

February 21, 2001

EA-00-262

Mr. Oliver D. Kingsley
President, Nuclear Generation Group
Commonwealth Edison Company
ATTN: Regulatory Services
Executive Towers West III
1400 Opus Place, Suite 500
Downers Grove, IL 60515

SUBJECT: FINAL SIGNIFICANCE DETERMINATION FOR A WHITE FINDING - NRC
INSPECTION REPORT 50-254/00-18(DRS); 50-265/00-18(DRS) - QUAD
CITIES NUCLEAR POWER STATION

Dear Mr. Kingsley:

The purpose of this letter is to provide you with the final results of our significance determination of the preliminary White finding identified in the subject inspection report. The inspection finding was assessed using the Significance Determination Process and was preliminarily characterized as White (i.e., an issue with low to moderate increased importance to safety, which may require additional NRC inspections). This White finding involved the failure to maintain radiation doses as-low-as-is-reasonably-achievable (ALARA) during the recently completed Unit 1 refueling outage (Q1R16).

At your request, a Regulatory Conference was held on February 13, 2001, to further discuss your views on this issue. A copy of the handout your staff provided at the conference is enclosed. During the meeting your staff described your assessment of the significance of the findings, and detailed corrective actions including the root cause evaluations of the radiological issues that resulted in the implementation of the Significance Determination Process. Specifically, your staff agreed with the facts presented in the subject inspection report, characterized their assessment of the issues and described those corrective actions planned for identified deficiencies. Your staff also discussed concerns they have with the NRC's ALARA Significance Determination Process.

After considering the information developed during the inspection and the information you provided at the conference, the NRC has concluded that the inspection finding is appropriately characterized as White (i.e., an issue with low to moderate increased importance to safety, which may require additional NRC inspections).

You have 10 business days from the date of this letter to appeal the staff's determination of significance for the identified White finding. Such appeals will be considered to have merit only if they meet the criteria given in NRC Inspection Manual Chapter 0609, Supplement 3.

Because plant performance for this issue has been determined to be in the regulatory response band, we will use the NRC Action Matrix to determine the most appropriate NRC response for this event. We will notify you, by separate correspondence, of that determination.

In accordance with 10 CFR 2.790 of the NRC's "Rules of Practice," a copy of this letter and its enclosure will be available electronically for public inspection in the NRC Public Document Room or from the Publicly Available Records (PARS) component of NRC's document system (ADAMS). ADAMS is accessible from the NRC Web site at <http://www.nrc.gov/NRC/ADAMS/index.html> (the Public Electronic Reading Room).

Sincerely,

/ RA /

John A. Grobe, Director
Division of Reactor Safety

Docket Nos. 50-254; 50-265
License Nos. DPR-29; DPR-30

Enclosure: Handout - NRC Regulatory Conference ALARA Finding Safety Relief Valve Replacement

cc w/o encl: D. Helwig, Senior Vice President, Nuclear Services
C. Crane, Senior Vice President, Nuclear Operations
H. Stanley, Vice President, Nuclear Operations
R. Krich, Vice President, Regulatory Services
DCD - Licensing
T. J. Tulon, Site Vice President
G. Barnes, Quad Cities Station Manager
W. Beck, Regulatory Affairs Manager
W. Leech, Manager of Nuclear
MidAmerican Energy Company

cc w/encl: M. Aguilar, Assistant Attorney General
State Liaison Officer, State of Illinois
State Liaison Officer, State of Iowa
Chairman, Illinois Commerce Commission

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cc w/encl: M. Aguilar, Assistant Attorney General
State Liaison Officer, State of Illinois
State Liaison Officer, State of Iowa
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