



JAN 12 2001

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U. S. Nuclear Regulatory Commission  
Document Control Desk  
Washington, DC 20555

Gentlemen:

**MONTHLY OPERATING REPORT  
SALEM GENERATING STATION UNIT 1  
DOCKET NO. 50-272**

In compliance with Section 6.9, Reporting Requirements for the Salem Unit 1 Technical Specifications, the operating statistics for **December 2000** are being forwarded. Also being forwarded, pursuant to the requirements of 10CFR50.59(b), is a summary of changes, tests, and experiments that were implemented in **December 2000**.

Sincerely,

A handwritten signature in black ink, appearing to read "D. F. Garchow".

D. F. Garchow  
Vice President – Operations

RBK  
Attachments

C Distribution

Handwritten initials "JED24" in black ink.

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DOCKET NO.: 50-272  
 UNIT: Salem 1  
 DATE: 1/4/01  
 COMPLETED BY: R. Knieriem  
 TELEPHONE: (856) 339-1782

Reporting Period December 2000

### OPERATING DATA REPORT

Design Electrical Rating (MWe-Net)  
 Maximum Dependable Capacity (MWe-Net)

No. of hours reactor was critical  
 No. of hours generator was on line (service hours)

Unit reserve shutdown hours  
 Net Electrical Energy (MWH)

1115		
1106		
Month	Year-to-date	Cumulative
709	8464	127008
687	8328	122580
0	0	0
744267	8952640	123573975

### UNIT SHUTDOWNS

NO.	DATE	TYPE F=FORCED S=SCHEDULED	DURATION (HOURS)	REASON (1)	METHOD OF SHUTTING DOWN THE REACTOR (2)	CORRECTIVE ACTION/ COMMENT
6	12/8/00 - 12/10/00	F	57	A	3	Solid State Protection System component failure resulted in loss of Feedwater and Steam Generator low level.

(1) Reason

- A - Equipment Failure (Explain)
- B - Maintenance or Test
- C - Refueling
- D - Regulatory Restriction
- E - Operator Training/License Examination
- F - Administrative
- G - Operational Error (Explain)
- H - Other

(2) Method

- 1 - Manual
- 2 - Manual Trip/Scram
- 3 - Automatic Trip/Scram
- 4 - Continuation
- 5 - Other (Explain)

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**Summary Of Monthly Operating Experience**

- Salem Unit 1 began the month of December 2000 operating at full power.
- On December 8, Salem Unit 1 tripped due to a Solid State Protection system component failure.
- Salem Unit 1 returned to service on December 10, and reached full power on December 11.
- Salem Unit 1 operated at full power for the remainder of the month.

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**SUMMARY OF CHANGES, TESTS, AND EXPERIMENTS**  
**FOR THE SALEM GENERATING STATION – UNIT 1**

**MONTH December 2000**

The following items completed during **December 2000** have been evaluated to determine:

1. If the probability of occurrence or the consequences of an accident or malfunction of equipment important to safety previously evaluated in the safety analysis report may be increased; or
2. If a possibility for an accident or malfunction of a different type than any evaluated previously in the safety analysis report may be created; or
3. If the margin of safety as defined in the basis for any technical specification is reduced.

The 10CFR50.59 Safety Evaluations showed that these items did not create a new safety hazard to the plant nor did they affect the safe shutdown of the reactor. These items did not change the plant effluent releases and did not alter the existing environmental impact. The 10CFR50.59 Safety Evaluations determined that no unreviewed safety or environmental questions are involved.

**Design Changes    Summary of Safety Evaluations**

**Modification 80005323, Steam Generator Blowdown Radiation Monitor (1R19)  
Sample Line Pressure Regulator and Flow Instrument Replacement**

This modification replaced the existing steam generator blowdown radiation monitor sample line pressure regulators and flow instrument with new components that incorporate design improvements to improve the overall reliability of the system.

Review of this modification under 10CFR50.59 was required because the modification of the steam generator blowdown radiation monitor sample system constituted a change to the facility as described in the UFSAR. This modification will maintain the original functionality of the system while improving system reliability. Therefore, this change would not increase the probability or consequences of an accident previously

**SUMMARY OF CHANGES, TESTS, AND EXPERIMENTS**  
**FOR THE SALEM GENERATING STATION – UNIT 1 – Cont.**

analyzed. Additionally, this change did not increase the probability or consequences of a malfunction of equipment important to safety. This change would not create any new accidents or malfunctions since no new failure modes were introduced. In addition the Technical Specification Bases were not affected and no changes to the Technical Specifications were required.

**Modification 80006696, 11 Auxiliary Feedwater (AFW) Pump Runout Protection Instrumentation**

This design change replaced obsolete instrumentation used to provide runout protection for the 11 AFW Pump because of the unavailability of replacement components for the original instrumentation.

Review of this modification under 10CFR50.59 was required because modification of the 11 AFW Pump Runout Protection instrumentation constituted a change to the facility as described in the UFSAR. This change was performed to maintain the reliability of the AFW system to ensure that it will meet the design basis flows required to mitigate all currently analyzed accident scenarios. Therefore, this change would not increase the probability or consequences of an accident previously analyzed. Additionally, this change did not increase the probability or consequences of a malfunction of equipment important to safety. This change would not create any new accidents or malfunctions since no new failure modes were introduced. In addition the Technical Specification Bases were not affected and no changes to the Technical Specifications were required.

