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OCTOBER 5-7, 2000

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REPORTS, LETTERS, AND MEMORANDA

REPORT

- Union of Concerned Scientists Report, "Nuclear Plant Risk Studies: Failing the Grade" (Report to Richard A. Meserve, Chairman, NRC, from Dana A. Powers, Chairman, ACRS, dated October 11, 2000)

LETTER

- Pressurized Thermal Shock Technical Basis Reevaluation Project (Letter to William D. Travers, Executive Director for Operations, NRC, from Dana A. Powers, Chairman, ACRS, dated October 12, 2000)

MEMORANDUM

- Proposed Revision to 10 CFR 73.55, "Requirements for Physical Protection of Licensed Activities in Nuclear Power Reactors Against Radiological Sabotage" (Memorandum to William D. Travers, Executive Director for Operations, NRC, from John T. Larkins, Executive Director, ACRS, dated October 11, 2000)

APPENDICES

- I. Federal Register Notice
- II. Meeting Schedule and Outline
- III. Attendees
- IV. Future Agenda and Subcommittee Activities
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CERTIFIED
MINUTES OF THE 476TH MEETING OF THE
ADVISORY COMMITTEE ON REACTOR SAFEGUARDS
October 5-7, 2000
ROCKVILLE, MARYLAND

The 476th meeting of the Advisory Committee on Reactor Safeguards (ACRS) was held in Conference Room 2B3, Two White Flint North Building, Rockville, Maryland, on October 5-7, 2000. Notice of this meeting was published in the *Federal Register* on September 20, 2000 (65 FR 56945) (Appendix I). The purpose of this meeting was to discuss and take appropriate action on the items listed in the meeting schedule and outline (Appendix II). The meeting was open to public attendance. There were no written statements or requests for time to make oral statements from members of the public regarding the meeting.

A transcript of selected portions of the meeting was kept and is available in the NRC Public Document Room at the One White Flint North Building, MS 1F-15, 11555 Rockville Pike, Rockville, MD 20852-2738. [Copies of the transcript are available for purchase from Ann Riley & Associates, Ltd., 1025 Connecticut Avenue, N.W., Suite 1014, Washington, D.C. 20036, and on the ACRS/ACNW Web page at (www.NRC.gov/ACRS/ACNW).]

ATTENDEES

ACRS Members: Dr. Dana A. Powers (Chairman), Dr. George Apostolakis (Vice Chairman), Dr. Mario V. Bonaca, Dr. Thomas S. Kress, Mr. Graham M. Leitch, Dr. William J. Shack, Dr. Robert L. Seale, Mr. John D. Sieber, Dr. Robert E. Uhrig, and Dr. Graham B. Wallis. For a list of other attendees, see Appendix III.

I. Chairman's Report (Open)

[Note: Dr. John T. Larkins was the Designated Federal Official for this portion of the meeting.]

Dr. Dana A. Powers, Committee Chairman, convened the meeting at 8:30 a.m. and reviewed the schedule for the meeting. He summarized the agenda topics for

this meeting and discussed the administrative items for consideration by the full Committee.

II. Discussion of Union of Concerned Scientists Report, "Nuclear Plant Risk Studies: Failing the Grade" (Open)

[Note: Mr. Michael T. Markley was the Designated Federal Official for this portion of the meeting.]

Dr. George Apostolakis, Chairman of the ACRS Subcommittee on Reliability and Probabilistic Risk Assessment introduced the topic. He stated that the purpose of this meeting was to review the Union of Concerned Scientists (UCS) August 2000 report entitled, "Nuclear Plant Risk Studies: Failing the Grade," concerning the use of risk information in NRC decision making. He stated that the UCS report was critical of the NRC's use of risk information in regulatory matters and noted that the ACRS has been supportive of initiatives related to risk-informed regulation. He introduced Mr. David Lochbaum, nuclear safety engineer at UCS, and noted that the Committee had met previously with Mr. Lochbaum during the October 5-7, 2000 ACRS meeting, to discuss this matter.

UCS Presentation

Mr. Lochbaum discussed the issues raised in the report as well as feedback that UCS has received from interested parties. He informed the Committee that the UCS report has been criticized for using obsolete results from outdated Individual Plant Examinations (IPEs). Mr. Lochbaum stated that UCS would prefer to have used updated probabilistic risk assessments (PRAs) and noted that licensee PRAs are not available for public review. UCS also noted that the same IPEs were used in the development of the significance determination process (SDP) of the revised reactor oversight process RROP. The Committee also discussed the UCS concern over differing PRA results for plants of like design or "sister" plants.

Mr. Lochbaum presented several slides taken from the NRC system reliability studies completed by the Office of Nuclear Regulatory Research (RES) (studies completed by the former Office for Analysis and Evaluation of Operational Data). He highlighted aspects of these studies (e.g., isolation condenser, high pressure core injection, reactor core isolation cooling systems, etc.) to illustrate the UCS view that the IPEs are seriously flawed in that they fail to estimate accurately and conservatively system unreliability. UCS contends that there is system-level bias in all these studies, in part, due to industry partitioning and rounding of operational data. UCS concluded that these inaccuracies and non-conservative estimates form an unsuitable foundation for regulatory decision making.

NRC Staff Comments

At the conclusion of the briefing, Messrs. Richard Barrett and Gareth Parry, NRR, provided comments on the UCS presentation and associated report. The staff acknowledged that there is a lot of information in the UCS report that is technically correct and took exception with the UCS assertion that NRC decisions have not been judicious. It was stated that lessons from past operating experience as well as issues related to PRA quality are being and will continue to be addressed. Identified discrepancies with "sister" plants are being evaluated (e.g., Wolf Creek and Callaway, etc.) and that the staff plans to examine the results of licensee/industry peer review processes as they become available. The staff also stated that IPEs were used as a first step in getting the SDP process started as a screening tool for inspection. The staff informed the Committee that they expect the SDP to become more plant-specific as experience is gained in using the SDP and in more closely examining PRAs related to plant operating experience.

Dr. Apostolakis questioned whether UCS viewed PRA as a weak, immature technology. Dr. Bonaca questioned whether UCS dislikes the use or treatment of bottom-line numbers (i.e., quantitative analysis). Mr. Lochbaum stated that the major concern is that the control over the inputs, assumptions, and methodology would not prevent abuses, whether institutional or accidental. Dr. Apostolakis questioned whether the NRC's use of risk information had led to incorrect or improper decisions. Mr. Lochbaum stated that he is not suggesting that the "risk-informed road" is wrong, just that the road should be fixed before going down it. Dr. Apostolakis noted that there is a dichotomy between Mr. Lochbaum's statements and the messages codified in the UCS report and agreed that key is the validity of regulatory decision making.

Dr. Bonaca noted that the trend curve presented by Mr. Lochbaum was encouraging in that it provided evidence of a decline in design-basis related accident sequence precursor (ASP) events, in part, due to the increased attention to design basis issues from 1995-1997. Mr. Lochbaum stated that the trend chart does not indicate that there were no risk-significant design-basis issues. Concerning the study of high pressure core injection reliability, Mr. Lochbaum noted that some data was outside the error band and questioned the use of generic data. Dr. Apostolakis agreed that plant-specific data should be used. Mr. Lochbaum also noted that many other parts of the NRC are unaware of the NRC reliability studies. Members of the Committee agreed and noted that the ACRS has previously expressed similar concerns regarding the awareness and limited use of these studies.

Dr. Powers questioned how the public can gain access to plant-specific risk analysis in order to have confidence. Mr. Lochbaum stated that it would have to be entered into the licensing docket for the individual plant and acknowledged that the information can be voluminous and difficult to review. Dr. Apostolakis suggested that the critical information to consider is the risk analysis which supports a particular regulatory decision and/or application.

Conclusion

The Committee provided a report to Chairman Meserve dated October 11, 2000, on this matter.

