

March 27, 1997

Mr. James Knubel
Chief Nuclear Officer
Power Authority of the State of
New York
123 Main Street
White Plains, NY 10601

SUBJECT: ISSUANCE OF EXEMPTION FROM THE REQUIREMENTS OF 10 CFR 70.24 - INDIAN
POINT NUCLEAR GENERATING UNIT NO. 3 (TAC NO. M97671)

Dear Mr. Knubel:

By letter dated December 20, 1996, as supplemented by letters dated March 5, 1997, and March 19, 1997, you requested an exemption from the requirements of the Code of Federal Regulations for the Indian Point Nuclear Generating Unit No. 3. With the requested exemption, you would not have to meet the requirements of 10 CFR 70.24.

Based upon the information provided, there is reasonable assurance that irradiated and unirradiated fuel will remain subcritical; furthermore, there is reasonable assurance that, should an inadvertent criticality occur, the licensee will detect such a criticality and workers will respond properly. Procedures, the use of a portable monitor, and training constitute good cause for granting an exemption to 10 CFR 70.24.

The Commission, pursuant to 10 CFR 70.14, has issued the enclosed exemption. The enclosed exempts you from the requirements of 10 CFR 70.24(a), (a)(1), and (a)(3) for the Indian Point Nuclear Generating Unit No. 3.

Sincerely,

George F. Wunder, Project Manager
Project Directorate I-1
Division of Reactor Projects - I/II
Office of Nuclear Reactor Regulation

Docket No. 50-286

Enclosure: Exemption

cc w/encl: See next page

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NAME	MYoung		SBjorn	ETenEyck	SVarge	RZimmerman
DATE	03/26/97		03/21/97	03/21/97	03/21/97	03/27/97
OFFICE	D: NRR					
NAME	SCollins					
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UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555-0001

March 27, 1997

Mr. James Knubel
Chief Nuclear Officer
Power Authority of the State of
New York
123 Main Street
White Plains, NY 10601

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Sincerely,

A handwritten signature in cursive script, appearing to read "George F. Wunder", is written over a horizontal line.

George F. Wunder, Project Manager
Project Directorate I-1
Division of Reactor Projects - I/II
Office of Nuclear Reactor Regulation

Docket No. 50-286

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cc w/encl: See next page

James Knubel
Power Authority of the State
of New York

Indian Point Nuclear Generating
Station Unit No. 3

cc:

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UNITED STATES OF AMERICA

NUCLEAR REGULATORY COMMISSION

In the Matter of)	
)	
POWER AUTHORITY OF THE STATE)	Docket No. 50-286
OF NEW YORK)	
)	
(Indian Point Nuclear Generating)	
Unit No. 3))	

EXEMPTION

I.

The Power Authority of the State of New York (the licensee) is the holder of Facility Operating License No. DPR-64, which authorizes operation of the Indian Point Nuclear Generating Unit No. 3 (IP3). The license provides that the licensee is subject to all rules, regulations, and orders of the Nuclear Regulatory Commission (the Commission) now or hereafter in effect.

The facility consists of a pressurized-water reactor at the licensee's site located in Westchester County, New York.

II.

The Code of Federal Regulations at subsection (a) of 10 CFR 70.24, "Criticality Accident Requirements," requires that each licensee authorized to possess special nuclear material shall maintain in each area where such material is handled, used, or stored, a criticality monitoring system "using gamma- or neutron-sensitive radiation detectors which will energize clearly audible alarm signals if accidental criticality occurs." Subsection (a)(1) of 10 CFR 70.24 specifies the detection, sensitivity, and coverage capabilities of the monitors required by 10 CFR 70.24(a). The specific requirements Subsection (a)(1) are that "the monitoring system shall be capable of detecting a criticality that produces an absorbed dose in soft tissue of 20 rads of combined neutron and gamma radiation at an unshielded distance of 2

meters from the reacting material within one minute." Subsection (a)(3) of 10 CFR 70.24 requires that the licensee shall maintain emergency procedures for each area in which this licensed special nuclear material is handled, used, or stored and provides (1) that the procedures ensure that all personnel withdraw to an area of safety upon the sounding of a criticality monitor alarm, (2) that the procedures must include drills to familiarize personnel with the evacuation plan, and (3) that the procedures designate responsible individuals for determining the cause of the alarm and placement of radiation survey instruments in accessible locations for use in such an emergency. Subsection (d) of 10 CFR 70.24 states that any licensee who believes that there is good cause why he should be granted an exemption from all or part of 10 CFR 70.24 may apply to the Commission for such an exemption and shall specify the reasons for the relief requested.

The purpose of 10 CFR 70.24(a), (a)(1), and (a)(3) is to ensure that any inadvertent criticality is detected and that action is taken to protect personnel and correct the problem. By letter dated December 20, 1996, as supplemented March 5, 1997, and March 19, 1997, the licensee requested an exemption from the requirements of 10 CFR 70.24. The licensee proposes to handle and store unirradiated fuel without having the criticality monitoring system specified in 10 CFR 70.24. The licensee also proposes to handle and store unirradiated fuel without the specific emergency procedures detailed in 10 CFR 70.24. The licensee believes that fuel handling procedures and design features make an inadvertent criticality unlikely. The licensee believes that a portable radiation monitoring system and existing plant procedures will provide adequate protection in the unlikely event of an accidental

criticality. The licensee also believes that current emergency procedures and training are adequate to meet the intent of 10 CFR 70.24(a)(3).

III.

