PSEG Nuclear LLC
P.O. Box 236, Hancocks Bridge, New Jersey 08038-0236

FEB 15 2001

LRN-01-0063

PSEG Nuclear LLC

U. S. Nuclear Regulatory Commission
Document Control Desk
Washington, DC 20555

Gentlemen:

MONTHLY OPERATING REPORT
HOPE CREEK GENERATING STATION UNIT 1
DOCKET NO. 50-354

In compliance with Section 6.9, Reporting Requirements for the Hope Creek Technical Specifications, the operating statistics for **January 2001** are being forwarded. Also being forwarded, pursuant to the requirements of 10CFR50.59(b), is a summary of changes, tests, and experiments that were implemented in **January 2001.**

Sincerely,

D. F. Garchow
Vice President - Operations

RAR
Attachments

C Distribution

IE24

INDEX

DOCKET NO.: 50-354
UNIT: Hope Creek
DATE: 02/09/01
COMPLETED BY: F. Todd
TELEPHONE: (856) 339-1316

Reporting Period **January 2001**

OPERATING DATA REPORT

Design Electrical Rating (MWe-Net)	**1067**		
Maximum Dependable Capacity (MWe-Net)	**1031**		
	Month	**Year-to-date**	**Cumulative**
No. of hours reactor was critical	**744**	**744**	**105191**
No. of hours generator was on line (service hours)	**744**	**744**	**103150**
Unit reserve shutdown hours	**0**	**0**	**0**
Net Electrical Energy (MWH)	**788170**	**788170**	**104358505**

UNIT SHUTDOWNS

NO.	DATE	TYPE F=FORCED S=SCHEDULED	DURATION (HOURS)	REASON (1)	METHOD OF SHUTTING DOWN THE REACTOR (2)	CORRECTIVE ACTION/ COMMENT

(1) Reason

A - Equipment Failure (Explain)
B - Maintenance or Test
C - Refueling
D - Regulatory Restriction
E - Operator Training/License Examination
F - Administrative
G - Operational Error (Explain)
H - Other

(2) Method

1 - Manual
2 - Manual Trip/Scram
3 - Automatic Trip/Scram
4 - Continuation
5 - Other (Explain)

DOCKET NO.: 50-354
UNIT: Hope Creek
DATE: 02/09/01
COMPLETED BY: R. Ritzman
TELEPHONE: (856) 339-1445

Summary Of Monthly Operating Experience

- Hope Creek started the month at approximately 100% reactor power.
- On January 6, a power reduction to approximately 58% for scheduled Deep/Shallow Control Rod Swap was initiated. The unit was returned to 100% power on January 7.
- At the end of January, Hope Creek completed 189 days of continuous on-line operation.

DOCKET NO.:50-354
UNIT: Hope Creek
DATE:01/08/01
COMPLETED BY:R. Ritzman
TELEPHONE: (856) 339-1445

SUMMARY OF CHANGES, TESTS, AND EXPERIMENTS FOR THE HOPE CREEK GENERATING STATION

MONTH January 2001

The following item completed during **January 2001** has been evaluated to determine:

1. If the probability of occurrence or the consequences of an accident or malfunction of equipment important to safety previously evaluated in the safety analysis report may be increased; or

2. If a possibility for an accident or malfunction of a different type than any evaluated previously in the safety analysis report may be created; or

3. If the margin of safety as defined in the basis for any technical specification is reduced.

The 10CFR50.59 Safety Evaluation showed that this item did not create a new safety hazard to the plant nor did it affect the safe shutdown of the reactor. This item did not change the plant effluent releases and did not alter the existing environmental impact. The 10CFR50.59 Safety Evaluation determined that no unreviewed safety or environmental questions are involved.

Design Changes Summary of Safety Evaluation

There were no reportable changes in this category implemented during January 2001.

Temporary Modifications Summary of Safety Evaluations

There were no reportable changes in this category implemented during January 2001.

Procedures Summary of Safety Evaluations

HC.IC-EU.KJ-0001(Q), Revision 0, Astro-Med Recorder/Equipment Setup for Emergency Diesel Generator Related Surveillance Testing. This procedure allows one train of the Emergency Diesel Generator and its connected equipment to be considered to be operable during its respective monthly and 24-hour endurance surveillance test while temporarily connected to electrical test equipment. This temporary configuration consists of opening the panel door, connecting test equipment to the potential transformer circuit, and connecting test equipment to a contact from an auxiliary start relay.

This new procedure does not introduce any new or different failure mechanisms and does not increase the probability or consequences of any previously postulated failure mechanisms. A seismic evaluation demonstrated that the open panel door and the additional test equipment do not affect the seismic qualifications of the Emergency Diesel Generator or its associated equipment. Therefore, this procedure does not increase the possibility or consequences of any accident or malfunction, does not reduce the margin of safety, and does not involve an Unreviewed Safety Question.

UFSAR Change Notices Summary of Safety Evaluations

HCN 00-062, Loss of Coolant Accident Control Room Dose Model. This UFSAR change notice incorporates the results of an updated analysis of the radiological consequences of a loss of coolant accident. The change notice also revises the descriptions of the computer codes used to generate the dose consequence information.

This UFSAR change notice consists of changes made to reflect Filtration, Recirculation, and Ventilation System changes previously approved by the NRC and changes that are administrative in nature. Therefore, this UFSAR change notice does not increase the possibility or consequences of any accident or malfunction, does not reduce the margin of safety, and does not involve an Unreviewed Safety Question.