III. NEI 00-02, "Industry PRA Peer Review Process Guidelines" (Open)

[Note: Mr. Michael T. Markley was the Designated Federal Official for this portion of the meeting.]

Dr. George Apostolakis, Chairman of the ACRS Subcommittee on Reliability and Probabilistic Risk Assessment, introduced the topic. He stated that the purpose of this meeting was to review the proposed industry certification process described in the document NEI 00-02, "Industry PRA Peer Review Process Guidelines."

Industry Presentation

Messrs. Biff Bradley of the Nuclear Energy Institute (NEI) and Karl Fleming of ERIN Engineering, Inc., led the discussion concerning NEI 00-02. Mr. Bradley provided an overview of NEI 00-02, including the industry's planned use of the peer review results to facilitate focused NRC review (i.e., as a template for NRC review) of risk-informed revisions to the special treatment requirements of 10 CFR Part 50 (Option 2). Mr. Fleming discussed a case study involving the Byron/Braidwood nuclear power plants. Significant points made during the presentation include:

- NEI 00-02 was developed from the peer review process initially developed by the BWR Owners Group.
- The certification process does not provide an overall grade for PRAs, and can be used as a complement to or in lieu of industrial standards for PRA quality. The industry supports development of a PRA Standard(s), however, an overly prescriptive Standard cannot ensure quality. Similarly,

a Standard cannot obviate the need for NRC review of specific applications.

- PRA is inherently judgmental. Therefore, a high-quality peer review will always be required.
- The peer review process will normally involve 6-7 reviewers with detailed knowledge of nuclear power plants and associated PRAs. It involves an evaluation of PRA elements and sub-elements via a structured process including the use of checklists to ensure a consistent, comprehensive review.
- The process will normally involve 2-3 person-months of effort, including document reviews, onsite review, and preparation of the final report. The first case study was for the Byron and Braidwood nuclear stations. All NSSS owners groups have committed to implementing NEI 00-02.
- The industry recognizes that some existing PRAs need to be improved in order to support regulatory decision making. NEI agreed with the UCS criticism that there is a need to improve industry PRAs.

At the conclusion of the meeting, NEI invited the ACRS and NRC staff to observe and/or participate in the peer review process in order to gauge the rigor applied to licensee risk-informed decisions using the peer review panel.

Dr. Apostolakis questioned whether licensees would request, using lower-level certified PRAs, regulatory decisions where the PRA is not sufficiently broad. He suggested that a lot of debate could be avoided if licensees would pursue certifying a Grade or Category 3 PRA rather than settling for limited approaches at the Grades 1 and 2 level. Mr. Fleming stated that resolving the A- and B-Type issues will require getting to Grade 3, and that the main focus was to benchmark the current PRAs, identify what information is needed to upgrade those PRAs, and decide on what needs to be done for additional applications.

Dr. Apostolakis requested to review a copy of the Byron and Braidwood case study documents. The NRC staff agreed to provide the docketed materials associated with this matter.

Conclusion

This briefing was for information only. No Committee report was required.

IV. Staff Views on ASME Standard for PRA for Nuclear Power Plant Applications
(Open)

[Mrs. Maggalean W. Weston was the Designated Federal Official for this portion of the meeting.]

Dr. George Apostolakis, Chairman of the ACRS Subcommittee on Probabilistic Risk Assessment, introduced this topic. He indicated that the Committee had reviewed the ASME Standard, however, did not have the opportunity to review the staff's comments on the standard.

NRC Staff Presentation

The presentation on the Staff Views of ASME Standard for PRA for Nuclear Power Plant Applications was made by Ms. Mary Drouin, RES and Mr. Gareth Parry, NRR. The staff comments were on Revision 12 of the draft Standard. SECY 00-0162 was used as guidance in formulating staff comments on the draft. The issue of PRA quality is central to risk-informed regulations and one that the Commission has continually raised with the staff. As a result of review of the draft standard, the staff concluded that the current version of the standard (1) does not address PRA quality, (2) is difficult to use in determining where there are strengths and weaknesses in the PRA results, (3) will provide limited assistance to staff in performing focused review of PRA submittals, and (4) will provide minimal assistance in making more efficient use of NRC resources.

Following this discussion was a chapter by chapter discussion of the major concerns that led to the conclusions. Among the concerns discussed were the definition of the categories within which the PRA applications would fit, the requirements for the risk assessment application process, the completeness and accuracy of the technical content of the document, and the changes needed to be made to the current document.

Conclusion

This briefing was for information only. No committee action was required.

V. Pressurized Thermal Shock Technical Bases Reevaluation Project (Open)

[Note: Mr. Noel F. Dudley was the Designated Federal Official for this portion of the meeting.]

Mr. William Shack, Chairman of the Materials and Metallurgy Subcommittee, stated that the Subcommittee had a very good and full discussion of the status of the Pressurized Thermal Shock (PTS) Technical Basis Reevaluation Project at its September 21, 2000 meeting. He explained that the staff would walk through the overall probabilistic fracture mechanics calculation to show the Committee how all the pieces of the Reevaluation Project fit together and would provide a more detailed description of the development of a fracture toughness distribution. Dr. Shack mentioned that the source term used in developing the PTS screening criterion may be inappropriate in that assumes steam oxidation of the fuel while PTS will result in air oxidation of the fuel.

Mr. Michael Mayfield, RES, explained that three different divisions in RES were working on the PTS Reevaluation Project and that the Reevaluation Project would continue over the next year. Mr. Terry Dickson, Oak Ridge National Laboratory, explained how the results of the Fracture Analysis of Vessels: Oak Ridge (FAVOR) code can be applied to the PTS Reevaluation Project.. He described the integration into the FAVOR code of the results of different evolving technologies, such as, embrittlement correlations and fracture toughness models. He explained the structure of the code and the overall methodology for incorporating probabilistic risk assessment information.

The ACRS Members and the staff discussed the effects of lowering the present acceptance value, the effects of surface breaking cracks, existence of a possible thermal plume in the hot leg downcomer, and neutron fluence maps.

Mr. Mark Kirk, RES, provided a status of the development of fracture toughness distributions and the associated uncertainty analysis. He explained that the goal of the activity is to characterize toughness using all available data in a way that is consistent with PRAs. He described the approach used and the addition of new data to the database. Mr. Kirk outlined the uncertainty framework and he summarized some of the initial results and the on-going activities.

The ACRS Members and the staff discussed the appropriateness of using a Weibull to characterize the fracture toughness data and a one dimensional finite analysis to characterize crack distributions. They also discussed the effects of neutron fluence, characterization of flaws, damage accumulation, and calculating thermal-hydraulic uncertainties.

Conclusion

The Committee provided a letter dated October 12, 2000, to the EDO on this matter.

VI. Discussion of Industry Issues (Open)

[Note: Dr. Richard P. Savio was the Designated Federal Official for this portion of the meeting]

The Committee met with Ralph Beedle, NEI Senior Vice President and Chief Nuclear Officer, Nuclear Generation Division and members of his staff to discuss NEI's current regulatory initiatives, emergency industry issues, NEI recent reorganization, and other issues of mutual interest. The mutual interest issues included: risk-informing 10 CFR Part 50; license renewal; decommissioning; and the revised reactor oversight process.

NEI representatives frequently meet with the ACRS to discuss particular regulatory issues. These discussions focus on strategic planning, regulatory philosophy, and ACRS and NEI views on emergency issues.

Conclusion

The discussions were informative and productive and the ACRS Chairman plans to schedule this type of stakeholder exchange on a regular basis.

VII. GSI-168, Equipment Qualification (Open)

[Mr. Amarjit Singh was the Designated Federal Official for this portion of the meeting.]

