



UNITED STATES  
NUCLEAR REGULATORY COMMISSION

WASHINGTON, D.C. 20555-0001

December 7, 2000

Mr. T. F. Plunkett  
President, Nuclear Division  
Florida Power and Light Company  
P.O. Box 14000  
Juno Beach, Florida 33408-0420

SUBJECT: ST. LUCIE UNITS 1 AND 2 - ISSUANCE OF AMENDMENTS REGARDING  
RELOCATION OF EMERGENCY DIESEL GENERATOR MAINTENANCE  
INSPECTION TO LICENSEE CONTROLLED DOCUMENT (TAC NOS. MA9274  
AND MA9275)

Dear Mr. Plunkett:

The Commission has issued the enclosed Amendment Nos. **168** and **111** to Facility Operating Licenses Nos. DPR-67 and NPF-16 for the St. Lucie Plant, Units Nos. 1 and 2. These amendments consist of changes to a Technical Specification Surveillance Requirement (SR), in response to your application dated June 21, 2000.

These amendments relocate SR 4.8.1.1.2.e.1, regarding the emergency diesel generator (EDG) inspection program, to a licensee-controlled maintenance program that will be incorporated by reference into the next revision of the Updated Final Safety Analysis Report for each St. Lucie unit. Upon relocation to the licensee-controlled maintenance program, the effectiveness of maintenance on the EDGs and support systems will be monitored pursuant to the Maintenance Rule, Title 10, *Code of Federal Regulations* (10 CFR) Section 50.65.

A copy of the Safety Evaluation is also enclosed. The Notice of Issuance will be included in the Commission's biweekly Federal Register notice.

Sincerely,

Kahtan N. Jabbour, Senior Project Manager, Section 2  
Project Directorate II  
Division of Licensing Project Management  
Office of Nuclear Reactor Regulation

Docket Nos. 50-335 and 50-389

Enclosures:

1. Amendment No. **168** to DPR-67
2. Amendment No. **111** to NPF-16
3. Safety Evaluation

cc w/enclosures: See next page

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These amendments relocate SR 4.8.1.1.2.e.1, regarding the emergency diesel generator (EDG) inspection program, to a licensee-controlled maintenance program that will be incorporated by reference into the next revision of the Updated Final Safety Analysis Report for each St. Lucie unit. Upon relocation to the licensee-controlled maintenance program, the effectiveness of maintenance on the EDGs and support systems will be monitored pursuant to the Maintenance Rule, Title 10, *Code of Federal Regulations* (10 CFR) Section 50.65.

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/RA/

Kahtan N. Jabbour, Senior Project Manager, Section 2  
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UNITED STATES  
NUCLEAR REGULATORY COMMISSION  
WASHINGTON, D.C. 20555-0001

FLORIDA POWER & LIGHT COMPANY

DOCKET NO. 50-335

ST. LUCIE PLANT UNIT NO. 1

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 168  
License No. DPR-67

1. The Nuclear Regulatory Commission (the Commission) has found that:
  - A. The application for amendment by Florida Power & Light Company (the licensee), dated June 21, 2000, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act) and the Commission's rules and regulations set forth in 10 CFR Chapter I;
  - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
  - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
  - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
  - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

2. Accordingly, Facility Operating License No. DPR-67 is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and by amending paragraph 2.C.(2) to read as follows:

(2) Technical Specifications

The Technical Specifications contained in Appendices A and B, as revised through Amendment No. 168, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. The license amendment is effective as of its date of issuance and shall be implemented at the facility within 60 days of the date of issuance. In addition, the licensee shall include the relocated information in the next Updated Final Safety Analysis Report (UFSAR) submitted to the NRC, pursuant to 10 CFR 50.71(e), as was described in the licensee's application dated June 21, 2000 and evaluated in the staff's safety evaluation dated ~~December~~ 7, 2000.

Before the relocated information is included in the UFSAR, the licensee shall control changes to it in accordance with the provisions of 10 CFR 50.59.

