Quad Cities Generating Station
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An Exelon Company

November 2, 2000

SVP-00-170

United States Nuclear Regulatory Commission
ATTN: Document Control Desk
Washington, DC 20555

Quad Cities Nuclear Power Station, Units 1 and 2
Facility Operating License Nos. DPR-29 and DPR-30
NRC Docket Nos. 50-254 and 50-265

Subject: Licensee Event Report Concerning Inadequate Calibration of Post-Accident Torus Temperature Monitors

Enclosed is Licensee Event Report (LER) 254/00-02, Revision 00, for Quad Cities Nuclear Power Station.

This report is submitted in accordance with the requirements of the Code of Federal Regulations, Title 10, Part 50.73(a)(2)(i)(B). The licensee shall report any operation or condition prohibited by the Plant's Technical Specifications.

We are committing to the following actions:

Determination of the root case for this event is in progress. A supplemental report will be submitted upon completion of the root cause determination.

Any other actions described in the submittal represent intended or planned actions by Commonwealth Edison (ComEd) Company. They are described for the NRC's information and are not regulatory commitments.

IE22

Should you have any questions concerning this letter, please contact Mr. C.C. Peterson at (309) 654-2241, extension 3609.

Respectfully,

George P. Barnes Jr

Joel P. Dimmette, Jr.
Site Vice President
Quad Cities Nuclear Power Station

Enclosure

cc: Regional Administrator – NRC Region III
NRC Senior Resident Inspector – Quad Cities Nuclear Power Station

bcc: Project Manager – NRR
Office of Nuclear Facility Safety, - IDNS
Senior Reactor Analyst – IDNS
Resident Inspector - IDNS
Manager of Energy Practice – Winston and Strawn
Director, Licensing and Compliance – ComEd
Vice President, Regulatory Services– ComEd
ComEd Document Control Desk Licensing (Hard Copy)
ComEd Document Control Desk Licensing (Electronic Copy)
W. Leech – MidAmerican Energy Company
D. Tubbs – MidAmerican Energy Company
Regulatory Assurance Manager – Dresden Nuclear Power Station
Regulatory Assurance Manager – Quad Cities Nuclear Power Station
NRC Coordinator – Quad Cities Nuclear Power Station
NSRB Site Coordinator – Quad Cities Nuclear Power Station
Site Vice President – Quad Cities Nuclear Power Station
Station Manager – Quad Cities Nuclear Power Station
SVP Letter File

NRC FORM 366 (6-1998) U.S. NUCLEAR REGULATORY COMMISSION

LICENSEE EVENT REPORT (LER)

APPROVED BY OMB NO. 3150-0104
EXPIRES 06/30/2001

Estimated burden per response to comply with this mandatory information collection request: 50 hrs. Reported lessons learned are incorporated into the licensing process and fed back to industry. Forward comments regarding burden estimate to the information and Records Management Branch (t-6 f33), U.S. Nuclear Regulatory Commission, Washington, DC 20555-0001, and to the Paperwork Reduction Project (3150-0104), Office Of Management And Budget, Washington, DC 20503. If an information collection does not display a currently valid OMB control number, the NRC may not conduct or sponsor, and a person is not required to respond to, the information collection.

FACILITY NAME (1)	DOCKET NUMBER (2)	PAGE (3)
Quad Cities Nuclear Power Station, Unit 1	05000254	1 of 3

TITLE (4)
Inadequate Calibration of Post Accident Torus Temperature Monitors

EVENT DATE (5)			LER NUMBER (6)			REPORT DATE (7)			OTHER FACILITIES INVOLVED (8)	
MONTH	DAY	YEAR	YEAR	SEQUENTIAL NUMBER	REVISION NUMBER	MONTH	DAY	YEAR	FACILITY NAME	DOCKET NUMBER
									Quad Cities Nuclear Power Station, Unit 2	05000265
10	04	2000	2000	002	00	11	02	2000	FACILITY NAME: N/A	DOCKET NUMBER: 05000

OPERATING MODE (9)	1	THIS REPORT IS SUBMITTED PURSUANT TO THE REQUIREMENTS OF 10 CFR § (Check one or more) (11)							
POWER LEVEL (10)	100		20.2201(b)		20.2203(a)(2)(v)	X	50.73(a)(2)(i)		50.73(a)(2)(viii)
			20.2203(a)(i)		20.2203(a)(3)(i)		50.73(a)(2)(ii)		50.73(a)(2)(x)
			20.2203(a)(2)(i))		20.2203(a)(3)(ii)		50.73(a)(2)(iii)		73.71
			20.2203(a)(2)(ii)		20.2203(a)(4)		50.73(a)(2)(iv)		OTHER
			20.2203(a)(2)(iii)		50.36(c)(1)		50.73(a)(2)(v)		Specify in Abstract below or in NRC Form 366A
			20.2203(a)(2)(iv)		50.36(c)(2)		50.73(a)(2)(vii)		

LICENSEE CONTACT FOR THIS LER (12)