Dr. Robert E. Uhrig, Chairman of the ACRS Subcommittee on Plant Systems, introduced this topic. He stated that the safety-related electrical cables, primarily low voltage as well as medium voltage, associated with transmission of signals from plant to I&C devices and instruments conduct power to safety-related devices. The environmental qualification issue started in 1971. He stated that the purpose of this briefing was to discuss the proposed resolution and the status of Generic Safety Issue (GSI) 168, "Environmental Qualification (EQ) of Low-Voltage Instrumentation and Control (I&C Cables)."

NRC Staff Presentation

Mr. Mike Mayfield led the staff's discussion of GSI-168 and stated that the staff is not prepared to present the closure package to the Committee. Mr. Edward Hackett presented a brief technical and regulatory background of the research results for resolution of GSI 168. RES sponsored the research program to resolve issues related to the process used for qualification of certain electric components used in commercial nuclear power plants. Brookhaven National Laboratory (BNL) was selected as the lead laboratory to provide the technical assistance to RES. The objective of this research program was to provide information to assist the staff to resolve specific issues related to the EQ process for low-voltage I&C electric cables. Initially, a comprehensive list of 43 issues was identified. Based on a thorough review and analysis of the literature, 24 issues were resolved by considering past research results, and 19 issues were unresolved. Of the latter, 6 issues were identified and required additional testing of the cables to resolve. These issues are summarized below:

- How do the properties of cables subjected to accelerated aging techniques used in the original qualification compare with the properties of naturally aged cables of equivalent age?
- What are the limitations in using an estimated value of the activation energy to predict the chemical degradation during thermal aging?
- Do multiconductor cables have different failure mechanisms than single conductor cables, and, if so, are these unique failure mechanisms properly accounted for in the qualification process?
- Do bonded jacket cables have different failure mechanisms than unbonded jacket cables, and, if so, are these unique failure mechanism properly accounted for in the qualification process?
- Are there any effective condition monitoring techniques for determining cable condition in situ?
- Can condition monitoring techniques be used to predict loss-of-coolant accident (LOCA) survivability?

Mr. Hackett concluded that the research test results suggested that some of the cable types would not function during a LOCA after 40 years of service. Most of them would not survive after 60 years of service, if they are operated at rated

temperatures. Further, the staff concluded that risk studies give relatively high CDF values, conditioned on all of cables failing during LOCA. Additionally, research data would not support a sufficiently low failure rate to reduce the CDF values to an acceptably low level.

Conclusion

This briefing was for information only and no Committee action was required on this matter.

VIII. ACRS Review of Generic Guidance Documents Associated with License Renewal (Open)

[Note: Mr. Noel F. Dudley was the Designated Federal Official for this portion of the meeting.]

Dr. Bonaca, Chairman of the Plant License Renewal Subcommittee, noted that the staff and industry were developing a set of guidance documents for preparing and reviewing license renewal applications. He explained that the Subcommittee planned to review these documents during the October 19-20, 2000 ACRS subcommittee meeting. The members identified and discussed questions resulting from their initial reviews of the documents.

Conclusion

This session was for information only.

IX. Annual Report to the Commission on the NRC Safety Research Program

[Note: Dr. Medhat El-Zeftawy was the Designated Federal Official for this portion of the meeting.]

The Committee continued its discussion of the NRC Safety Research Program and the format and content of the ACRS 2001 report. The Committee indicated that the focus of its report will be on the long-term research needed to facilitate the execution of the NRC's mission in the future. The Committee also discussed certain acceptance criteria that could be useful in evaluating the research programs. The acceptance criteria may include metrics such as regulatory needs,

objectives, level of effort, probability of success, why NRC should perform such research programs and not the industry, and public confidence.

Conclusion

The Committee will continue its discussion and preparation of the ACRS 2001 report to the Commission on the NRC research programs during future ACRS meetings. A subcommittee meeting on the report has been scheduled for November 1, 2000.

X. Executive Session (Open)

[Note: Dr. John T. Larkins was the Designated Federal Official for this portion of the meeting.]

A. Reconciliation of ACRS Comments and Recommendations (Open)

[Note: Mr. Sam Duraiswamy was the Designated Federal Official for this portion of the meeting.]

- The Committee discussed the response from the EDO, dated August 31, 2000, to ACRS comments and recommendations included in the ACRS letter dated July 20, 2000, concerning the proposed final ASME Standard for PRA for Nuclear Power Plant Applications.

The Committee decided that it was satisfied with the EDO's response.

B. Report on the Meeting of the Planning and Procedures Subcommittee (Open)

The Committee heard a report from Dr. Powers and the Executive Director, ACRS, on the Planning and Procedures Subcommittee meeting held on October 4, 2000. The following items were discussed:

Review of the Member Assignments and Priorities for ACRS Reports and Letters for the October ACRS Meeting

Member assignments and priorities for ACRS reports and letters for the October ACRS meeting are included in a separate handout. Reports and letters that would benefit from additional consideration at a future ACRS meeting was discussed.

Anticipated Workload for ACRS Members

The anticipated workload of the ACRS members through December 2000 was discussed. The objectives are to:

- Review the reasons for the scheduling of each activity and the expected work product and to make changes, as appropriate
- Manage the members' workload for these meetings
- Plan and schedule items for ACRS discussion of topical and emerging issues.

Issues Associated with Core Power Upgrades

The Planning and Procedures Subcommittee discussed the central issues pertaining to core power upgrades, especially those identified by an ACRS Senior Fellow, Dr. Cronenberg [Note: There has been outside interest regarding the report prepared by Dr. Cronenberg]. In addition, the Subcommittee discussed a proposed White paper prepared by Dr. Bonaca on "Potential Synergistic Effects of Industry Initiatives to Extend Plant Life, Increase Production, and Reduce Regulatory Burden." The full Committee agreed with the Subcommittee's recommendation that the issues raised by Dr. Bonaca in his White paper be included in the ensuing ACRS report to the Commission on the NRC Safety Research Program. Also, the Subcommittee decided to continue its discussion of the issues associated with the core power upgrades and related matters during its October 4, 2000 meeting. The staff will brief the Committee during the December 2000 ACRS meeting. Proposed topics include:

- NRR position regarding the need for applying risk-informed decisionmaking to "significant" power upgrade applications. Criteria for determining "special circumstances."
- NRR plans for revising guidance documents and developing a Standard Review Plan Section for power upgrade reviews.
- Synergisms among changes in nuclear plants and their potential impact on plant safety.
- Higher burnup fuel
- Power upgrades

- Use of “best-estimate” or “more-realistic” analyses
- Plant life extension
- RES activities associated with core power uprates.

Mixed Oxide Fuel Fabrication Facility

The Department of Energy (DOE) plans to construct a Mixed Oxide (MOX) Fuel Fabrication Facility (FFF) on its Savannah River site to manufacture fuel from weapons program plutonium. A consortium composed of Duke Power, Cogema Fuels, and Stone & Webster (DCS) will construct and operate the facility for DOE. The NRC staff expects the application for construction authorization to be submitted in December 2000 and the operating license application in June 2002. NRC staff review of the safety design basis for the MOX FFF will occur at the construction authorization stage. The NRC staff also expects DCS to submit the license amendment for loading MOX fuel test assemblies in McGuire Unit 2 in August 2001. A license amendment for batch loading of MOX fuel in the McGuire and Catawba reactors is expected in December 2003. The Commission is interested in the ACRS review of and comment on the MOX FFF. Review of the safety issues associated with the use of MOX fuel in operating reactors is within the purview of the ACRS.

Meeting with the Nuclear Energy Institute

As part of the CY 1998 and CY 1999 ACRS self assessment, feedback was solicited from various stakeholders, including some industry groups. Some stakeholders stated that the ACRS was not sufficiently informed of the industry's concerns and needs. As a result, a meeting with representatives of NEI was scheduled for the October 5-7, 2000 ACRS meeting to discuss items of mutual interest. Topics agreed to by NEI are provided below:

- Risk informing 10 CFR Part 50)
- License renewal
- Decommissioning
- Revised reactor oversight process

Estimation of Resources for FY 2001

Due to the anticipated high workload facing the ACRS in FY 2001, it is important to plan how to use member time most efficiently and effectively.

