2.43 psig
e. HPCI Turbine Area Temperature - High	1,2,3	2	F	SR 3.3.6.1.5 SR 3.3.6.1.6 SR 3.3.6.1.7	≤ 169°F
4. Reactor Core Isolation Cooling (RCIC) System Isolation					
a. RCIC Steam Line Flow - High	1,2,3	1	F	SR 3.3.6.1.2 SR 3.3.6.1.4 SR 3.3.6.1.7	≤ 175% rated steam flow
b. RCIC Steam Line Flow - Timer	1,2,3	1	F	SR 3.3.6.1.2 SR 3.3.6.1.6 SR 3.3.6.1.7	≥ 3.2 seconds and ≤ 8.8 seconds
c. RCIC Steam Supply Line Pressure - Low	1,2,3	4 ^(a)	F	SR 3.3.6.1.2 SR 3.3.6.1.4 SR 3.3.6.1.7	≥ 54 psig
d. RCIC Turbine Area Temperature - High	1,2,3	2	F	SR 3.3.6.1.5 SR 3.3.6.1.6 SR 3.3.6.1.7	≤ 169°F

(continued)

(a) Only inputs into one trip system.