



Davis-Besse Nuclear Power Station
5501 North State Route 2
Oak Harbor, Ohio 43449-9760

Guy G. Campbell
Vice President - Nuclear

419-321-8588
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Docket Number 50-346

License Number NPF-3

Serial Number 2669

August 22, 2000

United States Nuclear Regulatory Commission
Document Control Desk
Washington, D. C. 20555-0001

Subject: Inservice Inspection Summary Report of Twelfth Refueling Outage Activities for the
Davis-Besse Nuclear Power Station - Unit 1

Ladies and Gentlemen:

In accordance with the 1986 Edition of the American Society of Mechanical Engineers (ASME) Boiler and Pressure Vessel Code Section XI, IWA-6000, enclosed is the summary report covering inservice inspection activities, repairs, and replacements completed through the end of the twelfth refueling outage at the Davis-Besse Nuclear Power Station - Unit 1.

Should you have any questions concerning this report, please contact Mr. David H. Lockwood, Manager - Regulatory Affairs, at (419) 321-8450.

Sincerely yours,

GMW/s

Enclosures

A047

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COMMITMENT LIST

The following list identifies those actions committed to by the Davis-Besse Nuclear Power Station in this document. Any other actions discussed in the submittal represent intended or planned actions by Davis-Besse. They are described only as information and are not regulatory commitments. Please notify the Manager - Regulatory Affairs (419-321-8450) at Davis-Besse of any questions regarding this document or associated regulatory commitments.

COMMITMENTS

None

DUE DATE

N/A

ISI SUMMARY REPORT

REFUELING OUTAGE.: 12R

OUTAGE START DATE: April 1, 2000

OUTAGE COMPLETION DATE: May 18, 2000

REPORTING DATE: August 15, 2000

DAVIS -BESSE NUCLEAR POWER STATION

5501 North State Route #2

Oak Harbor

Ottawa County, Ohio 43449

OWNER: FIRST ENERGY CORPORATION

76 South Main St.

Akron, Ohio 44308

NRC DOCKET NUMBER: 50-346

OPERATING LICENSE: NPF-3

COMMERCIAL OPERATION: November 21, 1977

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ABSTRACT

This report provides a summary of the Inservice Inspection nondestructive examinations, Steam Generator Tubing eddy current examinations, pressure tests, and repairs/replacements performed during Cycle 12 and the 12th Refueling Outage at the Davis Besse Nuclear Power Station.

The nondestructive examination and pressure test program was performed by First Energy Nuclear Operating Company (FENOC) and Framatone Technologies personnel from June, 1998 through May, 2000. This program is in compliance with the requirements of 10 CFR 50.55a, the 1986 Edition of the ASME Boiler and Pressure Vessel Code Section XI, and the Davis Besse Unit #1 Technical Specifications as described in the Second Ten Year Inservice Inspection Program Plan. These examinations complete the examinations required for the Second Inservice Inspection Interval.

The NDE examinations consisted of Ultrasonic (UT), Liquid Penetrant (PT), Magnetic Particle (MT), and Visual (VT-1, VT-2, VT-3). A total of 286 ASME Class 1 and 2 components were examined. This included 5 augmented examinations and 56 preservice examinations. All examinations met the acceptance criteria of ASME Section XI.

Pressure tests consisted of system leakage, functional, inservice, and hydrostatic tests. In addition to tests conducted in accordance with the Pressure Test Program, tests were also conducted following repairs/replacements. In total, 60 ASME Class 1 and 2 pressure tests were successfully conducted.

One Hundred (100) percent of the tubing in both Once Through Steam Generators was eddy current examined. The eddy current results from the 12th Refueling Outage inspection were consistent in both number and size of indications of degradation compared to the results of the previous inspection, (April, 1998), meaning new forms of degradation are not emerging and pre-existing degradation rates are not increasing. A total of 20 tubes in Steam Generator 1-1 and 23 tubes in Steam Generator 1-2 were plugged.

All repairs/replacements performed at Davis Besse are controlled via Davis-Besse's Work Order system. Copies of Davis-Besse's Repair/Replacement NIS-2 forms for ASME Class 1 and 2 repairs/replacements are included in this report. All other records including testing and examination records are available in Davis-Besse's Records Management System for review.

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FORM NIS-1 OWNER'S DATA REPORT FOR INSERVICE INSPECTIONS

As required by the Provisions of the ASME Code Rules

1. Owner FIRST ENERGY CORPORATION, 46 S. MAIN ST., AKRON, OHIO 44308

(Name and Address of Owner)

2. Plant DAVIS BESSE NUCLEAR POWER STATION, OAK HARBOR, OHIO 43449

(Name and Address of Plant)

3. Unit #14. Owner Certificate of Authorization (if required) N/A5. Commercial Service Date 11/21/776. National Board Number for Unit N/A

7. Components Inspected

STEAM GENERATOR TUBING

Component or Appurtenance	Manufacturer or Installer	Manufacturer Or Installer Serial No.	State or Providence No.	National Board No.
OTSG 1-1	BABCOCK & WILCOX	620-0014-055-11	N/A	N-158
OTSG 1-2	BABCOCK & WILCOX	620-0014-055-12	N/A	N-159

Note: Supplemental sheets in form of lists, sketches, or drawings may be used provided (1) size is 8 1/2 in. x 11 in., (2) information in items 1 through 6 on this data report is included on each sheet, and (3) each sheet is numbered and the number of sheets recorded at the top of this form.

