

February 27, 2001

Mr. Guy G. Campbell, Vice President - Nuclear
FirstEnergy Nuclear Operating Company
5501 North State Route 2
Oak Harbor, OH 43449-9760

SUBJECT: DAVIS-BESSE NUCLEAR POWER STATION, UNIT 1 - ISSUANCE OF
AMENDMENT (TAC NO. MB0540)

Dear Mr. Campbell:

The U.S. Nuclear Regulatory Commission (NRC) has issued the enclosed Amendment No. 245 to Facility Operating License No. NPF-3 for the Davis-Besse Nuclear Power Station, Unit 1. The amendment is in response to your application dated November 9, 2000 (Serial Number 2675, Licensing Amendment Request No. 98-0017).

By letter dated November 9, 2000, the FirstEnergy Nuclear Operating Corporation (FENOC) requested a Technical Specification change for Davis-Besse Nuclear Power Station (DBNPS), Unit 1. The proposed Technical Specification (TS) change would relocate Technical Specification 3/4.4.9.2, Reactor Coolant System - Pressurizer, to the DBNPS Technical Requirements Manual (TRM). The TRM is a DBNPS controlled document which has been incorporated into the Davis-Besse Updated Safety Analysis Report (USAR).

The NRC staff has reviewed the FENOC submittal and supporting documentation and based on the review, the staff has found that the relocation of Technical Specification 3/4.4.9.2 to the DBNPS TRM is acceptable. A copy of the Safety Evaluation is also enclosed.

The Notice of Issuance will be included in the Commission's next biweekly Federal Register notice.

Sincerely,
/RA/

Stephen P. Sands, Project Manager, Section 2
Project Directorate III
Division of Licensing Project Management
Office of Nuclear Reactor Regulation

Docket No. 50-346

Enclosure: 1. Amendment No 245 to
License No. NPF-3
2. Safety Evaluation

DISTRIBUTION:

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PD3-2 r/f W. Beckner
G. Hill (2) G. Grant, RIII
ACRS

cc w/encl: See next page

OFFICE	PM:LPD3-2	LA:LPD3-2	RTSB	OGC	SC:LPD3-2
NAME	SSands <i>SPS</i>	THarris <i>TH</i>	WBeckner <i>WB</i>	<i>C. Marco</i>	<i>AMendiora</i>
DATE	02/8/01	02/7/01	02/9/01	02/21/01	02/26/01

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UNITED STATES
NUCLEAR REGULATORY COMMISSION

WASHINGTON, D.C. 20555-0001

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cc w/encl: See next page

Mr. Guy G. Campbell
FirstEnergy Nuclear Operating Company

cc:

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Davis-Besse Nuclear Power Station, Unit 1

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UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555-0001

FIRSTENERGY NUCLEAR OPERATING COMPANY

DOCKET NO. 50-346

DAVIS-BESSE NUCLEAR POWER STATION, UNIT 1

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 245
License No. NPF-3

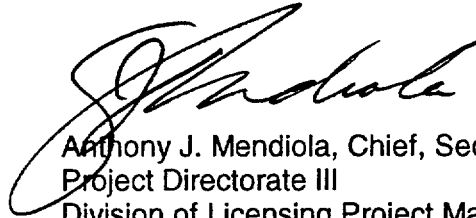
1. The U.S. Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment by the FirstEnergy Nuclear Operating Company (the licensee) dated November 9, 2000, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

(2) Technical Specifications

The Technical Specifications contained in Appendix A, as revised through Amendment No. 245, are hereby incorporated in the license. FirstEnergy Nuclear Operating Company shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of its date of issuance and shall be implemented not later than 120 days after issuance.

