

James A. FitzPatrick
Nuclear Power Plant
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Michael J. Colomb
Site Executive Officer

September 6, 2000
JAFP-00-0206

United States Nuclear Regulatory Commission
Attn: Document Control Desk
Mail Station P1-137
Washington, D.C. 20555

Subject: James A. FitzPatrick Nuclear Power Plant
Docket No. 50-333
**Additional Information Regarding Excess Flow Check Valve Testing
Relief Request VRR-07**

Reference: Letter from M. Colomb (NYPA) to U. S. Nuclear Regulatory Commission
(USNRC), dated April 24, 2000

Dear Sir:

In the referenced letter, NYPA submitted for your review and approval a request for relief from the Code requirements for testing excess flow check valves. Based on communications with NRC staff (via e-mail) the attached information is being provided to supplement the information in the referenced request.

If you have any questions, please contact Mr. George Tasick at (315) 349-6372.

Very truly yours,

A handwritten signature in black ink, appearing to read 'M. Colomb', with a long horizontal stroke extending to the right.

MICHAEL J. COLOMB

MJC:RS:las

cc: next page

A047

cc: U.S. Nuclear Regulatory Commission
Regional Administrator
475 Allendale Road
King of Prussia, PA 19406

Office of the Resident Inspector
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Attachment to JAFP-00-0206

RELIEF REQUEST VRR-07 ADDITIONAL INFORMATION

New York Power Authority

JAMES A. FITZPATRICK NUCLEAR POWER PLANT

Docket No. 50-333

DPR-59

RELIEF REQUEST VRR-07 ADDITIONAL INFORMATION

The instrument line failure analysis is bounded in the FSAR by the Main Steam Line Break (MSLB) analysis, Section 14.6.1.5. The Authority also performed an analysis that compared the dose release of the instrument line break to that of the MSLB. Results are summarized below. The results are bounded by the topical report. Also, see FSAR section 16.3.2.5, which states "The 1/4 in. orifice was selected to reduce the blowdown in the event of a break in a 1 in. section of instrument pipe outside the primary containment to a rate that will not result in overpressurizing the secondary containment. It is assumed that the Standby Gas Treatment System is operative and that plant shutdown and cooldown occur after the line break."

JAF Offsite and Control Room Radiation Exposures Following a Postulated Instrument Line Break (Note: Values are expressed in REM)		
	Thyroid	Whole Body
Site Boundary	8.49E-02	4.00E-04
Low Population Zone	2.48E-02	1.12E-04
Control Room	4.03E+00	1.33E-03

Instrument Line Break Bases: Primary coolant activity at 0.2uci/g I-131 DE. RCS Coolant released in 2 hours = 15,600 lb. RCS Coolant released in 8 hours = 30,000 lb. Fractional release rate to atmosphere = 0.6 per hour. Ground level release.

JAF Offsite and Control Room Radiation Exposures Following a Design Basis Main Steam Line break (Note: Values are expressed in REM)		
	Thyroid	Whole Body
Site Boundary	7.14E-01	9.06E-03
Low Population Zone	7.99E-02	1.11E-03
Control Room	2.11E+01	1.86E-02

MSLB Bases: primary coolant activity at 0.2uci/g I-131 DE. RCS Coolant released = 120,000 lb liquid. Fractional release rate to atmosphere = 3 per hour. Ground level release.

The instrument lines are separated physically by the containment itself. The "A" division generally runs along the east side of the containment and the "B" division generally runs along the west side of the containment. Pertinent sections in the FSAR that describe the arrangement and separation are 7.2.3.9 - RPS Instrumentation, 7.3.4.9 - Containment Isolation Instrumentation, and 7.4.4 - ECCS Instrumentation.