FOR THE NUCLEAR REGULATORY COMMISSION



Richard P. Correia, Chief, Section 2  
Project Directorate II  
Division of Licensing Project Management  
Office of Nuclear Reactor Regulation

Attachment:  
Changes to the Technical  
Specifications

Date of Issuance: ~~December~~ 7, 2000

ATTACHMENT TO LICENSE AMENDMENT NO. 168

TO FACILITY OPERATING LICENSE NO. DPR-67

DOCKET NO. 50-335

Replace the following page of the Appendix A Technical Specifications with the attached page. The revised page is identified by amendment number and contains a marginal line indicating the area of change.

Remove Page

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Insert Page

3/4 8-5

## **ELECTRICAL POWER SYSTEMS**

### **SURVEILLANCE REQUIREMENTS (Continued)**

- c. By sampling new fuel in accordance with ASTM D4057-81 prior to addition to the storage tanks and:
  - 1. By verifying in accordance with the tests specified in ASTM D975-81 prior to addition to the storage tanks that the sample has:
    - a) API Gravity within 0.3 degrees at 60°F or a specific gravity of within 0.0016 at 60/60°F, when compared to the supplier's certificate or an absolute specific gravity at 60/60°F of greater than or equal to 0.83 but less than or equal to 0.89 or an API gravity of 60°F of greater than or equal to 27 degrees but less than or equal to 39 degrees.
    - b) A kinematic viscosity at 40°C of greater than or equal to 1.9 centistokes, but less than or equal to 4.1 centistokes, if gravity was not determined by comparison with the supplier's certification.
    - c) A flash point equal to or greater than 125°F, and
    - d) A clear and bright appearance with proper color when tested in accordance with ASTM D4176-82.
  - 2. By verifying within 31 days of obtaining the sample that the other properties specified in Table 1 of ASTM D975-81 are met when tested in accordance with ASTM D975-81 except that the analysis for sulfur may be performed in accordance with ASTM D1552-79 or ASTM D2622-82.
- d. At least once every 31 days by obtaining a sample of fuel oil from the storage tanks in accordance with ASTM D2276-83 and verifying that total particulate contamination is less than 10 mg/liter when checked in accordance with ASTM D2276-83, Method A, or Annex A-2.
- e. At least once per 18 months during shutdown by:
  - 1. DELETED
  - 2. Verifying generator capability to reject a load of greater than or equal to 600 hp while maintaining voltage at  $4160 \pm 420$  volts and frequency at  $60 \pm 1.2$  Hz.
  - 3. Simulating a loss of offsite power by itself, and:
    - a) Verifying deenergization of the emergency busses and load shedding from the emergency busses.



UNITED STATES  
NUCLEAR REGULATORY COMMISSION  
WASHINGTON, D.C. 20555-0001

FLORIDA POWER & LIGHT COMPANY

ORLANDO UTILITIES COMMISSION OF

THE CITY OF ORLANDO, FLORIDA

AND

FLORIDA MUNICIPAL POWER AGENCY

DOCKET NO. 50-389

ST. LUCIE PLANT UNIT NO. 2

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 111  
License No. NPF-16

1. The Nuclear Regulatory Commission (the Commission) has found that:
  - A. The application for amendment by Florida Power & Light Company, et al. (the licensee), dated June 21, 2000, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act) and the Commission's rules and regulations set forth in 10 CFR Chapter I;
  - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
  - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
  - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
  - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

2. Accordingly, Facility Operating License No. NPF-16 is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and by amending paragraph 2.C.2 to read as follows:

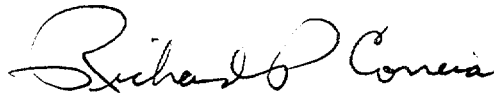
2. Technical Specifications

The Technical Specifications contained in Appendices A and B, as revised through Amendment No. **111**, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. The license amendment is effective as of its date of issuance and shall be implemented at the facility within 60 days of the date of issuance. In addition, the licensee shall include the relocated information in the next Updated Final Safety Analysis Report (UFSAR) submitted to the NRC, pursuant to 10 CFR 50.71(e), as was described in the licensee's application dated June 21, 2000 and evaluated in the staff's safety evaluation dated **December 7, 2000**.

