



Tennessee Valley Authority, Post Office Box 2000, Soddy-Daisy, Tennessee 37379

November 16, 2000

U.S. Nuclear Regulatory Commission
ATTN: Document Control Desk
Washington, D.C. 20555

Gentlemen:

In the Matter)
Tennessee Valley Authority) Docket No. 50-328

SEQUOYAH NUCLEAR PLANT (SQN) - UNIT 2 STEAM GENERATOR (SG)
TUBE PLUGGING REPORT AND FOLLOW-UP REPORT FOLLOWING NRC
NOTIFICATION FOR CATEGORY C-3 - SG TUBE INSPECTIONS DURING
UNIT 2 CYCLE 10 REFUELING OUTAGE

As required by SQN Technical Specification (TS) 4.4.5.5.a, Enclosure 1 provides a summary report of SG tube plugging during the Unit 2 Cycle 10 refueling outage. The inservice inspection of the Unit 2 SG tubes was completed on November 7, 2000. In accordance with SQN TS 4.4.5.5.b, TVA will submit a special report of the results from this inspection on or before November 7, 2001.

Pursuant to the reporting requirements of TS 4.4.5.5.c, NRC was notified of the SG tube inspections that fell into Category C-3. Notification of these results was made during two telephone conference calls on November 3, 2000 for SG 4 and on November 6, 2000 for SGs 2 and 3. Enclosure 2 provides the TVA followup report.

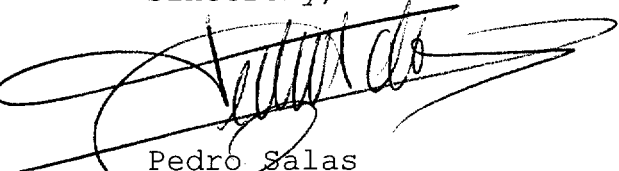
There are no commitments contained within this letter.

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U.S. Nuclear Regulatory Commission
Page 2
November 16, 2000

Please direct questions concerning this issue to me at (423) 843-7170 or J. D. Smith at (423) 843-6672.

Sincerely,



Pedro Salas
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Enclosure

cc (Enclosure):

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ENCLOSURE 1

SEQUOYAH NUCLEAR PLANT

UNIT 2 CYCLE 10 REFUELING OUTAGE

STEAM GENERATOR TUBE PLUGGING REPORT

<u>SG</u>	<u>ROW</u>	<u>COL</u>
1	2	84
1	10	47
1	12	90
1	19	69
1	19	82
1	24	33
1	25	39
1	43	61

Total Number of Tubes
Plugged This Generator: 8

SEQUOYAH NUCLEAR PLANT
UNIT 2 CYCLE 10 REFUELING OUTAGE
STEAM GENERATOR TUBE PLUGGING REPORT

<u>SG</u>	<u>ROW</u>	<u>COL</u>
2	1	38
2	1	41
2	1	77
2	2	59
2	3	59
2	3	93
2	7	63
2	7	86
2	9	51
2	13	72
2	14	33
2	15	33
2	19	82
2	21	50
2	29	37
2	30	49

Total Number of Tubes
Plugged This Generator: 16

SEQUOYAH NUCLEAR PLANT

UNIT 2 CYCLE 10 REFUELING OUTAGE

STEAM GENERATOR TUBE PLUGGING REPORT

<u>SG</u>	<u>ROW</u>	<u>COL</u>
3	1	30
3	1	58
3	1	90
3	7	7
3	10	27
3	12	30
3	12	36
3	12	73
3	12	83
3	13	35
3	19	46
3	22	44
3	32	17
3	34	43

Total Number of Tubes
Plugged This Generator: 14

SEQUOYAH NUCLEAR PLANT

UNIT 2 CYCLE 10 REFUELING OUTAGE

STEAM GENERATOR TUBE PLUGGING REPORT

<u>SG</u>	<u>ROW</u>	<u>COL</u>
4	1	6
4	1	7
4	1	8
4	1	11
4	1	94
4	2	61
4	2	68
4	3	24
4	4	6
4	5	5
4	5	7
4	6	37
4	7	3
4	8	26
4	9	69
4	13	38
4	16	42
4	17	28
4	18	68

SEQUOYAH NUCLEAR PLANT
UNIT 2 CYCLE 10 REFUELING OUTAGE
STEAM GENERATOR TUBE PLUGGING REPORT

(Continued from previous page)

<u>SG</u>	<u>ROW</u>	<u>COL</u>
4	20	28
4	23	59
4	26	30
4	28	36
4	38	61
4	39	63
4	45	58

Total Number of Tubes
Plugged This Generator: 26

Total Number of Tubes
Plugged from U2C10
SG Inspections: 64

ENCLOSURE 2

SEQUOYAH NUCLEAR PLANT

FOLLOW-UP REPORT FOR TS 4.4.5.5 c

STEAM GENERATOR CATEGORY C-3

In accordance with SQN Technical Specification 4.4.5.5.c, TVA reported to the NRC during two teleconference calls that the U-Bend Plus Point inspections were categorized as C-3 in steam generators 2, 3, and 4. Inspection results indicate that SGs 2 and 3 have two tubes with U-Bend Primary Water Stress Corrosion Cracks (PWSCC) and SG 4 have 7 tubes with U-Bend PWSCC. Because the original inspection scope for Row 1 and 2 U-Bends was 153 tubes in SG 2, 165 tubes in SG3, and 181 tubes in SG 4, the inspections were categorized as C-3. Two indications were in Row 2 and one indication was in Row 3. This resulted in an expansion to 100% inspection of Row 3 in all SGs and 20% of Row 4 in all SGs.

PWSCC at inner radius U-Bends is directly related to cold work and residual stresses associated with the tube manufacturing technique. The Unit 2 Row 1 and 2 U-Bends operated in this condition for multiple cycles and subsequently were in-situ stress relieved as a corrective measure. Cracking that initiated prior to stress relief continues to grow to detectable levels. The 100% inspection ensures that significant flaws are removed from service.

Bobbin inspection in SG 4 is also categorized as C-3 because greater than 10% of the tubes inspected were degraded. This is primarily due to Generic Letter 95-05 alternate repair criteria where very small voltage ODSCC axial indications are left in service inside the tube support plates.