

February 2, 2001

Mr. R. G. Lizotte
Master Process Owner - Assessment
c/o Mr. David A. Smith
Northeast Nuclear Energy Company
P. O. Box 128
Waterford, CT 06385-0128

SUBJECT: MILLSTONE NUCLEAR POWER STATION, UNIT NO. 3 - ISSUANCE OF
AMENDMENT RE: ELECTRICAL POWER SYSTEMS (TAC NO. MA9661)

Dear Mr. Lizotte:

The Commission has issued the enclosed Amendment No. 194 to Facility Operating License No. NPF-49 for the Millstone Nuclear Power Station, Unit No. 3, in response to your application dated July 31, 2000, as supplemented January 5, 2001.

The amendment approves changes to Technical Specifications (TSs) 3.8.1.1, "Electrical Power Systems - A.C. Sources - Operating," and 3.8.1.2, "Electrical Power Systems - A.C. Sources - Shutdown." The index and Bases pages for these TSs are modified to reflect the changes.

A copy of the related Safety Evaluation is also enclosed. Notice of Issuance will be included in the Commission's biweekly Federal Register notice.

Sincerely,

/RA/

Victor Nerses, Sr. Project Manager, Section 2
Project Directorate I
Division of Licensing Project Management
Office of Nuclear Reactor Regulation

Docket No. 50-423

Enclosures: 1. Amendment No. 194 to NPF-49
2. Safety Evaluation

cc w/encls: See next page

Mr. R. G. Lizotte
Master Process Owner - Assessment
c/o Mr. David A. Smith
Northeast Nuclear Energy Company
P. O. Box 128
Waterford, CT 06385-0128

February 2, 2001

SUBJECT: MILLSTONE NUCLEAR POWER STATION, UNIT NO. 3 - ISSUANCE OF
AMENDMENT RE: ELECTRICAL POWER SYSTEMS (TAC NO. MA9661)

Dear Mr. Lizotte:

The Commission has issued the enclosed Amendment No. 194 to Facility Operating License No. NPF-49 for the Millstone Nuclear Power Station, Unit No. 3, in response to your application dated July 31, 2000, as supplemented January 5, 2001.

The amendment approves changes to Technical Specifications (TSs) 3.8.1.1, "Electrical Power Systems - A.C. Sources - Operating," and 3.8.1.2, "Electrical Power Systems - A.C. Sources - Shutdown." The index and Bases pages for these TSs are modified to reflect the changes.

A copy of the related Safety Evaluation is also enclosed. Notice of Issuance will be included in the Commission's biweekly Federal Register notice.

Sincerely,

/RA/

Victor Nerses, Sr. Project Manager, Section 2
Project Directorate I
Division of Licensing Project Management
Office of Nuclear Reactor Regulation

Docket No. 50-423

Enclosures: 1. Amendment No. 194 to NPF-49
2. Safety Evaluation

cc w/encls: See next page

DISTRIBUTION:

PUBLIC	OGC	PDI-2 R/F	RSummers, RI
EAdensam (E-mail EGA1)	TClark	ACRS	VNerses
GHill (2)	WBeckner	JClifford	AWang

ADAMS ACCESSION NUMBER: ML010240253

**See previous concurrence

*SE dated 1/29/01, no major changes

OFFICE	PDI-2/PM	PDI-2/PM	PDI-2/LA	EEIB*	OGC**	PDI-2/SC	
NAME	VNerses	AWang	TClark	OChopra	O'Neill	JClifford	
DATE	1/31/01	1/31/01	01/31/01	01/29/01	01/30/01	2/2/01	

OFFICIAL RECORD COPY

Millstone Nuclear Power Station
Unit 3

cc:

Ms. L. M. Cuoco
Senior Nuclear Counsel
Northeast Utilities Service Company
P. O. Box 270
Hartford, CT 06141-0270

Edward L. Wilds, Jr., Ph.D.
Director, Division of Radiation
Department of Environmental Protection
79 Elm Street
Hartford, CT 06106-5127

Regional Administrator, Region I
U.S. Nuclear Regulatory Commission
475 Allendale Road
King of Prussia, PA 19406

First Selectmen
Town of Waterford
15 Rope Ferry Road
Waterford, CT 06385

Mr. J. T. Carlin
Vice President - Human Services - Nuclear
Northeast Nuclear Energy Company
P. O. Box 128
Waterford, CT 06385

