

May 22, 1989

Docket No. 50-244

Dr. Robert C. Mecredy, General Manager
Nuclear Production
Rochester Gas & Electric Corporation
89 East Avenue
Rochester, New York 14649-0001

Dear Dr. Mecredy:

SUBJECT: ISSUANCE OF AMENDMENT TO FACILITY OPERATING LICENSE NO. DPR-18

The Commission has issued the enclosed Amendment No.36 to Facility Operating License No. DPR-18 for the R. E. Ginna Nuclear Power Plant. This amendment is in response to your application dated February 24, 1989.

The amendment revises the rod insertion limits for the Cycle 19 fuel reload to ensure that all criteria for the reload are met. Since the change is not applicable to future cycles, it is presented as a change with a limited period of applicability.

A copy of the related Safety Evaluation is also enclosed. A Notice of Issuance will be included in the Commission's biweekly Federal Register notice.

Sincerely,

15/

Allen Johnson, Project Manager
Project Directorate I-3
Division of Reactor Projects I/II

Enclosures:

1. Amendment No.36 to:
License No. DPR-18
2. Safety Evaluation

cc w/ enclosures:
See next page

c/p-1
ent

DFOL
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OFC: PDI-3 :PDI-3 :SRXB :OGC :PDI-3 :
NAME:MRushbrook:AJohnson :MWHodges: :RWessman: :
DATE:4/5/89 :4/24/89 :4/25/89 :4/26/89 :4/28/89 : :



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

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Allen Johnson, Project Manager
Project Directorate I-3
Division of Reactor Projects I/II

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See next page

Amendment No. 36 to Facility Operating Licensee DPR-18 - R. E. Ginna
Nuclear Power Plant

DISTRIBUTION:

Docket File 50-244 ←

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Rochester Gas and Electric Corporation

R. E. Ginna Nuclear Power Plant

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UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

ROCHESTER GAS AND ELECTRIC CORPORATION
DOCKET NO. 50-244
R. E. GINNA NUCLEAR POWER PLANT
AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 36
License No. DPR-18

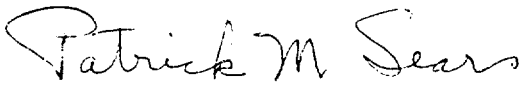
1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment by Rochester Gas and Electric Corporation (the licensee) dated February 24, 1989 complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act) and the Commission's rules and regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations; and
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.
2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment and Paragraph 2.C.(2) of Facility Operating License No. DPR-18 is hereby amended to read as follows:

(2) Technical Specifications

The Technical Specifications contained in Appendix A, as revised through Amendment No.36, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective on the date of issuance.

FOR THE NUCLEAR REGULATORY COMMISSION

for 
Richard H. Wessman, Director
Project Directorate I-3
Division of Reactor Projects I/II

Attachment:
Changes to the Technical
Specifications

Date of Issuance: May 22, 1989

ATTACHMENT TO LICENSE AMENDMENT NO. 36

FACILITY OPERATING LICENSE NO. DPR-18

DOCKET NO. 50-244

Revise Appendix A Technical Specifications by removing the pages identified below and inserting the enclosed pages. The revised pages are identified by the captioned amendment number and contain marginal lines indicating the area of change.

