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ROBERT C. MECREDY
Vice President
Nuclear Operations

February 13, 2001

U.S. Nuclear Regulatory Commission
Document Control Desk
Attn: Guy S. Vissing
Project Directorate I
Washington, D.C. 20555

Subject: 10CFR50.46 30 Day and Annual ECCS Report
R.E. Ginna Nuclear Power Plant
Docket No. 50-244

Ref. (a) Westinghouse Letter RGE-01-003, Subject: Notification of PCT Impact for R.E. Ginna related to 100 psig IFBA RIP Modeling Error and a Channel Splitting Error in SECY UPIEM Analysis, dated January 15, 2001

(b) Westinghouse Letter CAB-00-326, Subject: Ginna Cycle 29 Revision 1 Final RSE, dated October 5, 2000

(c) Westinghouse Letter RGE-00-212, Subject: 10CFR50.46 Appendix K(BART/BASH/NO TRUMP) Evaluation Model Mid-Year Notification and Report for 2000, dated June 30, 2000

Dear Mr. Vissing:

In accordance with the requirements in 10CFR50.46 paragraph (a)(3)(ii), this 30 day and annual ECCS report is hereby submitted.

Westinghouse, the provider of LOCA analysis for the R. E. Ginna Nuclear Power Plant, has provided RG&E with information regarding errors in the LOCA analysis that require 30 day NRC notification (Reference (a)). In addition, References (b) and (c) provide an update to the peak cladding temperature (PCT) margin.

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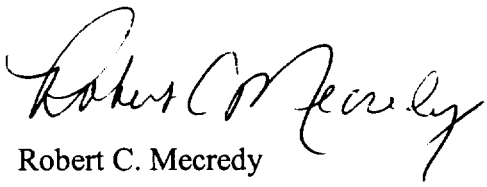
The large break LOCA PCT has increased 56°F from the value previously reported due to an error in modeling the rod internal pressure (RIP) in integral fuel burnable absorber (IFBA) fuel rods, an error in defining the momentum change between vertical channels (Channel Splitting Error), and the use of reconstituted fuel assemblies. The new large break LOCA PCT is 2153°F and is summarized in Attachment 1 to this letter. The 56°F PCT penalty is a result of Ginna specific computer runs. The computer runs utilized the latest code version and corrects previously identified errors. Therefore, the current PCT is the result of direct plant specific sensitivity reanalysis. No schedule for reanalysis is proposed because Ginna specific analysis has been completed in determining the PCT penalty.

The small break LOCA PCT has increased by 13°F due to correction of mixture level hangup problems (Mixture Level Tracking/Region Depletion Errors) (Reference (c)). The new small break LOCA PCT is 1346°F and is summarized in Attachment 1 to this letter.

Very truly yours,



Robert C. Mecredy

Attachment

cc: Mr. Guy S. Vissing (Mail Stop 8C2)
Project Directorate I
Division of Licensing Project Management
Office of Nuclear Reactor Regulation
U.S. Nuclear Regulatory Commission
Washington, D.C. 20555

Regional Administrator, Region I
U.S. Nuclear Regulatory Commission
475 Allendale Road
King of Prussia, PA 19406

U.S. NRC Ginna Senior Resident Inspector

ATTACHMENT I

LOCA PCT SUMMARY

FEBRUARY 2001 UPDATE

ATTACHMENT I

LOCA PT SUMMARY

Large Break LOCA
R.E. Ginna Nuclear Power Plant
Rochester Gas and Electric Corporation

Evaluation Model: UPI SECY Fuel: OFA
$F_Q = 2.45$ $F_{\Delta H} = 1.75$ SGTP = 15%

A.	Analysis of Record (5/95) (effective 6/96)	PCT = 2051° F
B.	1995 10CFR50.46 Model Assessments	
	1. Fixed heat transfer node assignment Error/Accumulator water injection error	ΔPCT = 48°F
C.	1996 10CFR50.46 Model Assessments	
	1. None	ΔPCT = 0°F
D.	1997 10CFR50.46 Model Assessment	
	1. Accumulator Initial Water Volume Restart Data Transportation Error Plant Specific Analytical Reassessment of 1995 Model Assessments	ΔPCT = 58°F
	2. Accumulator Initial Water Vol. = 1125 ft^3	ΔPCT = -25°F
	3. 1-D Transition Boiling Heat Transfer Error	ΔPCT = -13°F
	4. Vessel Channel DX Error	ΔPCT = 18°F
	5. Input Consistency	ΔPCT = -41°F
E.	1998 10CFR50.46 Model Assessments	
	1. None	ΔPCT = 0°F
F.	1999 10CFR50.46 Model Assessments	
	1. None	ΔPCT = 0°F
G.	2000 10CFR50.46 Model Assessments	
	1. 100 psig IFBA RIP Modeling Error	ΔPCT = 2°F
	2. Channel Splitting Error	ΔPCT = 52°F
H.	10CFR50.59 Evaluations	
	1. Service Water Temp. ≥ 30° (1997 evaluation; SEV-1090)	ΔPCT = 1°F
I.	Other Margin Allocations	
	1. Cycle 29 Fuel Reconstitution	ΔPCT = 2°F
	Licensing Basis	PCT = 2153°F

Revision Date: 2/2001

ATTACHMENT I

LOCA PT SUMMARY

Small Break LOCA
R.E. Ginna Nuclear Power Plant
Rochester Gas and Electric Corporation

Evaluation Model: NOTRUMP Fuel: OFA
$F_Q = 2.50$ $F_{\Delta H} = 1.75$ SGTP = 15%

A.	Analysis of Record (6/95) (effective 6/96)	Δ PCT = 1308°F
B.	1995 10CFR50.46 Model Assessments	
	1. Notrump Specific Enthalpy Error	Δ PCT = 20°F
	2. SALIBRARY Double Precision Errors	ΔPCT = -15°F
C.	1996 10CFR50.46 Model Assignments	
	1. SBLOCA Fuel Rod Initialization Error	ΔPCT = 10°F
D.	1997 10CFR50.46 Model Assessment	
	1. None	ΔPCT = 0°F
E.	1998 10CFR50.46 Model Assessments	
	1. None	ΔPCT = 0°F
F.	1999 10CFR50.46 Model Assessments	
	1. None	ΔPCT = 0°F
G.	2000 10CFR50.46 Model Assessments	
	1. NOTRUMP - Mixture Level Tracking/ Region Depletion Errors	ΔPCT = 13°F
H.	10CFR50.59 Evaluations	
	1. Annular Axial Pellets (1997 evaluation; SEV-1108)	ΔPCT = 10°F
I.	Other Margin Allocations	
	1. None	ΔPCT = 0°F
	Licensing Basis	PCT = 1346°F

Revision Date: 2/2001