Mr. F. C. Rothen
Vice President - Nuclear Work Services
Northeast Nuclear Energy Company
P. O. Box 128
Waterford, CT 06385

Ernest C. Hadley, Esquire
P.O. Box 1104
West Falmouth, MA 02574-1104

Deborah Katz, President
Citizens Awareness Network
P.O. Box 83
Shelburne Falls, MA 03170

Mr. Allan Johanson, Assistant Director
Office of Policy and Management
Policy Development & Planning Division
450 Capitol Avenue - MS# 52ERN
P. O. Box 341441
Hartford, CT 06134-1441

Ms. Terry Concannon
Co-Chair
Nuclear Energy Advisory Council
41 South Buckboard Lane
Marlborough, CT 06447

Mr. R. P. Necci
Vice President - Nuclear Technical Services
Northeast Nuclear Energy Company
P. O. Box 128
Waterford, CT 06385

Mr. Evan W. Woollacott
Co-Chair
Nuclear Energy Advisory Council
128 Terry's Plain Road
Simsbury, CT 06070

Mr. John W. Beck, President
Little Harbor Consultants, Inc.
44 Nichols Road
Cohasset, MA 02025-1166

Mr. L. J. Olivier
Senior Vice President and
Chief Nuclear Officer - Millstone
Northeast Nuclear Energy Company
P.O. Box 128
Waterford, CT 06385

Mr. C. J. Schwarz
Master Process Owner - Operate the Asset
Northeast Nuclear Energy Company
P.O. Box 128
Waterford, CT 06385

Millstone Nuclear Power Station
Unit 3

cc:

Senior Resident Inspector
Millstone Nuclear Power Station
c/o U.S. Nuclear Regulatory Commission
P. O. Box 513
Niantic, CT 06357

Nicholas J. Scobbo, Jr., Esquire
Ferriter, Scobbo, Caruso, & Rodophele, P.C.
75 State Street, 7th Floor
Boston, MA 02108-1807

Mr. G. D. Hicks
Master Process Owner - Training
Northeast Nuclear Energy Company
P.O. Box 128
Waterford, CT 06385

Citizens Regulatory Commission
ATTN: Ms. Geri Winslow
P. O. Box 199
Waterford, CT 06385

Mr. B. D. Kenyon
President and Chief Executive Officer-
NNECO
Northeast Nuclear Energy Company
P.O. Box 270
Hartford, CT 06141-0270

Mr. D. A. Smith
Process Owner - Regulatory Affairs
Northeast Nuclear Energy Company
P. O. Box 128
Waterford, CT 06385

Ms. Nancy Burton
147 Cross Highway
Redding Ridge, CT 00870

NORTHEAST NUCLEAR ENERGY COMPANY, ET AL.

DOCKET NO. 50-423

MILLSTONE NUCLEAR POWER STATION, UNIT NO. 3

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 194

License No. NPF-49

1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment by Northeast Nuclear Energy Company, et al. (the licensee) dated July 31, 2000, as supplemented January 5, 2001, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 2.C.(2) of Facility Operating License No. NPF-49 is hereby amended to read as follows:

- (2) Technical Specifications

The Technical Specifications contained in Appendix A, as revised through Amendment No. 194 , and the Environmental Protection Plan contained in Appendix B, both of which are attached hereto, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications and the Environmental Protection Plan.

3. This license amendment is effective as of the date of issuance, and shall be implemented within 30 days of issuance.

FOR THE NUCLEAR REGULATORY COMMISSION

/RA/

James W. Clifford, Chief, Section 2
Project Directorate I
Division of Licensing Project Management
Office of Nuclear Reactor Regulation

Attachment: Changes to the Technical
Specifications

Date of Issuance: February 2, 2001

ATTACHMENT TO LICENSE AMENDMENT NO. 194

FACILITY OPERATING LICENSE NO. NPF-49

DOCKET NO. 50-423

Replace the following pages of the Appendix A Technical Specifications, with the attached revised pages. The revised pages are identified by amendment number and contain marginal lines indicating the areas of change.