Assuming the number of ACRS members remain constant throughout FY 2001, the maximum member time that will be available is 1,300 days.

During last month's Planning and Procedures Subcommittee meeting, we discussed the need to manage better the number of subcommittee meetings and the number of members participating in subcommittee meetings. Senior staff engineers with input from Subcommittee chairmen were asked to revise the estimate of the number of subcommittee meetings for FY 2001. The current estimate shows 36 subcommittee meetings, 10 full Committee meetings and 1 retreat, consuming a total of approximately 1,155 days. The Planning and Procedures subcommittee will need to scrutinize these proposed subcommittee meetings to assess where some cuts might be made or combining of subcommittee meetings might be done. This needs to be done to make sure we do not exceed the maximum days available for members to work and also not to overburden members.

Progress Made in Addressing the Commitments Resulting from the CY 1999 ACRS/ACNW Self Assessment for CY 2000

Results of the ACRS/ACNW self assessment for CY 1999 were documented in SECY-00-0102 and sent to the Commission on May 5, 2000. In that SECY paper, the ACRS had made several commitments. The Subcommittee discussed the actions taken to address these commitments and provided feedback on the adequacy of these actions. The Subcommittee also provided guidance for conducting the CY 2000 self assessment.

ACRS Retreat for 2001

During the September meeting, the Committee agreed to have a retreat in January 2001 together with its visit to the San Onofre Nuclear Plant. Since the San Onofre visit did not materialize, a decision on the dates and location for the retreat should be made.

Proposed ACRS Meeting Dates for CY 2001

The proposed dates for CY 2001 ACRS meetings were discussed.

C. Future Meeting Agenda

Appendix IV summarizes the proposed items endorsed by the Committee for the 477th ACRS Meeting, November 2-4, 2000.

The 476th ACRS meeting was adjourned at 12:30 p.m. on October 7, 2000.

Room link at the NRC Web site (<http://www.nrc.gov>).

Dated at Rockville, Maryland, this 14th day of September 2000.

For the Nuclear Regulatory Commission.

Girija S. Shukla,

Project Manager, Section 2, Project Directorate IV and Decommissioning Division of Licensing Project Management Office of Nuclear Reactor Regulation.

[FR Doc. 00-24162 Filed 9-19-00; 8:45 am]

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NUCLEAR REGULATORY COMMISSION

Advisory Committee on Reactor Safeguards, Meeting of the Ad Hoc Subcommittee; Notice of Meeting

The Ad Hoc Subcommittee will hold a meeting on October 10-13, 2000, Room T-2B3, 11545 Rockville Pike, Rockville, Maryland.

The entire meeting will be open to public attendance.

The agenda for the subject meeting shall be as follows:

Tuesday October 10, 2000—1:00 p.m. until the conclusion of business

The Ad Hoc Subcommittee will discuss its approach for reviewing the technical merits of the Differing Professional Opinion (DPO) issues associated with steam generator tube integrity, and developing comments and recommendations for consideration by the full Committee.

Wednesday, October 11, 2000—8:30 a.m. until the conclusion of business

The Ad Hoc Subcommittee will hear presentations by and hold discussions with the DPO author and other interested persons regarding the DPO issues, views on the adequacy of the staff's approach for resolving these issues, and remaining major issues of contention.

Thursday October 12, 2000—8:30 a.m. until the conclusion of business

The Ad Hoc Subcommittee will hear presentations by and hold discussions with representatives of the NRC staff and other interested persons regarding the status of resolution of the DPO issues and related matters.

Friday, October 13, 2000—8:30 a.m. until the conclusion of business

The Ad Hoc Subcommittee will continue its discussion of the DPO issues with the staff and the DPO author, as needed, and will develop proposed comments and

recommendations for consideration by the full Committee.

The purpose of this meeting is to gather information, analyze relevant issues and facts, and to formulate proposed positions and actions, as appropriate, for deliberation by the full Committee.

Oral statements may be presented by members of the public with the concurrence of the Subcommittee Chairman; written statements will be accepted and made available to the Committee. Electronic recordings will be permitted only during those portions of the meeting that are open to the public, and questions may be asked only by members of the Ad Hoc Subcommittee, its consultants, and staff. Persons desiring to make oral statements should notify the cognizant ACRS staff engineers named below five days prior to the meeting, if possible, so that appropriate arrangements can be made.

During the initial portion of the meeting, the Ad Hoc Subcommittee, along with any of its consultants who may be present, may exchange preliminary views regarding matters to be considered during the balance of the meeting.

The Ad Hoc Subcommittee will then hear presentations by and hold discussions with the DPO author, representatives of the NRC staff, and other interested persons regarding this review.

Further information regarding topics to be discussed, whether the meeting has been canceled or rescheduled, and the Chairman's ruling on requests for the opportunity to present oral statements and the time allotted therefor can be obtained by contacting either Mr. Sam Duraiswamy (telephone 301-415-7364) or Ms. Undine Shoop (telephone 301-415-8086) between 7:30 a.m. and 4:15 p.m. (EDT). Persons planning to attend this meeting are urged to contact the above named individuals one or two working days prior to the meeting to be advised of any potential changes to the agenda, etc., that may have occurred.

Dated: September 12, 2000.

James E. Lyons,

Associate Director for Technical Support, ACRS/ACNW.

[FR Doc. 00-24159 Filed 9-19-00; 8:45 am]

BILLING CODE 7590-01-P

NUCLEAR REGULATORY COMMISSION

Advisory Committee on Reactor Safeguards; Meeting Notice

In accordance with the purposes of Sections 29 and 182b. of the Atomic

Energy Act (42 U.S.C. 2039, 2232b), the Advisory Committee on Reactor Safeguards will hold a meeting on October 5-7, 2000, in Conference Room T-2B3, 11545 Rockville Pike, Rockville, Maryland. The date of this meeting was previously published in the *Federal Register* on Thursday, October 14, 1999 (64 FR 55787).

Thursday, October 5, 2000

8:30 A.M.-8:45 A.M.: Opening Remarks by the ACRS Chairman (Open)—The ACRS Chairman will make opening remarks regarding the conduct of the meeting.

8:45 A.M.-10:00 A.M.: Discussion of Union of Concerned Scientists Report, "Nuclear Plant Risk Studies: Failing the Grade" (Open)—The Committee will hear presentations by and hold discussions with representatives of the Union of Concerned Scientists (UCS), the NRC staff, and other interested parties concerning the August 2000 UCS report on nuclear plant risk studies.

10:15 A.M.-11:30 A.M.: NEI 00-02, "Industry PRA Peer Review Process Guidelines" (Open)—The Committee will hear presentations by and hold discussions with representatives of the Nuclear Energy Institute (NEI) and the NRC staff regarding the proposed industry PRA certification guidelines described in the document NEI 00-02.

11:30 A.M.-12:30 P.M.: Staff Views on ASME Standard for PRA for Nuclear Power Plant Applications (Open)—The Committee will hear presentations by and hold discussions with representatives of the NRC staff regarding the staff's August 14, 2000 response to the American Society of Mechanical Engineers (ASME) draft Revision 12 ASME Standard for Probabilistic Risk Assessment for Nuclear Power Plant Applications.

1:30 P.M.-3:00 P.M.: Pressurized Thermal Shock Technical Bases Reevaluation Project (Open)—The Committee will hear presentations by and hold discussions with representatives of the NRC staff regarding the pressurized thermal shock technical bases reevaluation project.

3:30 P.M.-4:30 P.M.: Break and Preparation of Draft ACRS Reports (Open)—Cognizant ACRS members will prepare draft reports, as needed, for consideration by the full Committee.

4:30 P.M.-6:00 P.M.: Discussion of Proposed ACRS Reports (Open)—The Committee will discuss proposed ACRS reports on matters considered during this meeting.