**Modification 80010402, Installation of Crossflow Meters for Flow Nozzle Correction**

This design change installed Crossflow Meters to be used to provide greater accuracy for the flow correction used to compensate for Feedwater System flow nozzle fouling during the performance of reactor thermal power calculations. The correction factors provided by the Crossflow Meters will replace the correction factors currently provided by Leading Edge Flow Meters.

Review of this modification under 10CFR50.59 was required because the installation of the Crossflow meters constituted a change to the facility as described in the UFSAR. This change was performed to provide enhanced reactor thermal power calculation accuracy. Therefore, this change would not increase the probability or consequences of an accident previously analyzed. Additionally, this change did not increase the probability or consequences of a malfunction of equipment important to safety. This change would not create any new accidents or malfunctions since no new failure modes were introduced. In addition the Technical Specification Bases were not affected and no changes to the Technical Specifications were required.

**SUMMARY OF CHANGES, TESTS, AND EXPERIMENTS**  
**FOR THE SALEM GENERATING STATION – UNIT 1 – Cont.**

**Modification 80020525, Heater Drain Pump Piping Modification**

This modification replaced the welded piping associated with the Heater Drain Pump stuffing box cooling inlet and outlet lines, and the seal water inlet line, with flexible jacketed hose. The flexible jacketed hose was installed using non-welded mechanical connections and will facilitate maintenance on the Heater Drain Pumps by simplifying assembly and disassembly of the three connections.

Review of this modification under 10CFR50.59 was required because the installation of the flexible jacketed hose on the Heater Drain Pump stuffing box cooling inlet and outlet lines, and the seal water inlet line constituted a change to the facility as described in the UFSAR. The Heater Drain Pumps are used to return accumulated condensate in the Feedwater Heaters to the Condensate System and do not serve any safety-related function. Therefore, this change would not increase the probability or consequences of an accident previously analyzed. Additionally, this change did not increase the probability or consequences of a malfunction of equipment important to safety. This change would not create any new accidents or malfunctions since no new failure modes were introduced. In addition the Technical Specification Bases were not affected and no changes to the Technical Specifications were required.

**Temporary Modifications   Summary of Safety Evaluations**

There were no reportable changes in this category implemented during December 2000.

**Procedures   Summary of Safety Evaluations**

There were no reportable changes in this category implemented during December 2000.

**UFSAR Change Notices   Summary of Safety Evaluations**

**Salem UFSAR Change Notice 00-051, Evaluation of Main Steam Isolation Valves (MSIV) as Containment Isolation Valves**

This change incorporated the results of an evaluation that considered the effect on the design basis functions of the MSIVs during design basis accidents, and the effect on 10 CFR 100 dose limits of slower closure of the MSIVs, when the valves are closed using the hydraulic operator in Mode 3 and Mode 4 when steam pressure is insufficient for fast closure.

Review of these change under 10CFR50.59 was required because the change constitutes a change to the facility as described in the UFSAR, changed the Technical Specification Bases, and changed procedures as described in the UFSAR. This

**SUMMARY OF CHANGES, TESTS, AND EXPERIMENTS**  
**FOR THE SALEM GENERATING STATION – UNIT 1 – Cont.**

evaluation verified the ability of the MSIVs to perform their design function using hydraulic actuation when steam pressure is reduced during operation in Modes 3 and 4. Therefore, this change would not increase the probability or consequences of an accident previously analyzed. Additionally, this change did not increase the probability or consequences of a malfunction of equipment important to safety. This change would not create any new accidents or malfunctions since no new failure modes were introduced. In addition no changes to the Technical Specifications were required.

**Other - Summary of Safety Evaluations**

**Salem Offsite Dose Calculation Manual (ODCM), Revision 14**

Revision 14 to the Salem ODCM implemented administrative and editorial changes to more accurately reflect the relocation of the Radioactive Effluents Technical Specifications to the ODCM that was authorized by Salem License Amendments 234 (Salem Unit 1) and 215 (Salem Unit 2).

Since the ODCM is a requirement specified by Technical Specification 6.8, review of Revision 14 to the Offsite Dose Calculation Manual under 10CFR50.59 was required because the revision of the ODCM constituted a change to the facility as described in the UFSAR. The changes implemented by Revision 14 are administrative and editorial in nature and will not reduce the requirements for liquid and gaseous effluent releases. Therefore, this change would not increase the probability or consequences of an accident previously analyzed. Additionally, this change did not increase the probability or consequences of a malfunction of equipment important to safety. This change would not create any new accidents or malfunctions since no new failure modes were introduced. In addition the Technical Specification Bases were not affected and no changes to the Technical Specifications were required.

**Revision to Emergency Diesel Generator (EDG) Capacity and Loading Calculation (ES-9.002)**

This evaluation considered changes to the EDG capacity and loading calculation that were imposed as a result of a modification to the Auxiliary Building Ventilation fans. The modification to the Auxiliary Building Ventilation fans replaced the existing fan belt and adjustable pulley with a cogged wheel pulley and cogged belt.

Evaluation of the revision to calculation ES-9.002 was required under 10CFR50.59 because the update of EDG capacity and loading constituted a change to the facility as described in the UFSAR. This evaluation verified that all expected loads are within the EDG ratings and that the automatically connected loads will not exceed the EDG two-hour rating. Therefore, this change would not increase the probability or consequences of an accident previously analyzed. Additionally, this change did not increase the probability or consequences of a malfunction of equipment important to safety. This



**SUMMARY OF CHANGES, TESTS, AND EXPERIMENTS**  
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change would not create any new accidents or malfunctions since no new failure modes were introduced. In addition the Technical Specification Bases were not affected and no changes to the Technical Specifications were required.