



UNITED STATES
NUCLEAR REGULATORY COMMISSION

WASHINGTON, D.C. 20555-0001

February 20, 2001

Mr. W. R. McCollum, Jr.
Vice President, Oconee Site
Duke Energy Corporation
7800 Rochester Highway
Seneca, SC 29672

SUBJECT: REVISION TO TECHNICAL SPECIFICATION BASES B 3.6.1 RE: OCONEE
NUCLEAR STATION, UNITS 1, 2, AND 3

Dear Mr. McCollum:

By letter dated January 31, 2001, you informed the staff of a change to the Oconee Nuclear Station, Units 1, 2, and 3 Technical Specifications (TS) that only affects the Bases of TS 3.6.1, Containment. The change revises the Bases to add "UFSAR Chapter 18, Table 18-1" to the list of references.

The purpose of this letter is to distribute the enclosed TS page and List of Effective Pages to the appropriate TS manual holders.

Sincerely,

A handwritten signature in black ink, appearing to read "De LaBarge", is positioned above the typed name.

David E. LaBarge, Senior Project Manager, Section 1
Project Directorate II
Division of Licensing Project Management
Office of Nuclear Reactor Regulation

Docket Nos. 50-269, 50-270, and 50-287

Enclosure: Revised Bases Page and List of Effective Pages

cc w/encl: See next page

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cc:

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OCONEE NUCLEAR STATION
TECHNICAL SPECIFICATIONS - BASES
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BASES

SURVEILLANCE
REQUIREMENTS
(continued)

SR 3.6.1.2

Maintaining the containment OPERABLE requires compliance with the Type B and C leakage rate test requirements of 10 CFR 50, Appendix J, Option A (Ref. 1), as modified by approved exemptions. As left leakage prior to the first startup after performing a required 10 CFR 50, Appendix J, Option A, leakage test is required to be $< 0.6 L_a$ for combined Type B and C leakage. At all other times between required leakage rate tests, the acceptance criteria is based on an overall Type A leakage limit of $\leq 1.0 L_a$. At $\leq 1.0 L_a$ the offsite dose consequences are bounded by the assumptions of the safety analysis. SR Frequencies are as required by Appendix J, Option A, as modified by approved exemptions. Thus, SR 3.0.2 (which allows Frequency extensions) does not apply. These periodic testing requirements verify that the containment leakage rate does not exceed the leakage rate assumed in the safety analysis.

SR 3.6.1.3

This SR ensures that the structural integrity of the containment will be maintained in accordance with the provisions of the Containment Tendon Surveillance Program. Testing and Frequency are as described in Specification 5.5.7, "Pre-stressed Concrete Containment Tendon Surveillance Program."

REFERENCES

1. 10 CFR 50, Appendix J, Option A and B.
 2. UFSAR, Sections 15.13 and 15.14.
 3. UFSAR, Section 6.2.
 4. 10 CFR 50.36.
 5. UFSAR Chapter 18, Table 18-1.
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