6:00 P.M.-7:00 P.M.: Discussion of Topics for Meeting with the NRC Commissioners (Open)—The Committee will discuss issues associated with risk

informing 10 CFR 50, quality of PRAs, spent fuel pool fire safety study, more realistic (best estimate) thermal-hydraulic codes and status of ACRS activities on license renewals.

Friday, October 6, 2000

8:30 A.M.–8:35 A.M.: Opening Remarks by the ACRS Chairman (Open)—The ACRS Chairman will make opening remarks regarding the conduct of the meeting.

8:35 A.M.–9:15 A.M.: Discussion of Topics for Meeting with the NRC Commissioners (Open)—The Committee will discuss matters scheduled for the meeting with the NRC Commissioners associated with risk informing 10 CFR 50 and related matters.

9:30 A.M.–12:00 Noon: Meeting with the NRC Commissioners (Open)—The Committee will meet with the NRC Commissioners, Commissioners' Conference Room, One White Flint North to discuss risk informing 10 CFR 50 and related matters.

1:30 P.M.–3:00 P.M.: Discussion of Industry Issues (Open)—The Committee will hear a presentation by R. Beedle, Senior Vice President, NEI on issues of mutual interest.

3:15 P.M.–4:45 P.M.: GSI-168, Equipment Qualification (Open)—The Committee will hear presentations by and hold discussions with representatives of the NRC staff regarding the GSI-168, Equipment Qualification.

4:45 P.M.–5:30 P.M.: ACRS Review of Generic Guidance Documents Associated with License Renewal (Open)—The Committee will discuss concerns identified during their initial review of the draft guidance documents.

5:30 P.M.–5:50 P.M.: Future ACRS Activities/Report of the Planning and Procedures Subcommittee (Open)—The Committee will discuss the recommendations of the Planning and Procedures Subcommittee regarding items proposed for consideration by the full Committee during future meetings. Also, it will hear a report of the Planning and Procedures Subcommittee on matters related to the conduct of ACRS business, and organizational and personnel matters relating to the ACRS.

5:50 P.M.–6:00 P.M.: Reconciliation of ACRS Comments and Recommendations (Open)—The Committee will discuss the responses from the NRC Executive Director for Operations (EDO) to comments and recommendations included in recent ACRS reports and letters. The EDO responses are expected to be made available to the Committee prior to the meeting.

6:00 P.M.–6:30 P.M.: Break and Preparation of Draft ACRS Reports (Open)—Cognizant ACRS members will prepare draft reports, as needed, for consideration by the full Committee.

6:30 P.M.–7:30 P.M.: Discussion of Proposed ACRS Reports (Open)—The Committee will discuss proposed ACRS reports.

Saturday, October 7, 2000

8:30 A.M.–8:35 A.M.: Opening Remarks by the ACRS Chairman (Open)—The ACRS Chairman will make opening remarks regarding the conduct of the meeting.

8:35 A.M.–12:30 P.M.: Discussion of Proposed ACRS Reports (Open)—The Committee will continue its discussion of proposed ACRS reports.

12:30 P.M.–1 P.M.: Annual Report to the Commission on the NRC Safety Research Program (Open)—The Committee will discuss the format and content of the annual ACRS report to the Commission on the NRC Safety Research Program.

1 P.M.–1:30 P.M.: Miscellaneous (Open)—The Committee will discuss matters related to the conduct of Committee activities and matters and specific issues that were not completed during previous meetings, as time and availability of information permit.

Procedures for the conduct of and participation in ACRS meetings were published in the *Federal Register* on September 28, 1999 (64 FR 52353). In accordance with these procedures, oral or written views may be presented by members of the public, including representatives of the nuclear industry. Electronic recordings will be permitted only during the open portions of the meeting and questions may be asked only by members of the Committee, its consultants, and staff. Persons desiring to make oral statements should notify Mr. James E. Lyons, ACRS, five days before the meeting, if possible, so that appropriate arrangements can be made to allow necessary time during the meeting for such statements. Use of still, motion picture, and television cameras during the meeting may be limited to selected portions of the meeting as determined by the Chairman.

Information regarding the time to be set aside for this purpose may be obtained by contacting Mr. James E. Lyons prior to the meeting. In view of the possibility that the schedule for ACRS meetings may be adjusted by the Chairman as necessary to facilitate the conduct of the meeting, persons planning to attend should check with Mr. James E. Lyons if such rescheduling would result in major inconvenience.

Further information regarding topics to be discussed, whether the meeting has been canceled or rescheduled, the Chairman's ruling on requests for the opportunity to present oral statements, and the time allotted therefor can be obtained by contacting Mr. James E. Lyons (telephone 301-415-7371), between 7:30 a.m. and 4:15 p.m., EDT.

ACRS meeting agenda, meeting transcripts, and letter reports are available for downloading or viewing on the internet at <http://www.nrc.gov/ACRSACNW>.

Videoteleconferencing service is available for observing open sessions of ACRS meetings. Those wishing to use this service for observing ACRS meetings should contact Mr. Theron Brown, ACRS Audio Visual Technician (301-415-8066), between 7:30 a.m. and 3:45 p.m., EDT, at least 10 days before the meeting to ensure the availability of this service. Individuals or organizations requesting this service will be responsible for telephone line charges and for providing the equipment facilities that they use to establish the videoteleconferencing link. The availability of videoteleconferencing services is not guaranteed.

Dated: September 14, 2000.

Andrew L. Bates,
Advisory Committee Management Officer.
[FR Doc. 00-24160 Filed 9-19-00; 8:45 am]
BILLING CODE 7590-01-P

NUCLEAR REGULATORY COMMISSION

Biweekly Notice; Applications and Amendments to Facility Operating Licenses Involving No Significant Hazards Considerations

I. Background

Pursuant to Public Law 97-415, the U.S. Nuclear Regulatory Commission (the Commission or NRC staff) is publishing this regular biweekly notice. Public Law 97-415 revised section 189 of the Atomic Energy Act of 1954, as amended (the Act), to require the Commission to publish notice of any amendments issued, or proposed to be issued, under a new provision of section 189 of the Act. This provision grants the Commission the authority to issue and make immediately effective any amendment to an operating license upon a determination by the Commission that such amendment involves no significant hazards consideration, notwithstanding the pendency before the Commission of a request for a hearing from any person.



UNITED STATES
NUCLEAR REGULATORY COMMISSION
 ADVISORY COMMITTEE ON REACTOR SAFEGUARDS
 WASHINGTON, D.C. 20555-0001

September 13, 2000

SCHEDULE AND OUTLINE FOR DISCUSSION
476TH ACRS MEETING
OCTOBER 5-7, 2000

**THURSDAY, OCTOBER 5, 2000, CONFERENCE ROOM 2B3, TWO WHITE FLINT NORTH,
 ROCKVILLE, MARYLAND**

- 1) 8:30 - 8:45 A.M. Opening Remarks by the ACRS Chairman (Open)
 - 1.1) Opening statement (DAP/JTL/HJL)
 - 1.2) Items of current interest (DAP/NFD/HJL)
 - 1.3) Priorities for preparation of ACRS reports (DAP/JTL/HJL)

- 2) 8:45 - ^{9:45}~~10:00~~ A.M. Discussion of Union of Concerned Scientists Report, "Nuclear Plant Risk Studies: Failing the Grade" (Open) (GA/MTM)
 Briefing by and discussions with representatives of the Union of Concerned Scientists (UCS), the NRC staff, and other interested parties concerning the August 2000 UCS report on nuclear plant risk studies.

- ^{9:45}~~10:00~~ - 10:15 A.M. *****BREAK*****

- 3) 10:15 - ^{11:40}~~11:30~~ A.M. NEI 00-02, "Industry PRA Peer Review Process Guidelines" (Open) (GA/MTM)
 - 3.1) Opening remarks by the Subcommittee Chairman
 - 3.2) Briefing by and discussions with representatives of the Nuclear Energy Institute (NEI) and the NRC staff regarding the proposed industry PRA certification guidelines described in the document NEI 00-02.