Remove

xi
3/4 8-3
3/4 8-4
3/4 8-5
3/4 8-6
3/4 8-7
3/4 8-8
3/4 8-9
3/4 8-10
B 3/4 8-1b
B 3/4 8-1c

Insert

xi
3/4 8-3
3/4 8-4
3/4 8-5
3/4 8-6
3/4 8-7
3/4 8-8
3/4 8-9
3/4 8-10
B 3/4 8-1b
B 3/4 8-1c

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION
RELATED TO AMENDMENT NO. 194
TO FACILITY OPERATING LICENSE NO. NPF-49
NORTHEAST NUCLEAR ENERGY COMPANY, ET AL.
MILLSTONE NUCLEAR POWER STATION, UNIT NO. 3
DOCKET NO. 50-423

1.0 INTRODUCTION

By letter dated July 31, 2000, as supplemented January 5, 2001, the Northeast Nuclear Energy Company, et al. (the licensee), submitted a request for changes to the Millstone Nuclear Power Station, Unit No. 3 (MP3) Technical Specifications (TSs). The requested changes would modify Technical Specifications (TSs) 3.8.1.1, "Electrical Power Systems - A.C. Sources - Operating," and 3.8.1.2, "Electrical Power Systems - A.C. Sources - Shutdown." The index and Bases for these TSs would also be modified to reflect the proposed changes. The January 5, 2001, letter provided clarifying information that did not change the initial proposed no significant hazards consideration determination.

2.0 BACKGROUND

MP3 has two emergency diesel generators (EDGs). Each EDG is capable of supplying power to the respective emergency 4160 volt bus. During normal power operation, the EDGs are maintained in a standby mode. The EDGs will start on a loss-of-power to the respective emergency bus, a safety injection signal (SIS), or a containment depressurization accident signal (CDAS). If the normal and alternate offsite power sources are not available, the EDGs then automatically connect to the respective emergency bus and sequentially load. Upon receiving an automatic start signal, the EDGs accelerate to rated speed, frequency, and voltage within 11 seconds, and are ready to accept load in accordance with the unit's sequential loading schedule. Sequential loading is achieved by an emergency generator load sequencer. The sequencer is a solid-state digital system that provides relay contact outputs to shed loads, block manual starts, and sequentially load the plant safety buses during emergency conditions.

3.0 EVALUATION

The licensee proposed the following changes to the MP3 TSs which would allow certain EDG surveillance requirements (SR) to be performed when the plant is operating instead of shut down as currently required. The licensee states that performing these surveillances while the plant is operating would provide additional flexibility in the scheduling of maintenance activities

and reduce plant refueling outage duration and increase EDG availability when the plant is shut down. Additional changes would remove EDG accelerated testing and special reporting requirements, and also remove the SR to perform various EDG inspections in accordance with the recommendations of the manufacturer.

3.1 SR 4.8.1.1.2.a

The licensee proposed to modify this SR by replacing the phrase "In accordance with the frequency specified in Table 4.8.1" with the phrase "At least once per 31 days." The 31-day test frequency is the same as currently required. This change is necessary due to proposed deletion of Table 4.8-1 (see section 3.8). Since the SR remains unchanged, the licensee's proposal is only a relocation of an existing requirement from the table to the SR itself, therefore, the staff finds this change to be acceptable.

3.2 SR 4.8.1.1.2.b

The licensee proposed to divide this SR into two SRs, SR 4.8.1.1.2.b.1 and SR 4.8.1.1.2.b.2. This change is necessary to allow the current loading requirement in SR 4.8.1.1.2.b.2 to be excluded when the plant is shut down and the licensee is performing SR 4.8.1.2. This change is discussed in the proposed change to SR 4.8.1.2 (see section 3.9). The requirements of SR 4.8.1.1.2.b are maintained, and the staff concludes that this is a format change that does not result in any technical change to the current requirements and, therefore, is acceptable.

The proposed change would also add the phrase "from standby conditions" to the EDG starting requirements contained in the proposed SR 4.8.1.1.2.b.1. During standby conditions, the EDG engine coolant and lubricating oil are being circulated and temperatures are being maintained within design ranges. The staff agrees that starting the EDGs from standby conditions rather than from a cold start reduces stresses and mechanical wear on the EDGs, with potential improvement in EDG reliability; therefore, the proposed change is acceptable. This proposed change is also consistent with GL 84-15, "Proposed Staff Actions to Improve and Maintain Diesel Generator Reliability."

