

April 9, 2001

Mr. John G. Williams, Director
Nuclear Reactor Laboratory
University of Arizona
Tucson, AZ 85721-0020

SUBJECT: UNIVERSITY OF ARIZONA RESEARCH REACTOR - AMENDMENT ON
ADMINISTRATIVE REQUIREMENTS (TAC NO. MB1410)

Dear Mr. Williams:

The U.S. Nuclear Regulatory Commission (Commission) has issued the enclosed Amendment No. 18 to Facility License No. R-52 for the University of Arizona research reactor. The amendment consists of changes to the Technical Specifications (TS) in response to your application of March 7, 2001.

The amendment changes reporting requirements to be consistent with the current NRC organization and updates wording applicable to 10 CFR 50.59 changes, test and experiments. A copy of the safety evaluation supporting Amendment No. 18 is also enclosed.

Sincerely,

/RA/

Marvin M. Mendonca, Senior Project Manager
Events Assessment, Generic Communications and
Non-Power Reactors Branch
Division of Regulatory Improvement Programs
Office of Nuclear Reactor Regulation

Docket No. 50-113

Enclosures:

1. Amendment No. 18
2. Safety Evaluation

cc w/enclosures:

Please see next page

University of Arizona

Docket No. 50-113

cc:

Office of the Mayor
P.O. Box 27210
Tucson, AZ 85726-7210

Arizona Radiation Regulatory
Agency
4814 S. 40th Street
Phoenix, AZ 85040

Mr. Harry J. Doane, Reactor Supervisor
Nuclear Reactor Laboratory
University of Arizona
Tucson, AZ 85721

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UNIVERSITY OF ARIZONA

DOCKET NO. 50-113

AMENDMENT TO FACILITY LICENSE

Amendment No. 18
License No. R-52

1. The U.S. Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for an amendment to Facility License No. R-52 filed by the University of Arizona (the licensee) on March 7, 2001, conforms to the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the regulations of the Commission as stated in Chapter I of Title 10 of the *Code of Federal Regulations* (10 CFR);
 - B. The facility will be operated in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance that (i) the activities authorized by this amendment can be conducted without endangering the health and safety of the public and (ii) such activities will be conducted in compliance with the regulations of the Commission;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public;
 - E. This amendment is issued in accordance with the regulations of the Commission as stated in 10 CFR Part 51, and all applicable requirements have been satisfied; and
 - F. Prior notice of this amendment was not required by 10 CFR 2.105 and publication of a notice for this amendment is not required by 10 CFR 2.106.

2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the enclosure to this license amendment, and paragraph 2.C(2) of Facility License No. R-52 is hereby amended to read as follows:

(2) Technical Specifications

The Technical Specifications contained in Appendix A, as revised through Amendment No. 18, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of the date of its issuance.

FOR THE NUCLEAR REGULATORY COMMISSION

Marvin M. Mendonca, Senior Project Manager
Events Assessment, Generic Communications and
Non-Power Reactors Branch
Division of Regulatory Improvement Programs
Office of Nuclear Reactor Regulation

Enclosure:
Appendix A, Technical
Specifications Changes

Date of Issuance: April 9, 2001

ENCLOSURE TO LICENSE AMENDMENT No. 18

FACILITY LICENSE NO. R-52

DOCKET NO. 50-113

Replace the following pages of Appendix A, Technical Specifications, with the enclosed pages. The revised pages are identified by amendment number and contain vertical lines indicating the areas of change.