Representatives of the nuclear industry will provide their views, as appropriate.

- 4) ^{11:40-12:40}~~11:30 - 12:30~~ P.M. Staff Views on ASME Standard for PRA for Nuclear Power Plant Applications (Open) (GA/MTM)
 - 4.1) Remarks by the Subcommittee Chairman
 - 4.2) Briefing by and discussions with representatives of the NRC staff regarding the staff's August 14, 2000 response to the American Society of Mechanical Engineers (ASME) draft Revision 12 ASME Standard for Probabilistic Risk Assessment for Nuclear Power Plant Applications.

Representatives of the nuclear industry will provide their views, as appropriate.

- ^{12:40-1:40}~~12:30 - 1:30~~ P.M. *****LUNCH*****

- 5) ^{1:40}
~~1:30~~ - 3:30 P.M. Pressurized Thermal Shock Technical Bases Reevaluation Project (Open) (WJS/NFD)
5.1) Remarks by the Subcommittee Chairman
5.2) Briefing by and discussions with representatives of the NRC staff regarding the pressurized thermal shock technical bases reevaluation project.
- 6) ^{3:40-3:50}
~~3:30-4:30 P.M.~~ Break
Break and Preparation of Draft ACRS Reports (Open)
~~Cognizant ACRS members will prepare draft reports, as needed, for consideration by the full Committee.~~
- 7) ^{3:50}
~~4:30-~~ 6:00 P.M. Discussion of Proposed ACRS Reports (Open)
Preparation of proposed ACRS reports on:
3:55-7:15 7.1) Union of Concerned Scientists Report on Nuclear Plant Risk Studies (GA/MTM)
7.2) NEI 00-02, "Industry PRA Peer Review Process Guidelines" (GA/MTM)
7.3) Pressurized Thermal Shock Technical Bases Reevaluation Project (WJS/NFD)
- 8) ^{7:30} End of Session
~~6:00-7:00 P.M.~~ Discussion of Topics for Meeting with the NRC Commissioners (Open) (DAP, et al./JTL, et al.)
Discussion of topics and preparation for meeting with the NRC Commissioners scheduled for 9:30 a.m. - 12:00 Noon, Friday, October 6 concerning:
8.1) Risk Informing 10 CFR 50 (WJS/MTM)
- NEI Letter of January 19, 2000
- Proposed Revision to 10 CFR 50.44 Concerning Combustible Gas Control System and Advance Notice of Proposed Rulemaking (10 CFR 50.69 and Appendix T)
8.2) Quality of PRAs (GA/MWW)
- Assessment of the Quality of PRAs
- ASME Standard on PRAs
8.3) Spent Fuel Pool Fire Safety Study (TSK/MME)
8.4) More Realistic (Best Estimate) Thermal-Hydraulic Codes (GW/PAB)
8.5) Status of ACRS Activities on License Renewals (MVB/NFD)

FRIDAY, OCTOBER 6, 2000, CONFERENCE ROOM 2B3, TWO WHITE FLINT NORTH, ROCKVILLE, MARYLAND

- 9) 8:30 - 8:35 A.M. Opening Remarks by the ACRS Chairman (Open) (DAP/JTL)
- 10) 8:35 - 9:15 A.M. Discussion of Topics for Meeting with the NRC Commissioners (Open) (DAP, et al./JTL, et al.)
Discussion of topics listed under Item 8.
- 9:15 - 9:30 A.M. *****BREAK*****

- 11) 9:30 - 12:00 Noon Meeting with the NRC Commissioners (Open) (DAP, et al./JTL, et al.)
Meeting with the NRC Commissioners, Commissioners' Conference Room, One White Flint North, to discuss topics listed under Item 9 and other items of mutual interest.
- 12:00 - 1:30 P.M. *****LUNCH*****
- 12) 1:30 - 3:00 P.M. Discussion of Industry Issues (Open) (DAP/RPS)
Presentation by R. Beedle, Senior Vice President, NEI, on issues of mutual interest.
- 3:00 - ^{3:20}~~3:45~~ P.M. *****BREAK*****
- 13) ^{3:20}~~3:00~~ - ^{4:50}~~4:45~~ P.M. GSI-168, Equipment Qualification (Open) (REU/AS)
14.1) Remarks by the Subcommittee Chairman
14.2) Briefing by and discussions with representatives of the NRC staff regarding the GSI-168, Equipment Qualification.
- 14) 4:45 - ^{5:00}~~5:30~~ P.M. ACRS Review of Generic Guidance Documents Associated with License Renewal (Open) (MVB/NFD)
The Committee members will discuss concerns identified during their initial review of the draft guidance documents.
- 15) ^{5:00}~~5:30~~ - ^{6:30}~~5:50~~ P.M. Future ACRS Activities/Report of the Planning and Procedures Subcommittee (Open) (DAP/JTL/HJL)
15.1) Discussion of the recommendations of the Planning and Procedures Subcommittee regarding items proposed for consideration by the full Committee during future ACRS meetings.
15.2) Report of the Planning and Procedures Subcommittee on matters related to the conduct of ACRS business, and organizational and personnel matters relating to the ACRS.
- 16) 5:50 - 6:00 P.M. Reconciliation of ACRS Comments and Recommendations (Open) (DAP, et al./HJL, et al.)
Discussion of the responses from the NRC Executive Director for Operations to comments and recommendations included in recent ACRS reports and letters.
- 17) 6:00 - 6:30 P.M. Break and Preparation of Draft ACRS Reports
Cognizant ACRS members will prepare draft reports for consideration by the full Committee.
- 18) 6:30 - ^{7:45}~~7:30~~ P.M. Discussion of Proposed ACRS Reports (Open)
Preparation of proposed ACRS reports on:
19.1) GSI-168, Equipment Qualification (REU/AS)
19.2) Union of Concerned Scientists Report on Nuclear Plant Risk Studies (GA/MTM)
19.3) NEI 00-02, "Industry PRA Peer Review Process Guidelines" (GA/MTM)
19.4) Pressurized Thermal Shock Technical Bases Reevaluation Project (WJS/NFD)
- 10/7/2010 - 8:30 9:40

**SATURDAY, OCTOBER 7, 2000, CONFERENCE ROOM 2B3, TWO WHITE FLINT NORTH,
ROCKVILLE, MARYLAND**

- 19) 8:30 - 8:35 A.M. Opening Remarks by the ACRS Chairman (Open) (DAP/JTL)
- 20) 8:35 - 12:30 P.M. Discussion of Proposed ACRS Reports (Open) - The Committee will continue its discussion of proposed ACRS reports as noted in Item 19.
- 21) 12:30 - 1:00 P.M. Annual Report to the Commission on the NRC Safety Research Program (Open) (DAP/MME)
Discussion of the current status of the review by the members of the topical areas previously assigned.
- 22) 1:00 - 1:30 P.M. Miscellaneous (Open) (DAP/JTL)
Discussion of matters related to the conduct of Committee activities and matters and specific issues that were not completed during previous meetings, as time and availability of information permit.

NOTE:

- Presentation time should not exceed 50 percent of the total time allocated for a specific item. The remaining 50 percent of the time is reserved for discussion.
- Number of copies of the presentation materials to be provided to the ACRS - 35.