FOR THE NUCLEAR REGULATORY COMMISSION



Anthony J. Mendiola, Chief, Section 2
Project Directorate III
Division of Licensing Project Management
Office of Nuclear Reactor Regulation

Attachment: Changes to the Technical
Specifications

Date of Issuance: February 27, 2001

ATTACHMENT TO LICENSE AMENDMENT NO. 245

TO FACILITY OPERATING LICENSE NO. NPF-3

DOCKET NO. 50-346

Replace the following pages of the Appendix A Technical Specifications with the attached revised pages. The revised pages are identified by amendment number and contain marginal lines indicating the areas of change.

Remove

INDEX - pg. V
3/4 4-29

Insert

INDEX - pg. V
3/4 4-29

INDEX

LIMITING CONDITIONS FOR OPERATION AND SURVEILLANCE REQUIREMENTS

<u>SECTION</u>	<u>PAGE</u>
3/4.4.4 PRESSURIZER.	3/4 4-5
3/4.4.5 STEAM GENERATORS.	3/4 4-6
3/4.4.6 REACTOR COOLANT SYSTEM LEAKAGE	
Leakage Detection Systems.	3/4 4-13
Operational Leakage.	3/4 4-15
3/4.4.7 Deleted.	3/4 4-17
3/4.4.8 SPECIFIC ACTIVITY.	3/4 4-20
3/4.4.9 PRESSURE/TEMPERATURE LIMITS	
Reactor Coolant System.	3/4 4-24
<u>Deleted</u>	3/4 4-29
3/4.4.10 STRUCTURAL INTEGRITY.	3/4 4-30
3/4.4.11 Deleted.	3/4 4-32
3/4.5 <u>EMERGENCY CORE COOLING SYSTEMS (ECCS)</u>	
3/4.5.1 CORE FLOODING TANKS.	3/4 5-1
3/4.5.2 ECCS SUBSYSTEMS - $T_{avg} \geq 280^{\circ}\text{F}$	3/4 5-3
3/4.5.3 ECCS SUBSYSTEMS - $T_{avg} < 280^{\circ}\text{F}$	3/4 5-6
3/4.5.4 BORATED WATER STORAGE TANK.	3/4 5-7

DELETED



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555-0001

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION

RELATED TO AMENDMENT NO. 245 TO

FACILITY OPERATING LICENSE NO. NPF-3

DAVIS-BESSE NUCLEAR POWER STATION, UNIT 1

DOCKET NO. 50-346

1.0 INTRODUCTION

By letter dated November 9, 2000, FirstEnergy, the licensee, requested a change to the Davis-Besse Technical Specifications (TSs). The proposed change would relocate Technical Specification 3/4.4.9.2, Reactor Coolant System - Pressurizer, to the Davis-Besse Nuclear Power Station Technical Requirements Manual (TRM). The TRM is a Davis-Besse Nuclear Power Station controlled document which has been incorporated into the Davis-Besse Updated Safety Analysis Report (USAR). Any changes to the Davis-Besse USAR are controlled in accordance with 10 CFR Part 50.59.

2.0 BACKGROUND

The Commission's regulatory requirements related to the content of TSs are set forth in 10 CFR Part 50.36. This regulation requires that the TSs include items in five specific categories. These categories include 1) safety limits, limiting safety system settings and limiting control settings, 2) limiting conditions for operation, 3) surveillance requirements, 4) design features and, 5) administrative controls. However, the regulation does not specify the particular TSs to be included in a plant's license.

Additionally, 10 CFR 50.36(c)(2)(ii) sets forth four criteria to be used in determining whether a limiting condition for operation (LCO) is required to be included in the TS. These criteria are as follows:

1. Installed instrumentation that is used to detect, and indicate in the control room, a significant abnormal degradation of the reactor coolant pressure boundary.
2. A process variable, design feature, or operating restriction that is an initial condition of a design basis accident or transient analysis that either assumes the failure of or presents a challenge to the integrity of a fission product barrier.
3. A structure, system, or component that is part of the primary success path and which functions or actuates to mitigate a design-basis accident or transient that either assumes the failure of or presents a challenge to the integrity of a fission product barrier.

4. A structure, system or component which operating experience or probabilistic risk assessment has shown to be significant to public health and safety.