3.3 SR 4.8.1.1.2.g.1

This SR currently requires performance of EDG inspections every 18 months in accordance with procedures prepared in conjunction with the recommendations of the manufacturer. The licensee proposed to remove this SR from the TSs. Title 10 of the *Code of Federal Regulations*, Section 50.36(c)(3) "Surveillance Requirements" defines SR as "requirements relating to test, calibration, or inspection to assure that the necessary quality or systems and components is maintained, that facility operation will be within safety limits, and that the limiting conditions for operation will be met." This SR is a maintenance activity that does not verify the operability of the EDG or any EDG function as assumed in the safety analysis as defined by 10 CFR 50.36(c)(3). As this SR does not meet the above criteria, the licensee has proposed to relocate this SR from the TSs to the maintenance procedures, a licensee-controlled document. In addition, the licensee states the inspection requirements contained in the current SR will be maintained in the maintenance procedures and that any change to the maintenance procedures will require an evaluation in accordance with 10 CFR 50.59. Since these requirements are being relocated to the maintenance procedures without change, and any future change to these requirements would require a 10 CFR 50.59 evaluation, the staff finds the proposed change to be acceptable.

3.4 SR 4.8.1.1.2.g.7

This SR currently requires verification that the EDGs operate for at least 24 hours, at least once per 18 months, when the plant is shut down. During the first 2 hours of this test, the EDG shall be loaded between 5400-5500 kilowatt-electric (kWe), and during the remaining 22 hours of this test, the EDG shall be loaded between 4800-5000 kWe. The generator voltage and frequency shall be 4160 ± 420 volts and 60 ± 0.8 Hertz within 11 seconds after the start signal. The steady state generator voltage and frequency shall be maintained within these limits during this test. Within 5 minutes after completing this 24-hour test, SR 4.8.1.1.2.a.5 shall be performed. This test is presently performed only during shutdown.

The licensee has proposed to perform this surveillance during power operation. The licensee states that performing this test during power operation would help simplify and shorten the scheduling of the EDG testing and surveillance window during a refueling outage. In addition, SR 4.8.1.1.2.g.7 and the associated footnotes will be relocated as SR 4.8.1.1.2.j, so that the "during shutdown" restriction does not apply.

During the 24-hour functional test, an EDG is loaded by paralleling with the offsite power system. Only one EDG is paralleled to the offsite source at any one time. Thus, the testing does not affect the independent safe shutdown capabilities of the remaining EDG or the emergency bus. In the event of an emergency, with an EDG operating in the test mode and offsite power available, the emergency actuation signal (SIS or CDAS) overrides the test mode, returns the EDG to standby operation and automatically energizes the emergency bus loads with offsite power. This transfer is tested once per refueling cycle in accordance with SR 4.8.1.1.2.g.10. If a loss of offsite power (LOOP) occurs following the emergency actuation signal, the EDG will automatically connect to the emergency bus and be sequentially loaded as designed. Thus, with the EDG operating in the test mode, it will be available to perform its intended safety function.

In the scenario of a LOOP to the bus paralleled to the grid without an accident, no load shedding or sequencing will take place because the EDG will maintain the voltage on the associated bus. The EDG would attempt to provide power to the emergency bus and to the non-emergency 4160 volt bus through the closed bus tie breaker. The licensee states that in this case, the bus tie breaker will trip on directional overcurrent. The EDG will continue to operate in a droop mode of governor control supplying its respective emergency bus. Thus, the EDG in test will remain available to supply its emergency bus in the event of a LOOP. In addition, adequate capacity is available from the remaining EDG to power the remaining division, and the remaining division will have the required equipment operable to mitigate the consequences of a design basis accident or LOOP.

By letter dated January 5, 2001, the licensee states that procedural prerequisites and cautions will be implemented for the 24-hour test to ensure that only one EDG is paralleled to the grid for the monthly tests. In addition, these procedural restrictions require verification that a tornado watch/warning, hurricane advisory, or other possible LOOP conditions do not exist if the EDG is to be paralleled to the grid. The above ensures that the test will not be performed when severe weather is predicted or during unstable grid conditions.