APPENDIX III: MEETING ATTENDEES

476TH ACRS MEETING OCTOBER 5-7, 2000

NRC STAFF (October 5, 2000)

A. Levin, OCM/RAM
J. Williams, NRR
G. Parry, NRR
E. Throm, NRR
J. Bongarre, NRR
C. Douitt, NRR
J. Dozio, NRR
M. Cheok, NRR
S. Dinsmore, NRR
S. Magruder, NRR
R. Barrett, NRR
S. West, NRR
M. Rubin, NRR
E. McKenna, NRR
L. Lois, NRR
S. Mays, RES
J. Mitchell, RES
M. Drouin, RES
A. Ramey-Smith, RES
L. Abramson, RES
S. Malik, RES
N. Siu, RES
B. Jones, RES
M. Mayfield, RES
R. Woods, RES
E. Hackett, RES
D. Jackson, RES

ATTENDEES FROM OTHER AGENCIES AND GENERAL PUBLIC

N. Chapman, SERCH/Bechtel
S. Vance, Shaw Pittman
D. Lochbaum, UCS
J. Meyer, ISL
H. Fonticella, Dominion
B. Bradley, NEI
J. Mallay, Siemens
K. Fleming, Erin Engineering & Research
B. Hansen, ENS
M. Knapik, McGraw-Hill

T. Dickson, ORNL
B. Handles, CCNPPI
M. Natishan, PEAI

NRC STAFF (October 6, 2000)

S. West, NRR
E. McKenna, NRR
P. Shemanski, NRR
J. Calvo, NRR
J. Mitchell, RES
J. Vora, RES
H. VanderMolen, RES
E. Hackett, RES
J. Rosenthal, RES
A. Buslik, RES
C. Poslusny, NMSS

ATTENDEES FROM OTHER AGENCIES AND GENERAL PUBLIC

J. Petro, Winston & Strawn
D. Raleigh, SERCH/Bechtel
L. Hendricks, NEI
A. Marion, NEI
D. Walters, NEI
A. Nelson, NEI
B. Horin, Winston & Strawn
G. Toman, EPRI



**UNITED STATES
NUCLEAR REGULATORY COMMISSION
ADVISORY COMMITTEE ON REACTOR SAFEGUARDS
WASHINGTON, D.C. 20555-0001**

October 17, 2000

**SCHEDULE AND OUTLINE FOR DISCUSSION
477TH ACRS MEETING
NOVEMBER 2-4, 2000**

**THURSDAY, NOVEMBER 2, 2000, CONFERENCE ROOM 2B3, TWO WHITE FLINT NORTH,
ROCKVILLE, MARYLAND**

- 1) 8:30 - 8:35 A.M. Opening Remarks by the ACRS Chairman (Open)
 - 1.1) Opening statement (DAP/JTL/HJL)
 - 1.2) Items of current interest (DAP/NFD/HJL)
 - 1.3) Priorities for preparation of ACRS reports (DAP/JTL/HJL)
- 2) 8:35 - 10:45 A.M. Revised Report of the Final Technical Study of Spent Fuel Pool
Accident Risk at Decommissioning Nuclear Power Plants
(TSK/MME/MWW)
 - 2.1) Remarks by the Subcommittee Chairman
 - 2.2) Briefing by and discussions with representatives of the NRC
staff regarding the revised version of the report and the staff's
response to previous ACRS concerns.

Representatives of the nuclear industry will provide their views, as appropriate.

10:45 - 11:00 A.M. *BREAK*****

- 3) 11:00 - 12:30 P.M. Risk-Informed Regulation Implementation Plan (RIRIP) (Open)
(GA/MTM)
 - 3.1) Remarks by the Subcommittee Chairman
 - 3.2) Briefing by and discussions with representatives of the NRC
staff regarding the update to the RIRIP.

Representatives of the nuclear industry will provide their views, as appropriate.

12:30 - 1:30 P.M. *LUNCH*****

- 4) 1:30 - 2:30 P.M. Proposed Framework for Risk-Informed Changes to the Technical
Requirements of 10 CFR Part 50 (Open) (GA/MTM)
 - 4.1) Remarks by the Subcommittee Chairman
 - 4.2) Briefing by and discussions with representatives of the NRC
staff regarding the proposed NRC framework for risk-informed
changes to the technical requirements of 10 CFR Part 50
described in Attachment 1 to SECY-00-0198.

Representatives of the nuclear industry will provide their views, as appropriate.

- 5) 2:30 - 4:30 P.M. Differing Professional Opinion (DPO) on Steam Generator Tube Integrity (Open) (DAP/SD/US)
- 5.1) Report by the Chairman of the Ad Hoc Subcommittee on DPO Issues regarding the outcome of the October 10-14 subcommittee meeting, proposed subcommittee recommendations, schedule for completing the review, and related matters.
- 5.2) Briefing by and discussions with the DPO author and representatives of the NRC staff, as needed, on additional information related to DPO issues.
- 6) 4:30 - 5:30 P.M. Break and Preparation of Draft ACRS Reports (Open)
Cognizant ACRS members will prepare draft reports, as needed, for consideration by the full Committee.
- 7) 5:30 - 7:00 P.M. Proposed ACRS Reports (Open)
Discussion of proposed ACRS reports on:
- 7.1) Framework for Risk-Informed Changes to 10 CFR Part 50 (GA/MTM)
- 7.2) Spent Fuel Pool Accident Risk at Decommissioning Nuclear Power Plants (TSK/MME/MWW)
- 7.3) Risk-Informed Regulation Implementation Plan (GA/MTM)
- 7.4) DPO on Steam Generator Tube Integrity (DAP/SD/US)

FRIDAY, NOVEMBER 3, 2000, CONFERENCE ROOM 2B3, TWO WHITE FLINT NORTH, ROCKVILLE, MARYLAND

- 8) 8:30 - 8:35 A.M. Opening Remarks by the ACRS Chairman (Open) (DAP/JTL)
- 9) 8:35 - 10:30 A.M. Performance-Based, Risk-Informed Fire Protection Standard for LWRs and Related Issues (Open) (JDS/DAP/AS)
- 9.1) Remarks by the Subcommittee Chairman
- 9.2) Briefing by and discussions with representatives of the NRC staff, Nuclear Energy Institute, and National Fire Protection Association (NFPA) on the revised NFPA 805 standard, post-fire safe shutdown circuit analysis, and other related fire protection issues.
- 10:30 - 10:45 A.M. ***BREAK*****
- 10) 10:45 - 12:00 Noon ABB/CE and Siemens Digital I&C Applications (Open) (REU/AS)
Report by the Subcommittee Chairman on a subcommittee meeting on this matter and his recommendation regarding further review by the full Committee.
- 12:00 - 1:00 P.M. ***LUNCH*****

- 11) 1:00 - 3:00 P.M. License Renewal Guidance Documents (Open) (MVB/RLS/NFD)
 11.1) Remarks by the Subcommittee Chairman
 11.2) Briefing by and discussions with representatives of the NRC staff regarding proposed Standard Review Plan for License Renewal, Generic Aging Lessons Learned Report, Regulatory Guide, and NEI 95-10, Industry Guidelines for Implementing the Requirements of the License Renewal Rule..
- Representatives of the nuclear industry will provide their views, as appropriate.
- 3:00 - 3:15 P.M. *****BREAK*****
- 12) 3:15 - 4:30 P.M. Research Report to the Commission (Open) (DAP/MME)
 Discussion of the status of the draft ACRS report on the NRC Safety Research Program.
- Representatives of the NRC staff will provide their views, as appropriate.
- 13) 4:30 - 5:00 P.M. Future ACRS Activities/Report of the Planning and Procedures Subcommittee (Open) (DAP/JTL/HJL)
 13.1) Discussion of the recommendations of the Planning and Procedures Subcommittee regarding items proposed for consideration by the full Committee during future ACRS meetings.
 13.2) Report of the Planning and Procedures Subcommittee on matters related to the conduct of ACRS business, and organizational and personnel matters relating to the ACRS.
- 14) 5:00 - 5:15 P.M. Reconciliation of ACRS Comments and Recommendations (Open) (DAP, et al./HJL, et al.)
 Discussion of the responses from the NRC Executive Director for Operations to comments and recommendations included in recent ACRS reports and letters.
- 15) 5:15 - 6:00 P.M. Break and Preparation of Draft ACRS Reports
 Cognizant ACRS members will prepare draft reports, as needed, for consideration by the full Committee.
- 16) 6:00 - 7:30 P.M. Proposed ACRS Reports (Open)
 Discussion of proposed ACRS reports on:
 16.1) Framework for Risk-Informed Changes to 10 CFR Part 50 (GA/MTM)
 16.2) Spent Fuel Pool Accident Risk at Decommissioning Nuclear Power Plants (TSK/MME/MWW)
 16.3) Risk-Informed Regulation Implementation Plan (GA/MTM)
 16.4) DPO on Steam Generator Tube Integrity (DAP/SD/US)
 16.5) Performance-Based, Risk-Informed Fire Protection Standard (JDS/DAP/AS)

