

August 24, 2000

Mr. Oliver D. Kingsley, President
Nuclear Generation Group
Commonwealth Edison Company
Executive Towers West III
1400 Opus Place, Suite 500
Downers Grove, IL 60515

SUBJECT: DRESDEN - ISSUANCE OF AMENDMENT (TAC NO. MA5414)

Dear Mr. Kingsley:

The U.S. Nuclear Regulatory Commission (Commission) has issued the enclosed Amendment No. 178 to Facility Operating License No. DPR-19 for Dresden, Unit 2. The amendment is in response to your application dated April 30, 1999.

The amendment revises the expiration date of the operating license to allow 40 years of operation from the original date of issuance of the Dresden, Unit 2, Provisional Operating License.

A copy of the Safety Evaluation is also enclosed. The Notice of Issuance will be included in the Commission's biweekly Federal Register notice.

Sincerely,

/RA/

Lawrence W. Rossbach, Project Manager, Section 2
Project Directorate III
Division of Licensing Project Management
Office of Nuclear Reactor Regulation

Docket No. 50-237

Enclosures: 1. Amendment No. 178 to DPR-19
2. Safety Evaluation

cc w/encls: See next page

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UNITED STATES
NUCLEAR REGULATORY COMMISSION

WASHINGTON, D.C. 20555-0001

August 24, 2000

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Docket No. 50-237

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2. Safety Evaluation

cc w/encls: See next page

O. Kingsley
Commonwealth Edison Company

Dresden Nuclear Power Station
Units 2 and 3

cc:

Commonwealth Edison Company
Site Vice President - Dresden
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Morris, Illinois 60450-9765

Commonwealth Edison Company
Dresden Station Manager
6500 N. Dresden Road
Morris, Illinois 60450-9765

U.S. Nuclear Regulatory Commission
Dresden Resident Inspectors Office
6500 N. Dresden Road
Morris, Illinois 60450-9766

Regional Administrator
U.S. NRC, Region III
801 Warrenville Road
Lisle, Illinois 60532-4351

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Office of Nuclear Facility Safety
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Chicago, Illinois 60690-0767



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555-0001

COMMONWEALTH EDISON COMPANY

DOCKET NO. 50-237

DRESDEN NUCLEAR POWER STATION, UNIT 2

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 178
License No. DPR-19

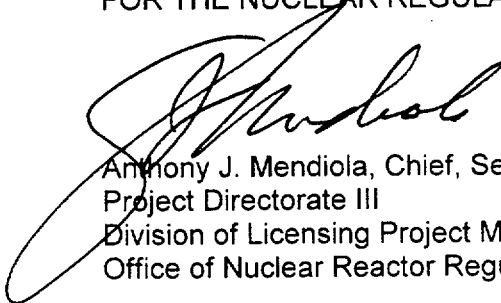
1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment by the Commonwealth Edison Company (the licensee) dated April 30, 1999, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.
2. Accordingly, the license is amended as indicated in the attachment to this license amendment; and paragraph 2.C.(2) of Facility Operating License No. DPR-19 is hereby amended to read as follows:

(2) Technical Specifications

The Technical Specifications contained in Appendix A, as revised through Amendment No. 178 , are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of the date of its issuance.

FOR THE NUCLEAR REGULATORY COMMISSION



Anthony J. Mendiola, Chief, Section 2
Project Directorate III
Division of Licensing Project Management
Office of Nuclear Reactor Regulation

Attachment:
Changes to the Operating License

Date of Issuance: August 24, 2000

ATTACHMENT TO LICENSE AMENDMENT NO. 178

FACILITY OPERATING LICENSE NO. DPR-19

DOCKET NO. 50-237

Replace the following page of Operating License No. DPR-19 with the attached revised page. The revised page is identified by amendment number and contains a line in the margin indicating the area of the change.

REMOVE

Page 5

INSERT

Page 5

- I. This license is effective as of the date of issuance and shall expire at midnight on December 22, 2009.