Remove

27
32
33

Insert

27
32
33

6.2 Review

- a. There shall be a Reactor Committee which shall review reactor operations to assure that the facility is operated in a manner consistent with public safety and within the terms of the facility license.
- b. The responsibility of the Committee includes, but is not limited to, the following:
 1. Review and approval of experiments utilizing the reactor facilities;
 2. Review and approval of all proposed changes to the facility, procedures, and Technical Specifications;
 3. Determination of whether a proposed change, test, or experiment would constitute a license amendment pursuant to 10 CFR 50.59(c)(2) as outlined in UARR 165;
 4. Review of the operation and operational records of the facility;
 5. Review of abnormal performance of plant equipment and operating anomalies;
 6. Review of unusual or abnormal occurrences and incidents which are reportable under 10 CFR 20 and 10 CFR 50;
 7. Review and audit of the retraining and requalification program for the operating staff; and
 8. Biennial audit of the Emergency Plan.
- c. The Committee shall be composed of at least five members, and shall include a health physicist and members competent in the field of reactor operations, radiation science, or reactor engineering. The membership of the Committee shall be such as to maintain a high level degree of technical proficiency.
- d. The Committee shall establish a written charter defining such matters as the authority of the Committee, review and audit functions, and other such administrative provisions as are required for effective functioning of the Committee. Minutes of all meetings of the Committee shall be kept and submitted to committee members and to the Vice President for Research and Graduate Studies in a timely manner.
- e. A quorum of the Committee shall consist of not less than three members of the Committee and shall include the chairman or his designee.
- f. The Committee shall meet at least quarterly.

6.7 Reporting Requirements

In addition to the requirements of applicable regulations, and in no way substituting therefor, reports shall be made to the NRC as follows:

- a. A report within 24 hours by telephone and telegraph or telefax (FAX) to the responsible NRC organization as listed in the Emergency Kit and posted as deemed necessary by the Reactor Committee of:
 1. Any accidental off-site release of radioactivity above limits permitted by 10 CFR 20, whether or not the release resulted in property damage, personal injury, or exposure;
 2. Any violation of a Safety Limit; and
 3. Any reportable occurrences as defined in Section 1.0 (Reportable Occurrence) of these specifications in writing.
- b. A written report within ten days to the U.S. Nuclear Regulatory Commission, Attn: Document Control Desk, Washington D.C. 20555, with a copy to the responsible NRC facility inspector of:
 1. Any significant variation of measured values from a corresponding predicted value of previously measured value of safety-connected operating characteristics occurring during operation of the reactor;
 2. Incidents or conditions relating to operation of the facility which prevented or could have prevented the performance of engineered safety features as described in these specifications;
 3. Any reportable occurrences as defined in Section 1.0 of these specifications;
 4. Any violation of a Safety Limit; and
 5. Any accidental off-site release of radioactivity above limits permitted by 10 CFR 20, whether or not the release resulted in property damage, personal injury, or exposure.
- c. A written report within 30 days to the U.S. Nuclear Regulatory Commission, Attn: Document Control Desk, Washington D.C. 20555, with a copy to the responsible NRC facility inspector of:
 1. Any substantial variance from performance specifications contained in these specifications or in the Safety Analysis Report;

2. Any significant change in the transient or accident analysis as described in the Safety Analysis Report;
 3. Any changes in facility organization; and
 4. Any observed inadequacies in the implementation of administrative or procedural controls.
- d. A written report within 60 days after completion of startup testing of the reactor to the U.S. Nuclear Regulatory Commission, Attn: Document Control Desk, Washington, D.C. 20555, with a copy to the responsible NRC facility inspector of: |
1. An evaluation of facility performance to date in comparison with design predictions and specifications; and
 2. A reassessment of the safety analysis submitted with the license application in light of measured operating characteristics when such measurements indicate that there may be substantial variance from prior analysis.
- e. A written annual report within 60 days following the 30th of June each year to the U.S. Nuclear Regulatory Commission, Attn: Document Control Desk, Washington, D.C. 20555, with a copy to the responsible NRC facility inspector of: |
1. A brief narrative summary of (1) operating experience (including experiments performed), (2) changes in facility design, performance characteristics, and operating procedures related to reactor safety and occurring during the reporting period, and (3) results of surveillance tests and inspections;
 2. Tabulation of the energy output (in megawatt days) of the reactor, amount of pulse operation, hours reactor was critical, and the cumulative total energy output since initial criticality;
 3. The number of emergency shutdowns and inadvertent scrams, including reasons therefore;
 4. Discussion of the major maintenance operations performed during the period, including the effect, if any, on the safety of the operation of the reactor, and the reasons for any corrective maintenance required;
 5. A brief description including a summary of the safety evaluations of changes in the facility or in procedures and of tests and experiments carried out pursuant to Section 50.59 of 10 CFR Part 50;

