

Robert L. Palla  
Senior Reactor Engineer

Education

M.S., Mechanical Engineering, University of Maryland, 1981

B.S., Mechanical Engineering, University of Maryland, 1975

Employment

U.S. Nuclear Regulatory Commission, 1981 - present

Performs technical evaluations of license applications and policy issues in the areas of severe accident progression and phenomena, containment performance, offsite consequences, and risk management.

Served as the lead on the following:

- development of staff guidelines for applying risk-informed decisionmaking in license amendment reviews (SECY-99-246)
- review of industry severe accident management guidelines (SAMG) and program implementation
- evaluation of severe accident mitigation alternatives for Watts Bar, Calvert Cliffs, Oconee, GE ABWR, CE System80+, and Westinghouse AP600
- review of revised Westinghouse methodology for assessing core damage
- review of Level 2 and 3 PRA and severe accident design aspects of advanced reactors (GE ABWR, CE System80+, Westinghouse AP600)
- review of EPRI Utility Requirements Document, Appendix 1A, PRA Ground Rules and Assumptions

Task force member and contributing author on:

- Risk assessment of severe accident induced steam generator tube rupture (NUREG-1570)
- Technical framework portion of revised reactor oversight process (SECY-99-007)

Member of Reactor Safety Team, USNRC Emergency Operations Center

Professional Societies

American Society of Mechanical Engineers

Publications

Use of PRA in Accident Management, ASME Winter Meeting, invited paper, 1991

Regulatory Approach to Enhanced Human Performance During Accidents, ANS Winter Meeting, invited paper, 1990

The NRC Regulatory Program for Accident Management, PSA 89, 1989