Before the relocated information is included in the UFSAR, the licensee shall control changes to it in accordance with the provisions of 10 CFR 50.59.

FOR THE NUCLEAR REGULATORY COMMISSION



Richard P. Correia, Chief, Section 2  
Project Directorate II  
Division of Licensing Project Management  
Office of Nuclear Reactor Regulation

Attachment:  
Changes to the Technical  
Specifications

Date of Issuance: **December 7, 2000**



ATTACHMENT TO LICENSE AMENDMENT NO. 111

TO FACILITY OPERATING LICENSE NO. NPF-16

DOCKET NO. 50-389

Replace the following page of the Appendix A Technical Specifications with the attached page. The revised page is identified by amendment number and contains a marginal line indicating the area of change.

Remove Page

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Insert Page

3/4 8-5

## **ELECTRICAL POWER SYSTEMS**

### **SURVEILLANCE REQUIREMENTS (continued)**

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- c. By sampling new fuel all in accordance with ASTM D4057-81 prior to addition to the storage tanks and:
  - 1. By verifying in accordance with the tests specified in ASTM D975-81 prior to addition to the storage tanks that the sample has:
    - a) An API Gravity of within 0.3 degrees at 60°F or a specific gravity of within 0.0016 at 60/60°F, when compared to the supplier's certificate or an absolute specific gravity at 60/60°F of greater than or equal to 0.83 but less than or equal to 0.89 or an API gravity of 60°F of greater than or equal to 27 degrees but less than or equal to 39 degrees.
    - b) A kinematic viscosity at 40°C of greater than or equal to 1.9 centistokes, but less than or equal to 4.1 centistokes, if gravity was not determined by comparison with the supplier's certification.
    - c) A flash point equal to or greater than 125°F, and
    - d) A clear and bright appearance with proper color when tested in accordance with ASTM D4176-82.
  - 2. By verifying within 31 days of obtaining the sample that the other properties specified in Table 1 of ASTM D975-81 are met when tested in accordance with ASTM D975-81 except that the analysis for sulfur may be performed in accordance with ASTM D1552-79 or ASTM D2622-82.
- d. At least once every 31 days by obtaining a sample of fuel oil from the storage tanks in accordance with ASTM D2276-83 and verifying that total particulate contamination is less than 10 mg/liter when checked in accordance with ASTM D2276-83, Method A, or Annex A-2.
- e. At least once per 18 months during shutdown by:
  - 1. DELETED
  - 2. Verifying generator capability to reject a load of greater than or equal to 453 kW while maintaining voltage at  $4160 \pm 420$  volts and frequency at  $60 \pm 1.2$  Hz.
  - 3. Verifying the generator capability to reject a load of 3685 kW without tripping. The generator voltage shall not exceed 4784 volts during and following the load rejection.



UNITED STATES  
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WASHINGTON, D.C. 20555-0001

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION

RELATED TO AMENDMENT NOS. 168 AND 111

TO FACILITY OPERATING LICENSES NOS. DPR-67 AND NPF-16

FLORIDA POWER AND LIGHT COMPANY, ET AL.

ST. LUCIE PLANT, UNITS NOS. 1 AND 2

DOCKET NOS. 50-335 AND 50-389

1. INTRODUCTION

By letter dated June 21, 2000, Florida Power & Light Company (FPL or the licensee) proposed to relocate Technical Specification (TS) Surveillance Requirement (SR) 4.8.1.1.2.e.1, regarding the emergency diesel generator (EDG) inspection program, to a licensee-controlled maintenance program that will be incorporated by reference into the next revision of the Updated Final Safety Analysis Report (UFSAR) for each St. Lucie unit. Upon relocation to the licensee-controlled maintenance program, the SR to perform the EDG inspections every 18 months during shutdown would be eliminated. However, the effectiveness of the maintenance on the EDGs and support systems will be monitored pursuant to the Maintenance Rule, Title 10 of the *Code of Federal Regulations* (10 CFR), Section 50.65.

