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United States Nuclear Regulatory Commission
ATTENTION: Document Control Desk
Washington, DC 20555

SHEARON HARRIS NUCLEAR POWER PLANT
DOCKET NO. 50-400/LICENSE NO. NPF-63
RESPONSE TO REQUEST FOR ADDITIONAL INFORMATION
SPENT FUEL POOL WATER LEVEL AND REVISED FUEL HANDLING ACCIDENT ANALYSES

Dear Sir or Madam:

Per telephone conversation with the NRC staff on 3/17/99, Harris Nuclear Plant (HNP) submits additional information for the proposed Spent Fuel Pool Water Level and Revised Fuel Handling Accident Analyses License Amendment request, dated September 1, 1998.

NRC Questions:

1. What is the value of the atmospheric dispersion coefficient (X/Q) used in evaluating the Main Control Room dose during a radiological accident?
2. Has the proposed submittal affected the Main Control Room dose with respect to the most limiting accident?

HNP Response:

1. The X/Q value in evaluating Main Control Room dose during a radiological accident is $7.68 \times 10^{-3}$ sec/m$^3$.
2. The HNP FSAR states that the LOCA event is the most limiting event for control room habitability dose criteria. The increase in dose resulting from the revised fuel handling accident analyses, described in the September 1, 1998 submittal, is small relative to the Main Control Room dose as a result of a LOCA. The LOCA accident remains the most limiting accident with respect to Main Control Room dose.

Please refer any questions regarding this submittal to Mr. J. H. Eads at (919) 362-2646.

Sincerely,

James Scarola

MSE/mse

c: Mr. J. B. Brady, NRC Sr. Resident Inspector
Mr. Mel Fry, Acting Director, N. C. DEHNR
Mr. R. J. Laufer, NRC Project Manager
Mr. L. A. Reyes, NRC Regional Administrator