FOR THE NUCLEAR REGULATORY COMMISSION

ORIGINAL SIGNED BY:

Thomas E. Murley, Director
Office of Nuclear Reactor Regulation

Attachment:

Appendix A - Technical Specifications

Appendix B - Additional Conditions

Date of Issuance: February 20, 1991



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555-0001

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION
RELATED TO AMENDMENT NO. 178 TO FACILITY OPERATING LICENSE NO. DPR-19

COMMONWEALTH EDISON COMPANY

DRESDEN NUCLEAR POWER STATION, UNIT 2

DOCKET NO. 50-237

1.0 INTRODUCTION

By letter dated April 30, 1999, Commonwealth Edison Company (ComEd, the licensee) submitted a request for a change to the operating license for its Dresden Nuclear Power Station, Unit 2. The amendment would extend the expiration date for the Dresden, Unit 2, operating license from January 10, 2006, to December 22, 2009.

Dresden, Unit 2, is currently licensed to operate for 40 years commencing with the issuance of its construction permit on January 10, 1966. A Provisional Operating License for Dresden, Unit 2, was issued on December 22, 1969. The Dresden, Unit 2, Provisional Operating License was replaced by a full-term Facility Operating License on February 20, 1991. The extended date for termination of the operating license would be 40 years after issuance of the Provisional Operating License. This action would extend the period of operation to the full 40 years, from the date of the Provisional Operating License, as provided by the Atomic Energy Act and the *Code of Federal Regulations*.

2.0 BACKGROUND

Section 103.c of the Atomic Energy Act of 1954 provides that a license is to be issued for a specified period not exceeding 40 years. The regulations in 10 CFR 50.51 specify that each license will be issued for a fixed period of time not to exceed 40 years from date of issuance. Also, 10 CFR 50.56 and 10 CFR 50.57 allow the issuance of an operating license pursuant to 10 CFR 50.51 after the construction of the facility has been substantially completed, in conformity with the construction permit and when other provisions specified in 10 CFR 50.57 are met. The currently licensed term for Dresden, Unit 2, is 40 years, commencing with the issuance of the construction permit on January 10, 1966. Accounting for the time that was required for plant construction, this represents an effective operating license term of approximately 36 years for Unit 2. Consistent with Section 103.c of the Atomic Energy Act and 10 CFR 50.51, 10 CFR 50.56 and 10 CFR 50.57 of the Commission's regulations, the licensee, by its application of April 30, 1999, seeks an extension that would permit the unit to operate for the full 40-year design-basis lifetime. The proposed extension is also consistent with the Commission policy stated in a memorandum dated August 16, 1982, from William Dircks, Executive Director for Operations, to the Commissioners, and as evidenced by the issuance of over 50 such extensions to other licensees.

3.0 EVALUATION

The NRC staff has evaluated the safety issues associated with issuance of the proposed license amendment that would allow approximately 4 additional years of plant operation. The issues addressed consist of general aging of plant structures and equipment, additional radiation exposure to the facility operating staff, impacts on the offsite population, and nonradiological impacts. The impact of additional radiation exposure to the facility operating staff and the impact on the general population in the vicinity of Dresden, Unit 2, and the nonradiological impacts are addressed in the staff's Environmental Assessment dated June 1, 2000 (65 FR 36474).

3.1 Reactor Vessel

The design of the reactor vessel and its internals considered the effects of 40 years of operation equivalent to 32 effective full power years (EFPY), with the exception of the core shroud repair which is designed for 30 EFPY. As of January 1, 1999, Dresden, Unit 2, had accumulated 16.7 EFPY, thus, the vessel and internals will not reach their design lifetime before the extended license expiration date of December 22, 2009.

Thermal and loading cycles over a 40 year lifetime were considered in the design of the Dresden, Unit 2, reactor vessel. Table 3.9-1 of the Dresden Updated Final Safety Analysis Report (UFSAR) compares the predicted and allowable thermal and loading cycles for the Dresden reactor vessel. Based upon cycles to date, allowable thermal and loading cycles are not predicted to be exceeded before the extended license expiration date of December 22, 2009.

Generic Letter (GL) 88-11 recommended that licensees use the methods in Regulatory Guide (RG) 1.99, Revision 2, "Radiation Embrittlement of Reactor Vessel Materials," to predict the effect of neutron irradiation on reactor vessel materials. GL 92-01, Revision 1, "Reactor Vessel Structural Integrity," requested information to confirm that licensees satisfy the requirement for ensuring reactor vessel integrity. GL 92-01, Revision 1, Supplement I, "Reactor Vessel Structural Integrity," requested licensees to perform a review and assessment of their reactor pressure vessel structural integrity relative to the requirements of Appendices G and H to 10 CFR Part 50, 10 CFR 50.60, and 10 CFR 50.61 requirements.