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FORM NIS-1 (back)

8. Examination Dates APR 2000 to APR 2000 9. Inspection Interval from SEPT 1990 to SEPT 2000
10. Applicable Editions of Section XI 1986 Addenda None
11. Abstract of Examinations. Include a list of examinations and a statement concerning status of work required for current interval.
SEE 2000 EDDY CURRENT EXAMINATION REPORT
12. Abstract of Conditions Noted.
SEE ATTACHED
13. Abstract of Corrective Measures Recommended and Taken
SEE ATTACHED

Certificate of Authorization No. (if applicable) N/A

Expiration Date N/A

We certify that the statements made in this report are correct and the examinations and corrective measures taken conform to the rules of the ASME Code, Section XI.

Date 8-10-00 Signed **FIRST ENERGY NUCLEAR
OPERATING CO.**

By Michael A. Gephling

CERTIFICATE OF INSERVICE INSPECTION

I, the undersigned, holding a valid commission issued by the National Board of Boiler and Pressure Vessel Inspectors and/or the State or Province of OHIO and employed by Factory Mutual Ins. Co. of Johnston, Rhode Island have inspected the components described in this Owners' Data Report during the period APRIL 2000 to APRIL 2000, and state that to the best of my knowledge and belief, the Owner has performed examinations and taken corrective measures described in this Owners' Data Report in accordance with the requirements of the ASME Code, Section XI.

By signing this certificate neither the Inspector nor his employer makes any warranty, expressed or implied, concerning the examinations and corrective measures described in this Owner's Date Report. Furthermore, neither the Inspector nor his employer shall be liable in any manner for any personal injury or property damage or a loss of any kind arising from or connected with this inspection.

Date **AUGUST 10, 2000**

Thomas H. Laps
Inspector's Signature

Commissions

NB9330 "NIA" Ohio COMM.
National Board, State, Province and No.

STEAM GENERATOR EDDY CURRENT EXAMINATION RESULTS

The tubing in the Once Through Steam Generators (OTSG's) at the Davis Besse Nuclear Power Station Unit #1 was examined in April, 2000, during the 12th Refueling Outage. The examinations were conducted by Framatone Technologies to meet the requirements of the Second Ten Year Inservice Inspection Plan and the Davis Besse Unit #1 Technical Specifications. All examinations met the requirements of techniques qualified in accordance with the PWR Steam Generator Examination Guidelines.

The eddy current examinations were performed utilizing a Bobbin Coil, a Rotating Plus Point and Pancake Coil (RPP), and a Rotating Pancake Coil (RPC). The bobbin coil technique was used to perform the standard ASME code examination for flaw detection. This technique was applied to the complete length of all in-service tubes. The rotating plus point and pancake techniques (RPP) were used to examine specific areas of interest. These areas included upper tube end roll expansion regions, tubes bordering the lane & wedge region, non-stress relieved roll transitions, lower tubesheet crevice region, and select dent locations. This technique was also used to characterize all indications reported by the bobbin technique. The pancake coil was used to provide final depth sizing of tube support plate wear indications. Both plus point and bobbin techniques were used to examine the sleeves. The rotating plus point probe was applied to the rolled regions and lower sleeve end while the bobbin probe was used to examine the unrolled portion. The rotating pancake technique was used to examine plugs.

The following examinations were performed on both Steam Generator 1-1 and Steam Generator 1-2.

- Full length bobbin examination of all in-service tubes (30,417 tubes)
- Full length bobbin examination of all sleeves (411 sleeves)
- Plus point examination of 20% of the sleeve roll expansions (83 sleeves)
- Plus point and pancake coil examination of all the non-stress relieved tube roll expansions - 11 RFO re-rolls and factory re-rolls (11 tubes)
- Pancake coil examination of 20% of the hot leg roll-tube plugs (94 plugs)
- Plus point and pancake coil examination of the tubes bordering the sleeve region (164 sleeves)
- Plus point and pancake coil examination of all of the flaw-like indications reported from bobbin (1347 indications)
- Plus point and pancake coil examination of the dent/ding indications to include all those located above the 14th tube support plate and a 20% sample of the remaining population (86 dented locations)

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- Pancake coil inspection of 500 lower tubesheet sludge pile region tubes of each Steam Generator (1000 tubes)
- Free-path verification for possible twisted tubes (4 tubes in Steam Generator 1-2)

The eddy current results from the 12th Refueling Outage inspection were consistent in both number and size of indications of degradation compared to the results of the previous inspection, (April, 1998), meaning new forms of degradation are not emerging and pre-existing degradation rates are not worst.

