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U.S. Nuclear Regulatory Commission  
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Washington, DC 20555-0001  
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SUBJECT: **Indian Point 3 Nuclear Power Plant**  
**Docket No. 50-286**  
**Annual 10CFR50.46 Report for Year 2000**  
**Emergency Core Cooling System Evaluation Model Changes**

REFERENCE: Westinghouse letter (INT-00-243), S. Ira to R. Barrett, "10CFR50.46 BART/BASH Evaluation Model Mid-Year Notification and Reporting for 2000," dated January 31, 2001.

Dear Sir:

This letter summarizes changes to the Indian Point 3 emergency core cooling system (ECCS) evaluation model, and the effects of this change on the peak cladding temperature (PCT). It satisfies the 10CFR50.46(a)(3)(ii) annual reporting requirements for the year 2000.

Peak cladding temperature decreased by approximately 25°F as the result of a Westinghouse determination that the cladding surface emissivity values used in the current evaluation model were substantially lower than the values expected during a large break LOCA reflood transient and a correction to the vapor film flow regime heat transfer.

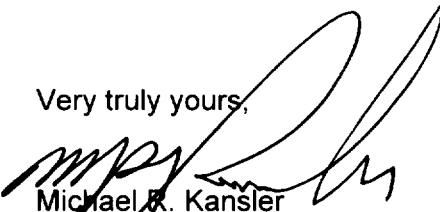
This report is based on the small and large break loss-of-coolant accident (LOCA) peak cladding temperature changes described in the referenced letter.

As defined by 10CFR50.46(a)(3)(i), this change is not significant. Indian Point 3 continues to comply with 10CFR50.46(a)(3)(ii) because the PCT remains below the maximum permissible fuel cladding temperature of 2200°F.

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There are no new commitments made by this letter. If you have any questions, please contact Ms. Charlene Faison at 914-272-3378.

Very truly yours,



Michael R. Kansler  
Senior Vice President and  
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cc: Regional Administrator  
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