

MAR 15 2001

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U. S. Nuclear Regulatory Commission
Document Control Desk
Washington, DC 20555

Gentlemen:

**MONTHLY OPERATING REPORT
SALEM GENERATING STATION UNIT 1
DOCKET NO. 50-272**

In compliance with Section 6.9, Reporting Requirements for the Salem Unit 1 Technical Specifications, the operating statistics for **February 2001** are being forwarded. Also being forwarded, pursuant to the requirements of 10CFR50.59(b), is a summary of changes, tests, and experiments that were implemented in **February 2001**. Future summaries of changes, tests, and experiments that are required to be reported pursuant to the requirements of 10CFR50.59(b) will be reported concurrently with the 10CFR50.71(e) submittal.

Sincerely,

A handwritten signature in black ink, appearing to read "D. F. Gardhow", written over a horizontal line.

D. F. Gardhow
Vice President - Operations

RAR
Attachments

C Distribution

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DOCKET NO.: 50-272
 UNIT: Salem 1
 DATE: 03/09/01
 COMPLETED BY: F. Todd
 TELEPHONE: (856) 339-1316

Reporting Period February 2001

OPERATING DATA REPORT

Design Electrical Rating (MWe-Net)
 Maximum Dependable Capacity (MWe-Net)

No. of hours reactor was critical
 No. of hours generator was on line (service hours)
 Unit reserve shutdown hours
 Net Electrical Energy (MWH)

1067		
1031		
Month	Year-to-date	Cumulative
672	1416	128424
672	1416	123996
0	0	0
743455	1566557	125140532

UNIT SHUTDOWNS

NO.	DATE	TYPE F=FORCED S=SCHEDULED	DURATION (HOURS)	REASON (1)	METHOD OF SHUTTING DOWN THE REACTOR (2)	CORRECTIVE ACTION/ COMMENT

(1) Reason

- A - Equipment Failure (Explain)
- B - Maintenance or Test
- C - Refueling
- D - Regulatory Restriction
- E - Operator Training/License Examination
- F - Administrative
- G - Operational Error (Explain)
- H - Other

(2) Method

- 1 - Manual
- 2 - Manual Trip/Scram
- 3 - Automatic Trip/Scram
- 4 - Continuation
- 5 - Other (Explain)

DOCKET NO.: 50-272
UNIT: Salem 1
DATE: 03/09/01
COMPLETED BY: R. Ritzman
TELEPHONE: (856) 339-1445

Summary Of Monthly Operating Experience

- Salem 1 entered February at approximately 100% power and operated there for the entire month.
- At the end of February, Salem 1 completed 80 days of continuous on-line operation.

DOCKET NO.: 50-272
UNIT: Salem 1
DATE: 03/09/01
COMPLETED BY: R. Ritzman
TELEPHONE: (856) 339-1445

SUMMARY OF CHANGES, TESTS, AND EXPERIMENTS
FOR THE SALEM GENERATING STATION UNIT 1

MONTH February 2001

The following items completed during **February 2001** have been evaluated to determine:

1. If the probability of occurrence or the consequences of an accident or malfunction of equipment important to safety previously evaluated in the safety analysis report may be increased; or
2. If a possibility for an accident or malfunction of a different type than any evaluated previously in the safety analysis report may be created; or
3. If the margin of safety as defined in the basis for any technical specification is reduced.

The 10CFR50.59 Safety Evaluations showed that these items did not create a new safety hazard to the plant nor did they affect the safe shutdown of the reactor. These items did not change the plant effluent releases and did not alter the existing environmental impact. The 10CFR50.59 Safety Evaluations determined that no unreviewed safety or environmental questions are involved.

Design Changes Summary of Safety Evaluation

1EE-0430, Package 1, Setpoint Change for the Fire Protection Jockey Pump Start Switch. This design change increases the start setpoint for a fire water jockey pump from 110 psig to 115 psig. This setpoint change is being made to avoid inadvertent starts of the Hope Creek electric fire pump when the Salem and Hope Creek fire water systems are cross-tied. The Hope Creek electric fire pump start setpoint is also 110 psig.

This design change does not reduce the ability of the fire water jockey pump to maintain system pressure or affect the fire water system from functioning per design. Safe shutdown capability is also maintained; therefore, the dose consequences of accidents are not impacted. This design change does not increase the possibility or

SUMMARY OF CHANGES, TESTS, AND EXPERIMENTS
FOR THE SALEM GENERATING STATION UNIT 1 – Cont'd

consequences of any accident or malfunction, does not reduce the margin of safety, and does not involve an Unreviewed Safety Question.

80015933, Replacement of Portable Liquid Radwaste Processing Equipment. This design change replaced a portable radioactive waste processing system with one of similar treatment capacity, but lower operating pressures. In order to support this change-out, piping and electrical configuration changes were also required.

The design and licensing bases of the Waste Liquid Disposal system and its supporting systems were not impacted by this design change. This design change does not increase the possibility or consequences of any accident or malfunction, does not reduce the margin of safety, and does not involve an Unreviewed Safety Question.

80021000 and 80022594, Heater Drain Pump Piping Modifications. This design change replaces piping associated with the heater drain pumps with flexible jacketed hose. The affected lines are the stuffing box cooling inlet and outlet line and the seal water inlet line. The design change does not affect system flow characteristics and the equipment affected by this design change is not safety related. This design change does not increase the possibility or consequences of any accident or malfunction, does not reduce the margin of safety, and does not involve an Unreviewed Safety Question.

Temporary Modifications Summary of Safety Evaluations

There were no reportable changes in this category implemented during February 2001.

Procedures Summary of Safety Evaluations

There were no reportable changes in this category implemented during February 2001.

UFSAR Change Notices Summary of Safety Evaluations

SCN 00-045, Net Positive Suction Head for Service Water System Pumps. This UFSAR change notice revises the available and required Net Positive Suction Head description for the service water pumps. The previous calculation did not include all of the bounding licensing basis assumptions.

This UFSAR change notice does not affect the actual system operation or any components. The plant installed equipment is capable of performing in accordance with the new assumptions. This UFSAR change notice consists of calculation changes only. Therefore, this UFSAR change notice does not increase the possibility or consequences of any accident or malfunction, does not reduce the margin of safety, and does not involve an Unreviewed Safety Question.

SUMMARY OF CHANGES, TESTS, AND EXPERIMENTS
FOR THE SALEM GENERATING STATION UNIT 1 – Cont'd

SCN 01-002, PSEG Nuclear Organization Change. This UFSAR change notice involves an organizational change to the PSEG Nuclear structure, providing greater emphasis on a one-site philosophy and equipment reliability. This organizational change created the new position of Vice President – Nuclear Reliability and incorporated the role of the Vice President – Plant Support with the role of the Vice President – Maintenance. Some line functions now report to a different vice president; however, these organizational changes do not eliminate or alter any functions that are important to safety.

This UFSAR change notice consists of organizational changes that are administrative in nature. Therefore, this UFSAR change notice does not increase the possibility or consequences of any accident or malfunction, does not reduce the margin of safety, and does not involve an Unreviewed Safety Question.