NAME	TELEPHONE NUMBER (Include Area Code)
Charles Peterson, Regulatory Assurance Manager	(309) 654-2241 ext 3609

COMPLETE ONE LINE FOR EACH COMPONENT FAILURE DESCRIBED IN THIS REPORT (13)

CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO EPIX		CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO EPIX

SUPPLEMENTAL REPORT EXPECTED (14)				EXPECTED SUBMISSION DATE (15)	MONTH	DAY	YEAR
X	YES (If yes, complete EXPECTED SUBMISSION DATE).		NO		12	04	00

ABSTRACT (Limit to 1400 spaces, i. e., approximately 15 single-spaced typewritten lines) (16)

At 1630 hours on October 4, 2000, with Unit 1 at 100% power and Unit 2 at 83% power, it was determined that a full loop calibration was not being performed on the torus water temperature recorders (TR) 1(2)-1640-200A and 200B as required by Technical Specifications. Technical Specification Table 4.2.F.1, "Accident Monitoring Instrumentation Surveillance Requirements," requires that the torus water temperature post-accident monitors receive a channel calibration once every 18 months. Although the recorders were being calibrated as required, no calibrations were being performed on the other instruments in the loop. These instruments included signal conditioners, summers and temperature elements.

The cause of this event is being investigated. A supplemental report will be issued upon completion of the root cause report.

On October 5, 2000, the loops on Unit 1 and Unit 2 were calibrated. Although the as-found condition of two of the 16 Unit 2 temperature transmitters was outside acceptable limits, the cumulative effect on the loop output was found to be within acceptable limits. In addition, daily channel checks are performed by Operations that compare the average temperature indicated on TR 1640-200A and 200B with torus temperature indicated by other instrumentation and confirm agreement within 10°F. Therefore, the safety significance of this event is minimal.

PLANT AND SYSTEM IDENTIFICATION:

General Electric - Boiling Water Reactor - 2511 MWt rated core thermal power
Energy Industry Identification System (EIIS) Codes are identified in the text as [XX] and are obtained from IEEE Standard 805-1984, IEEE Recommended Practice for System Identification in Nuclear Power Plants and Related Facilities.

EVENT IDENTIFICATION:

Inadequate Calibration of Post Accident Torus Temperature Monitors

A. PLANT CONDITIONS PRIOR TO EVENT:

Unit: 1	Event Date: October 4, 2000	Event Time: 1630 hours
Reactor Mode: 1	Mode Name: Power Operation	Power Level: 100%
Unit: 2	Event Date: October 4, 2000	Event Time: 1630 hours
Reactor Mode: 1	Mode Name: Power Operation	Power Level: 83%

Power Operation (1) - Mode switch in the RUN position with average reactor coolant temperature at any temperature.

B. DESCRIPTION OF EVENT:

At 1630 hours on October 4, 2000, with Unit 1 at 100% power and Unit 2 at 83% power, it was determined that a full loop calibration was not being performed on the torus water temperature recorders [TR] (TR) 1(2)-1640-200A and 200B as required by Technical Specifications. Technical Specification Table 4.2.F.1, "Accident Monitoring Instrumentation Surveillance Requirements," requires that the torus water temperature post-accident monitors [IP] receive a channel calibration once every 18 months. Although the recorders were being calibrated as required, no calibrations were being performed on the other instruments in the loop. These instruments included signal conditioners, summers and temperature elements [TE].

C. CAUSE OF EVENT:

The cause of this event is being investigated. A supplemental report will be issued upon completion of the root cause report.

D. SAFETY ANALYSIS

On October 5, 2000, the other instruments in the loops on Unit 1 and Unit 2 were calibrated. The temperature transmitters provide eight temperature signals to the average temperature indication for each temperature recorder. The as-found conditions of all of the individual components were evaluated and the cumulative effect on the loop output was found to be within acceptable limits. This indicates that the average temperature indications on TR 1640-200A and 200B remained within tolerance since the last calibration performed during installation in 1991.

The as-found conditions of two of the Unit 2 temperature transmitters [TT] were outside acceptable limits. The

components were restored to the required tolerances. The remaining transmitters and summers were within tolerance. The as-found condition of the two transmitters did not prevent the loop calibration from being within acceptable limits.

In addition, daily channel checks are performed by Operations that compare the average temperature indicated on TR 1640-200A and 200B with torus temperature indicated by other instrumentation and confirm agreement within 10°F. A review of the history indicates that TR 1640-200A and 200B were not found to exceed this limit during that same period. Therefore, the safety significance of this event is minimal.

E. CORRECTIVE ACTIONS:

On October 5, 2000, the torus temperature loops were calibrated.

Determination of the root case for this event is in progress. A supplemental report will be submitted upon completion of the root cause determination.

F. PREVIOUS OCCURRENCES:

Previous occurrences for this event will be provided in a supplemental report.

G. COMPONENT FAILURE DATA:

There were no component failures associated with this event.