Based on the above, the staff concludes that the performance of the 24-hour EDG functional test during power operation is acceptable due to the following provisions: (1) the EDGs are equipped with a design feature that allows the EDGs to automatically switch from the test mode to the standby mode on the receipt of an accident signal; (2) during the 24-hour test of an EDG, no other EDG is operated in parallel with the offsite power grid; and (3) assuming a LOOP and a single failure of an EDG, adequate capacity is available from the remaining EDG to power the remaining division.

In addition to the above change, the phrase “excluding the requirement to start the diesel generator from standby conditions” will be added to the requirement to perform SR 4.8.1.1.2.a.5 within 5 minutes of completing the 24-hour test. This clarification is consistent with the intent of the requirement to verify the EDG can restart from hot conditions and, therefore, is acceptable.

3.5 SR 4.8.1.1.2.g.11

This SR currently requires verification that the fuel transfer pump transfers fuel from each fuel storage tank to the day tank of each EDG via the installed cross-connection lines, at least once per 18 months when the plant is shut down. The licensee has proposed to perform this SR during power operation. The proposed change will not result in any technical changes to the current requirement. In addition, SR 4.8.1.1.2.g.11 will be relocated as SR 4.8.1.1.2.k so that the “during shutdown” restriction does not apply. Allowing this test to be performed when the plant is operating will provide additional flexibility in the scheduling of this maintenance activity.

The licensee states that the verification of the ability to transfer fuel oil from each fuel oil storage tank to each EDG fuel oil day tank requires the two manual fuel oil cross-connect valves to be opened. This is a simple task that can be performed within a short period of time. During performance of this SR, the day tank level control of one of the EDGs is placed in manual, resulting in that EDG being declared inoperable. This is typically a short duration test of less than 1 hour. The licensee states that there will be an insignificant increase in overall system inoperable time or unavailability. The staff concludes that this test can be accomplished in a short time without posing any threat to the safety of the plant as the redundant train remains operable. Therefore, the proposed change is acceptable.

3.6 SR 4.8.1.1.2.g.13

This SR currently requires verification that certain EDG lockout features prevent the EDG from starting at least once per 18 months when the plant is shutdown. The licensee has proposed to perform this SR during power operation. The proposed change will not result in any technical change to the current requirement. In addition, SR 4.8.1.1.2.g.13 will be relocated as SR 4.8.1.1.2.l so that the “during shutdown” restriction does not apply.

SR 4.8.1.1.2.g.13 verifies that engine overspeed, the lube oil low pressure generator differential, and emergency stop conditions prevent the EDG from starting. During performance of this SR the EDG will be inoperable. The licensee states that this SR can be performed well within one half of the 72-hour allowed outage time. Since this test is only required to be performed once per 18 months, the licensee states that the increase in inoperability due to the test is not significant. In addition, the licensee states that allowing this test to be performed when the plant is operating will provide additional flexibility in the scheduling of this maintenance activity. This will reduce plant refueling outage duration and improve EDG availability when shut down. The staff concludes that performance of this SR will not cause

perturbations to any of the electrical distribution systems that could result in a challenge to steady state operation or to plant safety systems because the EDG in this test will not be tied to the grid. Further, performance of this test will not affect the remaining operable EDG, and system restoration can be readily accomplished. Therefore, the proposed change is acceptable.

3.7 SR 4.8.1.1.2.h

This licensee has proposed to add the phrase “from standby conditions” to the EDG starting requirements contained in SR 4.8.1.1.2.h. During standby conditions, the EDG engine coolant and lubricating oil are being circulated and temperatures are being maintained within design ranges. The staff agrees that starting the EDGs from standby conditions rather than from a cold start reduces stresses and mechanical wear on the EDGs, with potential improvement in EDG reliability; therefore, the proposed change is acceptable. This proposed change is also consistent with GL 84-15, “Proposed Staff Actions to Improve and Maintain Diesel Generator Reliability.”