REMOVE

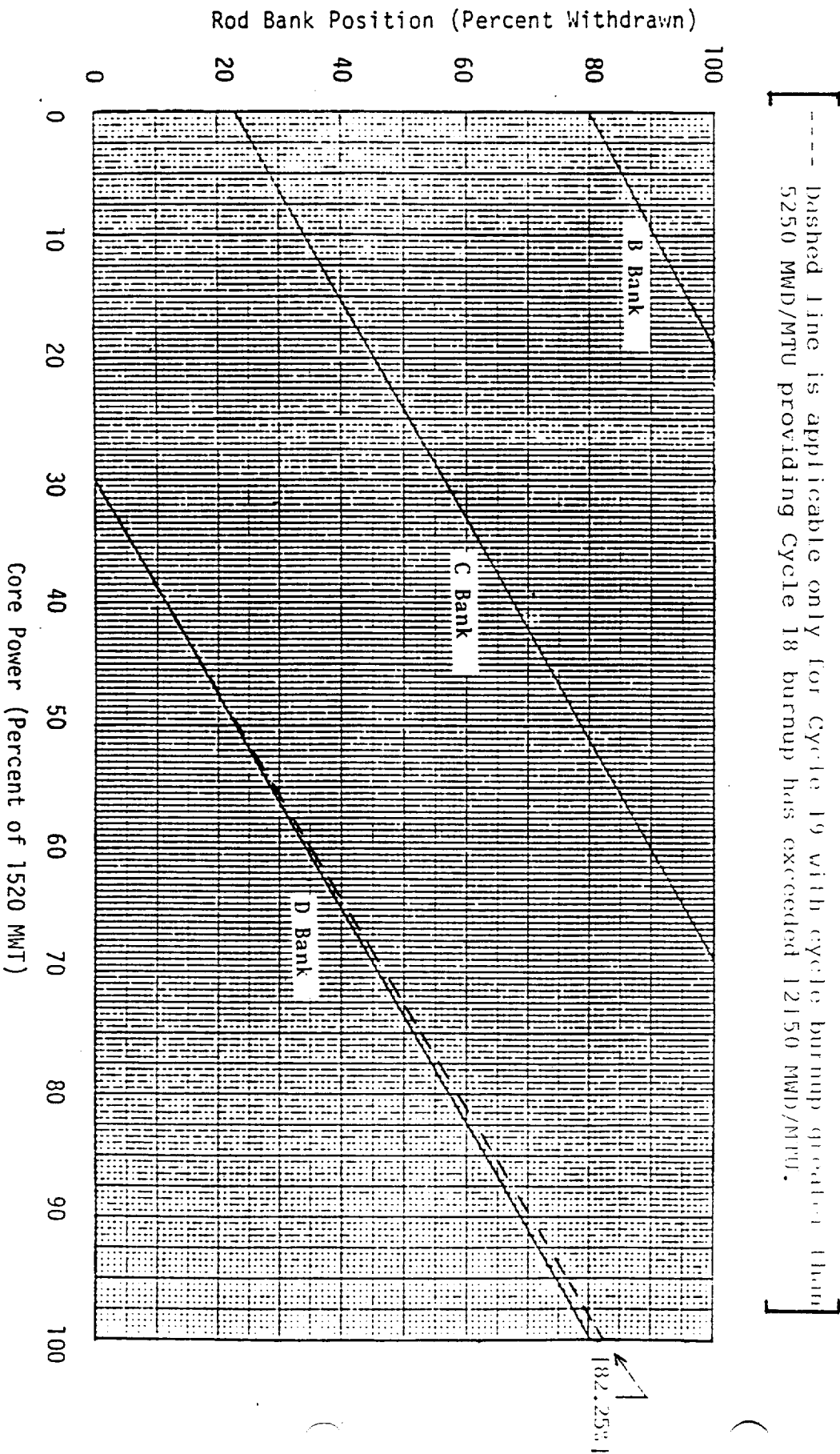
3.10-20

INSERT

3.10-20

3.10-20

FIGURE 3.10-1
CONTROL ROD INSERTION LIMITS VERSUS CORE POWER
FOR BOL THROUGH EOL





UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION
SUPPORTING AMENDMENT NO. 36 TO FACILITY OPERATING LICENSE NO. DPR-18

ROCHESTER GAS AND ELECTRIC CORPORATION

R. E. GINNA NUCLEAR POWER PLANT

DOCKET NO. 50-244

1.0 INTRODUCTION

By letter dated February 24, 1989, the licensee, Rochester Gas & Electric Corporation, proposed changes to the Ginna Technical Specifications which would modify the rod insertion limits for Cycle 19 fuel reload to ensure that all criteria for the reload are met. Since the change is not applicable to future cycles, it is presented as a change with a limited period of applicability.

2.0 EVALUATION

The Cycle 19 reload has been designed by Westinghouse using standard design methodology. Based on this methodology, all criteria for the reload are met provided a slight adjustment is made to the bank D insertion limits. This Technical Specification change will slightly modify the rod insertion limits for Cycle 19 to ensure all criteria are met.

The adjustment is only necessary for Cycle 19, since Cycle 18 exceeded a burnup of 12150 MWD/MTU, and is only applicable for Cycle 19 burnups from 5250 to 9500 MWD/MTU. (Since 9500 MWD/MTU is close to Cycle 19 end of cycle, the 9500 burnup has been extended to end of cycle for this application).

The amount by which the 1.66 FW limit could be exceeded with the current insertion limits varies from 0% at a Cycle 18 burnup of 12150 MWD/MTU to a maximum of 0.33% at a Cycle 18 burnup of 12300 MWD/MTU. The limit is not exceeded if the bank D insertion limit is raised by 2.25% (5 steps) at full power. Since this change is not applicable to future cycles, it is a change with a limited period of applicability with appropriate qualifiers. No other safety criteria are affected by the Cycle 19 core reload.

The proposed change would require the bank D control rods to be withdrawn 5 steps farther during full power operation. Withdrawing the rods above the insertion limit increases the shutdown margin, decreases the ejected rod worth, reduces power peaking, and does not alter stuck rod worth. Therefore, withdrawing the rods is conservative and ensures that the safety criteria for the Cycle 19 reload are met, there is no significant increase in the probability or consequences of an accident previously evaluated. Thus the staff finds the proposed change acceptable for Cycle 19.

3.0 ENVIRONMENTAL EVALUATION

This amendment involves a change in the installation or use of a facility component located within the restricted areas as defined in 10 CFR Part 20. The staff has determined that the amendment involves no significant increase in the amounts, and no significant change in the types, of any effluents that may be released offsite, and that there is no significant increase in individual or cumulative occupational radiation exposure. The Commission has previously published in the Federal Register a proposed finding that this amendment involves no significant hazards consideration, (54FR15837) April 19, 1988. There has been no public comment on such finding. Accordingly, this amendment meets the eligibility criteria for categorical exclusion set forth in 10 CFR 51.22(c)(9). Pursuant to 10 CFR 51.22(b), no environmental impact statement or environmental assessment need be prepared in connection with the issuance of this amendment.

4.0 CONCLUSION

We have concluded, based on the consideration discussed above, that there is a reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, and such activities will be conducted in compliance with the Commission's regulations. The issuance of the amendment will not be inimical to the common defense and security or to the health and safety of the public.

Principal Contributor: P. Sears

Dated: May 22, 1989