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION

SUPPORTING AMENDMENT NO. 18 TO

FACILITY LICENSE NO. R-52

THE UNIVERSITY OF ARIZONA

DOCKET NO. 50-113

1.0 INTRODUCTION

By letter dated March 7, 2001, the University of Arizona (the licensee) submitted a request for amendment to Facility License No. R-52 for the University of Arizona Research Reactor. The requested changes would amend administrative requirements in the technical specifications (TSs). Specifically, the licensee proposed changes to the reporting requirements to be consistent with the current NRC organization. The licensee also proposed changes to the Reactor Committee administrative responsibilities to be consistent with the recently revised 10 CFR 50.59 for changes, test and experiments.

2.0 EVALUATION

The proposed TS 6.2.b.3, updated the Reactor Committee responsibilities to be consistent with the revision of 10 CFR 50.59. The change referred to the applicable section of the revised 10 CFR 50.59 and deleted the now obsolete wording. Therefore, this change is acceptable.

The next change was to TS 6.7.a for the 24-hour reporting requirement. It deleted an obsolete reference to an NRC organization, and provides that the licensee will send reports to the responsible NRC organization. The staff finds this change will ensure that NRC organization changes will not require modification of the TSs. The change is also specific to ensure that the licensee will promptly notify responsible NRC personnel of conditions that require reporting within 24 hours. The licensee had originally proposed notifying only the responsible NRC facility inspector, but the licensee changed this to "the responsible NRC organization" based on telephone conversations with the NRC staff. This change to "responsible NRC organization" was to ensure that, in the absence of the inspector from the office, the licensee would inform other responsible persons in the organization. The licensee also changed TS 6.7.a to specify that they would list the responsible NRC organization in the Emergency Kit and would post this information as deemed necessary by the Reactor Committee. Based on a telephone conversation with the NRC staff, the licensee added the phrase "by the Reactor Committee." This was to ensure the responsibility for determining the necessary posting location is specified. Based on the above, this change is acceptable.

The remainder of the proposed changes concern written reporting requirements, TSs 6.7.b, c, d and e. The change deleted an obsolete reference to an NRC organization, and provides that the licensee will send written reports to the responsible NRC facility inspector. This does not change the requirement to send these written reports to the document control desk. This change is acceptable because the document control desk will ensure proper distribution and the inspector will be the focal point for evaluating such reports.

3.0 ENVIRONMENTAL CONSIDERATION

This amendment involves changes in recordkeeping, reporting, or administrative procedures or requirements. Accordingly, the amendment meets the eligibility criteria for categorical exclusion set forth in 10 CFR 51.22(c)(10). Pursuant to 10 CFR 51.22(b), no environmental impact statement or environmental assessment need be prepared in connection with the issuance of this amendment.

4.0 CONCLUSION

The staff has concluded, on the basis of the considerations discussed above, that (1) because the amendment does not involve a significant increase in the probability or consequences of accidents previously evaluated, or create the possibility of a new or different kind of accident from any accident previously evaluated, and does not involve a significant reduction in a margin of safety, the amendment does not involve a significant hazards consideration; (2) there is reasonable assurance that the health and safety of the public will not be endangered by the proposed activities; and (3) such activities will be conducted in compliance with the Commission's regulations and the issuance of this amendment will not be inimical to the common defense and security or the health and safety of the public.

Principal Contributor: Marvin M. Mendonca

Date: April 9, 2001