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T. A. Sullivan
Vice President, Operations-JAF

January 26, 2001
JAFP-01-0021

United States Nuclear Regulatory Commission
ATTN: Document Control Desk
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Washington, DC 20555

SUBJECT: James A. FitzPatrick Nuclear Power Plant
Docket No. 50-333
In - Vessel Visual Inspection Summary Report
2000 Refuel Outage (Reload 14/Cycle 15)

Dear Sir:

This letter is provided for information and submits the In - Vessel Visual Inspection (IVVI) Summary Report for Refueling Outage 14 for the James A. Fitzpatrick Nuclear Power Plant (Commercial Service Date: July 28, 1975). The IVVI examinations were conducted inside the RPV by General Electric Nuclear Energy (GENE) utilizing personnel certified and qualified to ASME XI and the guidelines as outlined in BWRVIP - 03, Revision 1, Section 2, "General Procedures," Subsection 2.5, "Generic Standards for Visual Inspection of Reactor Pressure Vessel Internals, Components, and Associated Repairs," Sub-Subsection 4.0 "Personnel Training/Experience." Per requirements of these documents, specific training was conducted for all inspection personnel addressing plant specific procedural requirements, core shroud and core spray configuration details, previous inspection results, inspection procedures, and any other pertinent information related to the IVVI inspections. Attachment 1 provides a summary report of the IVVI examinations conducted during the 2000 refuel outage (Reload 14/Cycle 15) and their results. All relevant indications recorded during the IVVI examination were satisfactorily dispositioned in accordance with the internal corrective action tracking system, and were found acceptable for continued operation.

If you have any questions, please contact Mr. Arthur Zaremba at (315) 349-6005.

Very truly yours,

A handwritten signature in black ink, appearing to read "T. A. Sullivan".

T. A. Sullivan
Vice President, Operations - JAF

cc: See next page
Attachment as stated

A047

cc: Regional Administrator
U.S. Nuclear Regulatory Commission
475 Allendale Road
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ATTACHMENT 1

IVVI SUMMARY REPORT (RO-14)

JAMES A. FITZPATRICK NUCLEAR POWER PLANT (JAF)

Components in BWRVIP Scope	Date or Frequency of Inspection	Inspection Method Used	Summarize the Following Information: Inspection Results, Repairs, Replacements, Re-Inspections
Core Shroud	RO-14 Fall 2000	EVT-1	<p>Re-inspected per BWRVIP-76 Guidelines accessible areas of Vertical Welds SV5A, SV5B, SV6A and SV6B. Relevant indications found in these welds are as follows:</p> <ul style="list-style-type: none"> SV5A OD - There are 7 indications total with a combined indication length of 11.7 inches vertical and 3.3 inches circ. SV5B OD - There are 19 indications total with a combined indication length of 50.7 inches vertical. SV6A OD - There is 1 vertical indication that is measured to be 1 inch long. SV6B ID - There is 1 vertical indication in the weld that is measured to be 1.25 inches long. SH4 ID - There are 2 vertical indications across SH4 with a total combined length of 6.4 inches. The closest indication is 3 inches from SV5B. This indication has some branches near the bottom. <p>All relevant indications were satisfactorily dispositioned with the response to Deviation Event Report No. DER-00-05312 and by Calculation JAF-CALC-NBS-04221.</p>
Core Spray Piping	RO-14	EVT-1	<p>Re-inspected accessible areas of all loop 'A' and 'B' creviced and T-box-to-pipe welds, including repair clamp welds per BWRVIP-18 Guidelines. A relevant indication was noted on weld CSB-12 (Indication length is 3.5 inch long). No other relevant indications were noted. The relevant indication was satisfactorily dispositioned with the response to Deviation Event Report No. DER-00-05134 and by Nuclear Safety Evaluation JAF-SE-00-049.</p>
Top Guide (Rim, etc.)	RO-14	VT-1/VT-3	<p>A total of 4 hold down assemblies were examined by VT-1 and 3 alignment pin assemblies by VT-3 per BWRVIP-26 Guidelines. No indications were noted.</p>
Core Plate (Rim, etc.)	RO-14	VT-3	<p>Inspected core plate plugs at 5 core locations. No recordable indications noted.</p>
SLC	RO-14	EVT-2	<p>Performed Enhanced VT-2 on SLC nozzle-to-safe end weld during RPV System Leakage Test per BWRVIP-27 Guidelines. Examination results were acceptable.</p>
CRD Guide Tube	RO-14	EVT-1/VT-3	<p>Inspected accessible surfaces at 3 guide tubes per BWRVIP-47 Guidelines (EVT-1/VT-3). Inspected accessible surfaces at 8 guide tubes (VT-3). Inspected a total of 11 guide tubes. No relevant indications noted.</p>

ATTACHMENT 1
IVVI SUMMARY REPORT (RO-14)
JAMES A. FITZPATRICK NUCLEAR POWER PLANT (JAF)

Components in BWRVIP Scope	Date or Frequency of Inspection	Inspection Method Used	Summarize the Following Information: Inspection Results, Repairs, Replacements, Re-Inspections
Dry Tube	RO-14	VT-1	Inspected 4 IRM/SRM In-Core Dry Tubes per GE-SIL-409 and GE RICSIL-073 Guidelines (VT-1). No recordable indications noted.
Instrument Penetrations	RO-14	VT-2	Performed VT-2 ISI System Leakage Exam Test at 6 instrument nozzles (during RPV System Test) per BWRVIP-49 Guidelines. Test was conducted to the extent possible with insulation installed and shield doors closed. Examination results were acceptable.
Fuel Support Castings	RO-14	VT-3	Inspected accessible areas of fuel support castings during in-process control rod blade change-out. No relevant indications noted.
CRD Nozzle NIR	RO-14	EVT-1	The Control Rod Drive Nozzle Inner Radius and adjacent vessel wall area were examined. No relevant indications noted.
Steam Dryer Moisture Separator	RO-14	EVT-1/VT-3	Re-inspected by VT-3 all areas of the steam dryer support ring and by EVT-1 previously found cracks (1992/1994). A total of 10 indications were noted in 2000 (RO-14). The indications are typical of cracking discovered at other plants and are pre-existing as verified by a review of the IVVI video tapes recorded during the 10th refueling in 1992. No crack growth was noted.
Surveillance Capsule Specimen Holder	RO-14	VT-1/VT-3	Inspected at one location the upper and lower mounting bracket (VT-1) and the condition of the specimen holder (VT-3). No recordable indications noted.
Lower Plenum	RO-14	VT-1/VT-3	Inspected by VT-3 accessible areas of lower plenum per BWRVIP-47 Guidelines. No recordable indications noted. Inspected by VT-1 accessible areas of bottom head drain. After removal of debris the area was re-examined and found acceptable.