In response to these requests, ComEd submitted the results of their updated assessments of the reactor pressure vessel structural integrity of Dresden, Unit 2. The NRC staff reviewed and accepted (References 1, 2, 3, 4 and 5) ComEd's responses to these GLs. These analyses were consistent with the plants 40 year (32 EFPY) design lifetime. Also, by References 6 and 7, ComEd submitted revised Reactor Vessel Pressure-Temperature limits that will be used until the 32 EFPY reactor vessel fluence estimate is updated in 2001. The NRC staff accepted use of the current fluence estimate until November 30, 2001 (Reference 8). Hence, previous NRC conclusions that the reactor pressure vessel structural integrity was properly assessed are not changed by the proposed license extension to recapture the construction period and a separate assessment for the purpose of this proposed amendment will not be performed.

Periodic reactor vessel inservice inspection and testing requirements provide further assurance that any degradation will be identified in a timely manner. A comprehensive vessel materials

surveillance program is maintained in accordance with 10 CFR Part 50, Appendix H, which ensures that fracture toughness requirements of 10 CFR Part 50, Appendix G, are met. As stated in the UFSAR, reactor vessel surveillance capsules are periodically removed for Charpy V-notch and tensile strength tests. By Reference 5, the NRC staff noted that the licensee's reactor pressure vessel integrity data for Dresden, Unit 2, was complete.

The staff concludes that there is reasonable assurance that the reactor pressure vessel will, for the proposed license term extension, be in conformance with the applicable provisions of the rules and regulations of the Commission and the Dresden, Unit 2, license.

3.2 Structures

Critical plant structures (the primary containment, the reactor building, and the other safety-related structures such as the turbine building including the control room) for Dresden, Unit 2, and supports were designed to various codes and standards applicable at the time of plant construction. The design basis, fabrication, construction and quality assurance criteria for the plant were reviewed and found acceptable by the NRC staff in its original Safety Evaluation Report dated October 17, 1969. Industrial experience with such structures and supports confirms that a service life in excess of 40 years can be anticipated. The NRC also completed an Integrated Plant Safety Assessment Report (IPSAR) for Dresden, Unit 2, under the Systematic Evaluation Program and published the results of that assessment in NUREG-0823 dated February 1983. Issues remaining from the IPSAR were reviewed and found acceptable in the safety evaluation for issuance of a full-term 40-year operating license for Dresden, Unit 2, on February 20, 1991. These previous assessments remain valid because the 40-year plant lifetime considered in these previous assessments is not changed by the proposed change in the license expiration date.

Dresden, Unit 2, has an established maintenance and inspection program by which it regularly inspects concrete surfaces and protective coatings under 10 CFR 50.65 (the Maintenance Rule) and other inservice inspection (ISI) requirements to assess the condition of structures and complete required repairs in accordance with the applicable codes. Such periodic inspection and testing provide reasonable assurance that structural integrity remains adequate throughout the operating life of the plant, including the proposed extension period.

The staff concludes that there is reasonable assurance that the critical plant structures will, for the proposed license term extension, be in conformance with the applicable provisions of the rules and regulations of the Commission and the Dresden, Unit 2, license.

3.3 Mechanical Equipment

With regard to equipment lifetime, Dresden, Unit 2, was designed, licensed, and constructed for a 40-year service life. This does not mean that some components will not wear out during the plant's lifetime. Design features were incorporated which provide for inspectability and testing of structures, systems and components during this lifetime. Surveillance and maintenance practices were implemented in accordance with 10 CFR 50.65 (the Maintenance Rule), the requirements for inservice testing of pumps and valves in accordance with the ASME Code as set forth in 10 CFR 50.55a, and Technical Specifications. These activities provide assurance that any degradation in plant safety equipment will be identified and corrected to provide safe

operation during the life of the plant, including the period of the proposed license term extension.

