



Crystal River Nuclear Plant  
Docket No. 50-302  
Operating License No. DPR-72

Ref: 10 CFR 50.90

February 15, 2001  
3F0201-03

Document Control Desk  
U. S. Nuclear Regulatory Commission  
Washington, DC 20555-0001

Subject: Response to NRC Request for Clarifications Related to License Amendment Request (LAR) No. 261

Reference: FPC to NRC letter, 3F0800-13, dated August 31, 2000, License Amendment Request No. 261, Revision 0, "Remote Shutdown System Instrumentation"

Dear Sir:

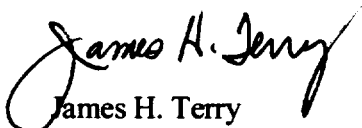
In the above referenced letter, Florida Power Corporation (FPC) requested changes to Improved Technical Specifications (ITS) 3.3.18, Remote Shutdown System Instrumentation. As part of this request, FPC proposed deleting Nuclear Services Closed Cycle Cooling System (SW) instrumentation from ITS Table 3.3.18-1.

During a recent telephone conversation with the NRC Project Manager, FPC offered clarifications related to the justification provided in LAR No. 261 for deleting the SW instrumentation from ITS Table 3.3.18-1. The information provided during that conversation is contained in the attachment to this letter.

This submittal contains no new regulatory commitments.

If you have any questions regarding this submittal, please contact Mr. Sid Powell, Supervisor, Licensing and Regulatory Programs at (352) 563-4883.

Sincerely,

  
James H. Terry  
Manager Engineering

JHT/jal

Attachment: Response to NRC Request for Clarifying Information

xc: Regional Administrator, Region II  
Senior Resident Inspector  
NRR Project Manager

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## **Response to NRC Request for Clarifying Information**

The following information is intended to clarify the justification provided in License Amendment Request No. 261 for deleting Nuclear Services Closed Cycle Cooling System (SW) instrumentation from ITS Table 3.3.18.

The specific actions in AP-990, "Shutdown from Outside the Control Room," related to the SW system are: 1) to ensure an emergency duty SW pump is running, 2) to ensure SW is aligned to the Reactor Building fans, and 3) to ensure that SW is aligned to makeup pump MUP-1C, if required. The guidance provided in AP-990 to perform these actions is limited to directing component manipulations and/or verifying component positions/status. No guidance is provided related to monitoring parameters such as flow, pressure or temperature to verify successful completion of these actions or proper system operation.

During normal operation, SW flow is supplied by the normal duty non-safeguards SW pump, which has a nominal flow rate of 6900 gpm (FSAR Chapter 9, Table 9-12). Starting an emergency duty SW pump will increase SW flow to 8400 gpm (FSAR Chapter 9, Table 9-12) with a proportional increase in system heat removal capacity. In addition to the increase in SW heat removal capacity resulting from starting an emergency duty SW pump, actions in AP-990 that secure equipment, such as the reactor coolant pumps, will reduce the SW system heat loads. There are no actions (other than starting the pump) required to supply the SW system from an emergency duty SW pump, and there are no actions in AP-990 that would impact the ability of the emergency SW pump to supply the SW system. These factors of increased SW flow, reduced SW system heat loads, and the physical configuration/operation of the SW system assure proper system operation and adequate heat removal capability.

The Bases change that deletes the criteria related to monitoring support systems is only intended to ensure consistency between information provided in the Bases and the instruments listed in the revised specification table (none of which monitor support systems). Post fire actions to monitor support system performance will be accomplished using local instrumentation and indications that will remain available following the fire.