2. EVALUATION

Each St. Lucie unit is equipped with two seismically qualified, Class 1E EDGs to provide onsite emergency ac power to essential safety systems in the event of a loss of offsite power. The licensee conforms to the Station Blackout (SBO) Rule, 10 CFR 50.63. To ensure that EDG reliability remains greater than or equal to the target reliability associated with the SBO Rule, FPL maintains an EDG reliability program for St. Lucie Units 1 and 2 based on Regulatory Guide 1.155. The program monitors and evaluates EDG performance and reliability, requires remedial actions if one or more reliability values are not satisfied, requires a root-cause evaluation and corrective actions for individual EDG failure, and monitors EDG unavailability. In addition to FPL's reliability program, the effectiveness of maintenance on the EDGs and support systems are monitored pursuant to the Maintenance Rule, 10 CFR 50.65.

The requirements of the existing SR 4.8.1.1.2.e.1 would be relocated to a licensee-controlled maintenance program for the EDGs. The licensee-controlled maintenance program would be incorporated by reference into the next revision of the UFSAR for each unit. Changes to licensee-controlled EDG maintenance program would be controlled under 10 CFR 50.59. Under the licensee-controlled maintenance program, the EDGs would be inspected in accordance with procedures prepared in conjunction with the manufacturer's recommendations for the class of standby service. Upon relocation to the licensee-controlled maintenance

program, the requirement to perform the EDG inspections every 18 months during shutdown would be eliminated. However, the effectiveness of the maintenance on the EDGs and support systems would be monitored pursuant to the Maintenance Rule, 10 CFR 50.65.

Based on its review of the licensee's submittal, the staff concludes that: (1) changes to the licensee-controlled EDG maintenance program will be controlled under 10 CFR 50.59; (2) under the licensee controlled maintenance program, the EDGs will be inspected in accordance with procedures prepared in conjunction with the manufacturer's recommendations for the class of standby service provided by the EDGs; and (3) the proposed change is consistent with the Improved Standard Technical Specifications. Therefore, the staff finds that the relocation of SR 4.8.1.1.2.e.1 to a licensee-controlled maintenance program is acceptable.

The staff notes that the licensing basis for the EDG inspections that are the subject of these amendments provides for inspections during shutdown. Upon issuance of these amendments, any change to this portion of the licensing basis will be controlled by the requirements of 10 CFR 50.59.

### 3.0 STATE CONSULTATION

Based upon a letter dated March 8, 1991, from Mary E. Clark of the State of Florida, Department of Health and Rehabilitative Services, to Deborah A. Miller, Licensing Assistant, U.S. Nuclear Regulatory Commission, the State of Florida does not desire notification of issuance of license amendments.

### 4.0 ENVIRONMENTAL CONSIDERATION

These amendments change a surveillance requirement. The NRC staff has determined that the amendments involve no significant increase in the amounts, and no significant change in the types, of any effluents that may be released offsite, and that there is no significant increase in individual or cumulative occupational radiation exposure. The Commission has previously issued a proposed finding that the amendments involve no significant hazards consideration and there has been no public comment on such finding (65 FR 48750). Accordingly, these amendments meet the eligibility criteria for categorical exclusion set forth in 10 CFR 51.22(c)(9). Pursuant to 10 CFR 51.22(b) no environmental impact statement or environmental assessment need be prepared in connection with the issuance of these amendments.

### 5.0 CONCLUSION

The Commission has concluded, based on the considerations discussed above, that: (1) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, (2) such activities will be conducted in compliance with the Commission's regulations, and (3) the issuance of the amendments will not be inimical to the common defense and security or to the health and safety of the public.

Principal Contributor: N. Trehan, NRR

Date: December 7, 2000

Mr. T. F. Plunkett  
Florida Power and Light Company

**ST. LUCIE PLANT**

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