3.4 Environmental Qualification of Electrical Equipment

Aging analysis has been performed for all safety-related electrical equipment in accordance with 10 CFR 50.49, "Environmental Qualification of Electric Equipment Important to Safety for Nuclear Power Plants," identifying qualified lifetimes for this equipment. These lifetimes have been incorporated into plant equipment maintenance and replacement practices to ensure that all safety-related electrical equipment remains qualified and available to perform its safety-related function regardless of the overall age of the plant. The staff's safety evaluation for environmental qualification of safety-related electrical equipment for Dresden, Unit 2, dated February 12, 1986, documents Dresden's compliance with 10 CFR 50.49 requirements. This previous assessment by the staff is not changed by the proposed license term extension.

4.0 SUMMARY OF FINDINGS

The staff published its original safety evaluation for Dresden, Unit 2, on October 17, 1969. While changes have been made to the plant design since the original plant construction was completed, each of these changes that involved an unreviewed safety question has been reviewed and approved by the staff with the details being documented in the staff's related safety evaluation. Further, as required by 10 CFR 50.71(e), these changes and their effect on accident analyses, if any, are routinely updated in the UFSAR. Based on this ongoing review process, the staff has not identified any concerns associated with approval of the proposed amendment to extend the expiration date of the license that are not already addressed by licensee commitments, operating procedures, and license requirements.

The NRC staff concluded in the Environmental Assessment that the annual radiological effects during the additional years of operation that would be authorized by the proposed license amendment are not more than were previously estimated in the Final Environmental Statement, and are acceptable.

The staff concludes from its considerations of the design, operation, testing, and monitoring of the mechanical equipment, structures, and the reactor vessel that an extension of the operating license for Dresden, Unit 2, to a 40-year service life is consistent with the UFSAR, Safety Evaluation Reports, and submittals made by the licensee, and that there is reasonable assurance that the unit will be able to continue to operate safely for the additional period authorized by this amendment.

In summary, the staff finds that the extension of the operating license for Dresden, Unit 2, to allow a 40-year service life is consistent with the Final Environmental Statement and the Safety Evaluation Report for Dresden, Unit 2, and that the Commission's previous findings are not changed.

5.0 STATE CONSULTATION

In accordance with the Commission's regulations, the Illinois State official was notified of the proposed issuance of the amendment. The State official had no comments.

6.0 ENVIRONMENTAL CONSIDERATION

Pursuant to 10 CFR 51.21, 10 CFR 51.32, and 10 CFR 51.35, an environmental assessment and finding of no significant impact was published in the *Federal Register* on June 8, 2000 (65 FR 36474) for this amendment. Accordingly, based upon the environmental assessment, the Commission has determined that issuance of this amendment will not have a significant effect on the quality of the human environment.

7.0 CONCLUSION

The Commission has concluded, based on the considerations discussed above, that (1) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, (2) such activities will be conducted in compliance with the Commission's regulations, and (3) the issuance of the amendment will not be inimical to the common defense and security or to the health and safety of the public.

8.0 REFERENCES

1. NRC letter to T. J. Kovach (ComEd) enclosing Amendment No. 114 to Dresden, Unit 2, Technical Specifications and documenting the NRC review of ComEd's responses to GL 88-11, dated September 5, 1991.
2. NRC letter to L. A. England documenting the NRC review of BWR Owners' Group Topical Report NEDO-32205, Revision 1, "10CFR50 Appendix G Equivalent Margin Analysis for Low Upper Shelf Energy in BWR/2 Through BWR/6 Vessels," dated December 12, 1993.
3. NRC letter to D. L. Farrar (ComEd) documenting the NRC review of ComEd's responses to GL 92-01, Revision 1, dated April 14, 1994.
4. NRC letter to I. Johnson (ComEd) documenting the NRC review of ComEd's responses to GL 92-01 Revision 1, Supplement 1, dated April 21, 1997.
5. NRC letter to O. D. Kingsley (ComEd) documenting the NRC review of ComEd's response to a request for additional information on ComEd's response to GL 92-01 Revision 1, Supplement 1, dated September 13, 1999.
6. ComEd letter to NRC submitting a proposed a Technical Specifications Amendment to Modify P-T Limits for a Maximum of 32 EFPY, dated February 23, 2000.
7. ComEd letter to NRC on Interim Approval of P-T Limits, dated July 17, 2000.
8. NRC Safety Evaluation on Dresden Fluence Estimate, dated July 26, 2000.

Principal Contributor: L. Rossbach, NRR

Date: August 24, 2000