Existing LCOs and related surveillances included as TS requirements which satisfy any of the criteria stated above must be retained in the TSs. Those TS requirements which do not satisfy these criteria may be relocated to other licensee-controlled documents.

3.0 EVALUATION

The existing TS 3/4.4.9.2 conditions, actions, and surveillance requirements for the pressurizer temperature limits will be relocated to the TRM. These requirements define the limitations on the pressurizer heatup and cooldown, spray water temperature differential, and minimum temperature when pressure is greater than 625 psig to assure that the pressurizer remains within the design criteria assumed for the pressurizer fatigue analysis. As discussed in Section 5.5.10 of the Davis-Besse USAR, the total stresses resulting from thermal expansion, pressure, and mechanical and seismic loadings are considered in the design of the pressurizer. The total stresses expected in the pressurizer are within the maximum allowed by the American Society of Mechanical Engineers (ASME) Boiler and Pressure Vessel (B&PV) Code, Section III. The staff finds that the licensee's safety analysis adequately shows that the proposed TS change meets the applicable sections of the ASME B&PV code, Section III, and is therefore acceptable.

The staff evaluated the existing TS against the four criteria set forth in 10 CFR 50.36(c)(2)(ii). Pressurizer temperature limits are not a form of instrumentation, nor a structure, system or component, and therefore, do not meet criteria 1, 3, or 4. The pressurizer temperature limits are process variables. However, these process variables are consistent with boundaries assumed in the structural analyses of the pressurizer and not as an initial condition for a design-basis accident or transient analysis. Therefore, pressurizer temperature limits do not meet criteria 2 for inclusion in the TSs. Since TS 3/4.4.9.2 requirements do not satisfy these criteria, TS 3/4.4.9.2 may be relocated to other licensee-controlled documents.

The licensee has incorporated the TRM into the Davis-Besse USAR. As such, changes to the TRM would be controlled in accordance with approved station procedures and the requirements of 10 CFR Part 50.59. Therefore, the staff believes that sufficient regulatory controls exist and concludes that TS 3/4.4.9.2 may be relocated from the TSs to the licensee's TRM.

The staff notes that TS 3/4.4.4, Reactor Coolant System - Pressurizer, which requires the pressurizer to be operable, is retained in the Davis-Besse TSs and is not affected by the proposed change.

4.0 CONCLUSION

The staff has reviewed the licensee's submittal and supporting documentation. The staff finds the proposed relocation of TS 3/4.4.9.2 to the Davis-Besse TRM to be acceptable. Additionally, the staff concludes that there is reasonable assurance that plant operation in this manner poses no undue risk to the health and safety of the public.

5.0 STATE CONSULTATION

In accordance with the Commission's regulations, the Ohio State official was notified of the proposed issuance of the amendments. The State official had no comments.

6.0 ENVIRONMENTAL CONSIDERATION

The amendment changes a requirement with respect to installation or use of a facility component located within the restricted area as defined in 10 CFR Part 20, or changes a surveillance requirement. The NRC staff has determined that the amendment involves no significant increase in the amounts, and no significant change in the types, of any effluents that may be released offsite, and that there is no significant increase in individual or cumulative occupational radiation exposure. The Commission has previously issued a proposed finding that the amendment involves no significant hazards consideration, and there has been no public comment on such finding (65 FR 81919, dated December 27, 2000). Accordingly, the amendments meet the eligibility criteria for categorical exclusion set forth in 10 CFR 51.22(c)(9). Pursuant to 10 CFR 51.22(b), no environmental impact statement or environmental assessment need be prepared in connection with the issuance of the amendments.

7.0 CONCLUSION

The Commission has concluded, based on the considerations discussed above, that (1) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, (2) such activities will be conducted in compliance with the Commission's regulations, and (3) the issuance of the amendments will not be inimical to the common defense and security or to the health and safety of the public.

Principle Contributor: K.Kavanagh

Date: February 27, 2001