- 16.6) Research Report to the Commission (DAP/MME)
- 16.7) License Renewal Guidance Documents (MVB/RLS/NFD)

SATURDAY, NOVEMBER 4, 2000, CONFERENCE ROOM 2B3, TWO WHITE FLINT NORTH, ROCKVILLE, MARYLAND

- 17) 8:30 - 1:00 P.M. Proposed ACRS Reports (Open) - The Committee will continue its discussion and preparation of proposed ACRS reports listed under item 16.
- 18) 1:00 - 1:30 P.M. Miscellaneous (Open) (DAP/JTL)
Discussion of matters related to the conduct of Committee activities and matters and specific issues that were not completed during previous meetings, as time and availability of information permit.

NOTE:

- Presentation time should not exceed 50 percent of the total time allocated for a specific item. The remaining 50 percent of the time is reserved for discussion.
- Number of copies of the presentation materials to be provided to the ACRS - 35.

APPENDIX V
LIST OF DOCUMENTS PROVIDED TO THE COMMITTEE
476th ACRS MEETING
OCTOBER 5-7, 2000

[Note: Some documents listed below may have been provided or prepared for Committee use only. These documents must be reviewed prior to release to the public.]

MEETING HANDOUTS

AGENDA
ITEM NO.

DOCUMENTS

- 1 Opening Remarks by the ACRS Chairman
 1. Items of Interest, dated October 5-7, 2000
- 2 Discussion of Union of Concerned Scientists Report, "Nuclear Plant Risk Studies: Failing the Grade"
 2. Presentation to ACRS on Nuclear Plant Risk Studies: Failing the Grade by D. Lochbaum, Union of Concerned Scientists [Viewgraphs]
- 3 NEI 00-02, "Industry PRA Peer Review Process Guidelines"
 3. Industry PSA Peer Review Process presentation by NEI [Viewgraphs]
 4. Role of the Industry PRA Peer Review Process to Support Quality PRA Applications presentation by K. Fleming, Erin Engineering and Research, Inc. [Viewgraphs]
- 4 Staff Views on ASME Standard for PRA for Nuclear Power Plant Applications
 5. Recent Activities on ASME Revision 12 PRA Standard presentation by M. Drouin, RES, and G. Parry, NRR [Viewgraphs]
- 5 Pressurized Thermal Shock Technical Bases Reevaluation Project
 6. Status of the Favor Code Development presentation by Oak Ridge National Laboratory [Viewgraphs]
 7. Fracture Toughness Distributions and Uncertainty Analysis presentation by M. Kirk, RES [Viewgraphs]
- 12 Discussion of Industry Issues
 8. Nuclear Energy Institute Organizational Chart, presentation by R. Beedle, Senior Vice President & Chief Nuclear Officer [Handout]
- 13 GSI-168, Equipment Qualification
 9. Environmental Qualification (EQ) of Low-Voltage I&C Cables presentation by RES [Viewgraphs]
 10. GSI-168 (Pre-Decisional) author RES [Handout 13-1]

- 14 ACRS Review of Generic Guidance Documents Associated with License Renewal
 - 11. Guidance for ACRS Review of License Renewal Generic Document presentation by Dr. M. Bonaca [Handout]
- 15 Future ACRS Activities/Report of the Planning and Procedures Subcommittee
 - 12. Final Draft Minutes of Planning and Procedures Subcommittee Meeting - **XX**, 1998 [Handout #15.2]
- 16 Reconciliation of ACRS Comments and Recommendations
 - 13. Reconciliation of ACRS Comments and Recommendations [Handout #**XX**]

MEETING NOTEBOOK CONTENTS

TAB

DOCUMENTS

- 2 Discussion of Union of Concerned Scientists Report, "Nuclear Plant Risk Studies: Failing the Grade"
 - 1 Table of Contents
 - 2 Proposed Schedule
 - 3 Project Status Report, dated October 5, 2000
 - 4 NRC Staff's Talking Points on UCS report, dated August 22, 2000
 - 5 UCS report August 2000

- 3 NEI 00-02, "Industry PRA Peer Review Process"
 - 6 Table of Contents
 - 7 Proposed Schedule
 - 8 Status Report dated October 5, 2000
 - 9 NRR Request for additional information (RAI) dated September 19, 2000
 - 10 RES RAI dated August 11, 2000
 - 11 RES memorandum to NRR regarding request for technical assistance
 - 12 Letter from NRR to R. Beedle, NEI, dated June 9, 2000
 - 13 User needs request dated June 19, 2000

- 4 Staff Views on ASME Standard for PRA for Nuclear Power Plants
 - 14 Table of Contents
 - 15 Proposed Schedule
 - 16 Status Report dated October 5, 2000
 - 17 Memorandum dated August 17, 2000, from Ashok Thadani, RES, to John W. Craig, Assistant for Operations, EDO, Subject: Staff Comments on June 2000 Draft ASME Standard on PRA Quality
 - 18 Talking Points for 9/19/00 Conference Call on ASME Standard, RES
 - 19 Principles/Objectives for the ASME Standard, RES
 - 20 Letter dated September 7, 2000, from Dana A. Powers, Chairman, ACRS, to Richard A. Meserve, Chairman, NRC, Subject: Assessment of the Quality of Probabilistic Risk Assessment
 - 21 SECY-00-0162, Memorandum dated July 28, 2000, from William D. Travers, EDO, NRC for the Commissioners, Subject: Addressing PRA Quality in Risk-Informed Activities
 - 22 Letter dated July 20, 2000, from Dana A. Powers, Chairman, ACRS, to William D. Travers, EDO, Subject: Proposal Final ASME Standard for Probabilistic Risk Assessment for Nuclear Power Plant Applications
 - 23 Letter dated August 31, 2000, from William D. Travers, EDO, to Dana A. Powers, Chairman, ACRS, Subject: ACRS Letter dated July 20, 2000,

"Proposed Final ASME Standard for Probabilistic Risk Assessment for
Nuclear Power Plant Operations"

- 5 Pressure Thermal Shock Technical Basis Reevaluation Project
 - 24. Table of Contents
 - 25. Proposed Schedule
 - 26. Status Report dated October 5, 2000
 - 27. Bowman, K. and Williams, P., Oak Ridge National Laboratory, "Technical Basis for Statistical Models of Extend K_{Ic} and K_{Ia} Fracture Toughness Databases for RPV Steels," dated February 2000
 - 28. Li, F. et al. University of Maryland, " K_{Ic}/K_{Ia} Uncertainty Characterization," dated June 23, 2000

- 12 Discussion of Industry Issues
 - 29. Table of Contents
 - 30. Proposed Schedule
 - 31. Status Report dated October 6, 2000
 - 32. NEI Strategic Plan - Nuclear Energy: The Renaissance Revealed - A Strategic Director for the 21st Century.
 - 33. NEI 1999 Annual Report

- 13 Proposed Resolution of Generic Safety Issue 168, "Environmental Qualification of Electrical Equipment"
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 - 40. Memorandum from Noel Dudley, ACRS/ACNW, to ACRS Members, Subject: ACRS Review Plans for License Renewal Guidance Documents, dated August 29, 2000
 - 41. Status of License Renewal Issue Inventory