3.8 SR 4.8.1.1.3, “Reports,” and Table 4.8-1, “Diesel Generator Test Schedule”

The licensee has proposed to delete SR 4.8.1.1.3 and Table 4.8-1 from the MP3 TSs. This is in accordance with GL 94-01, “Removal of Accelerated Testing and Special Reporting Requirements For Emergency Diesel Generators.” As part of the resolution of Generic Safety Issue B-56, “Diesel Generator Reliability,” the staff concluded that licensees may propose TS changes to remove special reporting requirements for EDGs from their TSs. Licensees may also request relief from a docketed commitment for accelerated testing (Table 4.8-1) of EDGs and reporting each EDG failure to the Nuclear Regulatory Commission (SR 4.8.1.1.3). The staff approval of this option was contingent upon a commitment to implement a maintenance program for monitoring and maintaining EDG performance in accordance with the provisions of 10 CFR 50.65, “Requirements for Monitoring the Effectiveness of Maintenance at Nuclear Power Plants,” and the guidance contained in Regulatory Guide 1.160, “Monitoring the Effectiveness of Maintenance at Nuclear Power Plants.”

The licensee has implemented the Maintenance Rule in accordance with the requirements of 10 CFR 50.65 utilizing the guidance contained in Regulatory Guide 1.160. The licensee has established a program to monitor EDG performance. This program includes criteria for EDG reliability and unavailability. The licensee’s program includes a provision that they make a cause determination for each failure classified as a maintenance rule failure. If any performance criteria is exceeded, the licensee will evaluate the EDG for monitoring under 10 CFR 50.65.a.1. Based on the above, the staff finds the proposed changes to be consistent with GL 94-01 for meeting the maintenance rule and, therefore, are acceptable.

3.9 SR 4.8.1.2

This SR is applicable during shutdown. It currently requires that the licensee demonstrate operability of the required A.C. electrical power sources by performance of each of the requirements of SRs 4.8.1.1.1, 4.8.1.1.2 (except for SR 4.8.1.1.2.a.6), and 4.8.1.1.3. The licensee has proposed to modify this SR to exclude loading of EDG during performance of the 184-day test (SR 4.8.1.1.2.b.2) and remove reference to SR 4.8.1.1.3 (EDG failure reporting requirement).

The licensee states that in Modes 5 and 6, only one EDG is required to be operable. Therefore, it is not prudent when performing the 184-day surveillance to require the only operable EDG to be paralleled with the grid. Performance of SR 4.8.1.1.2.b.1 (which verifies the EDG will start) is sufficient to provide reasonable assurance of the operability of that EDG. The staff agrees with the licensee that it is not prudent to connect the only available EDG with the grid to satisfy the loading requirement because connecting the only operable EDG with the grid would increase the likelihood that a grid fault could result in the loss of both normal and emergency power. The above is also consistent with the current requirement contained in this SR that excludes loading of the EDG during the 31-day test (SR 4.8.1.1.2.a.6). Therefore, the staff finds the proposed change to be acceptable.

The proposed change to remove the reference to SR 4.8.1.1.3 is consistent with the proposed change to delete SR 4.8.1.1.3 as discussed in section 3.8 and, therefore, is acceptable.

3.10 Bases Changes

The Bases for the above TSs would be modified to be consistent with the proposed changes previously discussed. The staff does not object to the proposed TS Bases changes.

4.0 STATE CONSULTATION

In accordance with the Commission's regulations, the Connecticut State official was notified of the proposed issuance of the amendment. The State official had no comments.

5.0 ENVIRONMENTAL CONSIDERATION

The amendment changes a requirement with respect to installation or use of a facility component located within the restricted area as defined in 10 CFR Part 20 and changes surveillance requirements. The NRC staff has determined that the amendment involves no significant increase in the amounts, and no significant change in the types, of any effluents that may be released offsite, and that there is no significant increase in individual or cumulative occupational radiation exposure. The Commission has previously issued a proposed finding that the amendment involves no significant hazards consideration, and there has been no public comment on such finding (65 FR 54087). Accordingly, the amendment meets the eligibility criteria for categorical exclusion set forth in 10 CFR 51.22(c)(9). Pursuant to 10 CFR 51.22(b) no environmental impact statement or environmental assessment need be prepared in connection with the issuance of the amendment.

6.0 CONCLUSION

The Commission has concluded, based on the considerations discussed above, that: (1) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, (2) such activities will be conducted in compliance with the Commission's regulations, and (3) the issuance of the amendment will not be inimical to the common defense and security or to the health and safety of the public.

Principal Contributor: O. Chopra

Date: February 2, 2001