Special nuclear material, as nuclear fuel, is stored in the spent fuel pool or the new (unirradiated) fuel storage racks. The spent fuel pool is used to store irradiated fuel under water after its discharge from the reactor, and new fuel prior to loading into the reactor. The new fuel racks are used to store new fuel in a dry condition upon arrival on site.

Special nuclear material is also present in the form of fissile material incorporated into fission chambers for nuclear instrumentation, primary source assemblies, and Health Physics calibration sources. The small quantity of special nuclear material present in these items precludes an inadvertent criticality.

Consistent with Technical Specification Section 5.4, the spent fuel pool is designed to store the fuel in a geometric array using a solid neutron absorber that precludes criticality. The spent fuel racks are designed such that the effective neutron multiplication factor, K_{eff} , will remain less than or equal to 0.95 under normal and accident conditions for fuel of maximum enrichment of 5.0 wt% U-235. The staff has found this design adequate.

The new fuel storage racks may be used to receive and store new fuel in a dry condition upon arrival on site and prior to loading in the reactor or spent fuel pool. The spacing between new fuel assemblies in the storage racks is sufficient to maintain the array in a subcritical condition even under accident conditions assuming the presence of moderator. The maximum enrichment of 5.0 wt% U-235 for the new fuel assemblies results in a maximum K_{eff} of less than 0.95 under conditions of accidental flooding. The staff has

found the design of the licensee's new fuel storage racks to be adequate to store fuel enriched to no greater than 5.0 wt% U-235.

Nuclear fuel is moved between the new fuel storage racks, the reactor vessel, and the spent fuel pool to accommodate refueling operations. In addition, fuel is moved into the facility and within the reactor vessel, or within the spent fuel pool. Fuel movements are procedurally controlled and designed to preclude conditions involving criticality concerns. Fuel handling procedures and the design features of the fuel handling system are discussed in the licensee's Final Safety Analysis Report.

Technical Specification Section 3.8 precludes certain movements of heavy loads over the spent fuel pool to prevent a fuel handling accident. Previous accident analyses have demonstrated that a fuel handling accident (i.e., a dropped fuel assembly) will not create conditions which could result in inadvertent criticality.

Procedures and controls prevent an inadvertent criticality during fuel handling; nevertheless the licensee will provide monitoring in the IP3 Fuel Storage Building during dry fuel handling operations. During dry fuel handling operations, the licensee will have in operation at least one portable detector that will meet the detection and sensitivity criteria of Sections 5.6 and 5.7 of ANSI/ANS 8.3 (1986), "American National Standard Criticality Accident Alarm System." Upon detection, this instrument shall automatically cause an immediate alarm audible in all areas from which evacuation is necessary to minimize exposure. The staff has determined that the detection and sensitivity criteria in the ANSI standard are as rigorous as those specified in 10 CFR 70.24(a)(1). The staff has also determined that, because fuel handling equipment design and procedures make a criticality unlikely, one

detector will be adequate and that in the case of fuel handling at IP3 two detectors as required by 10 CFR 70.24(a)(1) are not necessary.

The licensee has procedures and conducts training on dealing with radiological emergencies consistent with 10 CFR 50.47 and Part 50, Appendix E. In addition to this training, the licensee gives training on responding to a criticality monitor alarm to radiation workers accessing the fuel handling building. This training will be provided as necessary until dry fuel handling in 1997 is complete and the subject material has been incorporated into general employee training. The staff has determined that the licensee's procedures and training meet the intent of 10 CFR 70.24(a)(3); therefore, adherence to the specific requirements of this section is not necessary to serve the underlying purpose of the rule.

Because inadvertent criticality is precluded by both design and procedure, because adequate radiation monitoring is present, and because the licensee maintains emergency procedures for the areas in which fuel is handled, the staff has concluded that there is reasonable assurance that irradiated and unirradiated fuel will remain subcritical; furthermore, there is reasonable assurance that, should an inadvertent criticality occur, the licensee will detect such a criticality and workers will respond properly. The combination of plant design features, fuel handling procedures, the use of a portable criticality monitor, radiological emergency procedures and radiation worker training constitute good cause for granting an exemption to the requirements of 10 CFR 70.24.

IV.

Accordingly, the Commission has determined that, pursuant to 10 CFR 70.14, this exemption is authorized by law, will not endanger life or property

or the common defense and security, and is otherwise in the public interest.

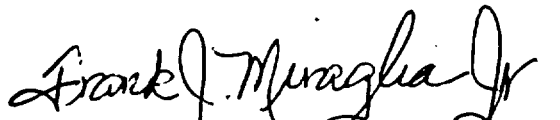
Therefore, the Commission hereby grants the following exemption:

The Power Authority of the State of New York is exempt from the requirements of 10 CFR 70.24(a), 10 CFR 70.24(a)(1), and 10 CFR 70.24(a)(3) for Indian Point Nuclear Generating Unit No. 3. This exemption is contingent on the facility's maintaining the hardware, procedure, and training described in Section III above.

Pursuant to 10 CFR 51.32, the Commission has determined that the granting of this exemption will have no significant impact on the quality of the human environment (62 FR 14705).

This exemption is effective upon issuance.

FOR THE NUCLEAR REGULATORY COMMISSION

A handwritten signature in black ink, reading "Frank J. Miraglia, Jr.", with a stylized flourish at the end.

Frank J. Miraglia, Jr., Acting Director
Office of Nuclear Reactor Regulation

Dated at Rockville, Maryland,
this 27th day of March 1997.

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Docket File

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