In the initial 21% random upper roll transition examination, one tube was discovered which had no roll expansion region. A re-analysis of the bobbin data for 100% of the upper and lower tube ends in both steam generators revealed 3 additional tubes which had no roll expansion.

A total of 43 tubes were removed from service (20 in Steam Generator 1-1 and 23 in Steam Generator 1-2). The majority of these tubes contained indications of volumetric inner granular attack for which no valid depth sizing technique exists.

All flaw-like indications detected during the 12th Refueling Outage were screened to determine if in situ pressure/leak testing was required per the Electric Power Research Institute (EPRI) screening methodologies. The results of the screening were that no tubes required testing and therefore no tubes were tested.

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As required by the Provisions of the ASME Code Rules

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(Name and Address of Owner)

2. Plant DAVIS BESSE NUCLEAR POWER STATION, OAK HARBOR, OHIO 43449

(Name and Address of Plant)

3. Unit #14. Owner Certificate of Authorization (if required) N/A5. Commercial Service Date 11/21/776. National Board Number for Unit N/A

7. Components Inspected

NDE EXAMINATIONS

Component or Appurtenance	Manufacturer or Installer	Manufacturer or Installer Serial No.	State or Providence No.	National Board No.
REACTOR VESSEL	BABCOCK & WILCOX	620-0014-51,52	N/A	N-156
OTSG 1-2	BABCOCK & WILCOX	620-0014-11	N/A	N-158
PRESSURIZER	BABCOCK & WILCOX	620-0014-59	N/A	N-157
CLASS 1 PIPING	BABCOCK & WILCOX	VARIOUS	N/A	N/A
DECAY HEAT COOLER	ATLAS INDUSTRIAL MFG.	1591	N/A	1401
CLASS 2 PIPING	ITT GRINNELL	VARIOUS	N/A	N/A

Note: Supplemental sheets in form of lists, sketches, or drawings may be used provided (1) size is 8 1/2 in. x 11 in., (2) information in items 1 through 6 on this data report is included on each sheet, and (3) each sheet is numbered and the number of sheets recorded at the top of this form.

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FORM NIS-1 (back)

8. Examination Dates **JUNE 1998** to **MAY 2000** 9. Inspection Interval from **SEPT 1990** to **SEPT 2000**

- 10. Applicable Editions of Section XI 1986 Addenda None**

- 11. Abstract of Examinations.** Include a list of examinations and a statement concerning status of work required for current interval.

SEE 2000 INSERVICE INSPECTION REPORT

- ## 12. Abstract of Conditions Noted.

SEE ATTACHED

- ### 13. Abstract of Corrective Measures Recommended and Taken

SEE ATTACHED

Certificate of Authorization No. (if applicable) N/A

Expiration Date N/A

We certify that the statements made in this report are correct and the examinations and corrective measures taken conform to the rules of the ASME Code, Section XI.

Date 8-10-00 Signed **FIRST ENERGY NUCLEAR
OPERATING COMPANY**

By Michael D. Shepherd

CERTIFICATE OF INSERVICE INSPECTION

I, the undersigned, holding a valid commission issued by the National Board of Boiler and Pressure Vessel Inspectors and/or the State or Province of OHIO and employed by Factory Mutual Ins. Co. of Johnston, Rhode Island have inspected the components described in this Owners' Data Report during the period JUNE 1998 to MAY 2000, and state that to the best of my knowledge and belief, the Owner has performed examinations and taken corrective measures described in this Owners' Data Report in accordance with the requirements of the ASME Code, Section XI.

By signing this certificate neither the Inspector nor his employer makes any warranty, expressed or implied, concerning the examinations and corrective measures described in this Owner's Date Report. Furthermore, neither the Inspector nor his employer shall be liable in any manner for any personal injury or property damage or a loss of any kind arising from or connected with this inspection.

Date **AUGUST 10, 2000**

Thomas S. Lepo
Inspector's Signature

Commissions NB 9330 "NIA" OHIO COMM.
National Board, State, Province and No.

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NONDESTRUCTIVE EXAMINATIONS

The Nondestructive Examinations were completed by FENOC and Framatone Technologies personnel from June, 1998, to May, 2000, during Cycle 12 and the 12th Refueling Outage for the Davis Besse Nuclear Power Station Unit #1. Examinations were completed on 286 ASME Class 1 and 2 components during Cycle 12 and the 12th Refueling Outage. This included 5 augmented and 56 preservice examinations.

The majority of the preservice examinations were performed on component supports following reinstallation of snubbers upon completion of snubber testing. Preservice examinations were also performed on two modifications installed during the 12th Refueling Outage. One modification replaced the Atmospheric Vent Valves and the other installed a piping tie between the High Pressure Injection System and the discharge of the Decay Heat System.

The Second 10-Year Inservice Inspection Interval ends on September 20, 2000. The examinations performed during Cycle 12 and the 12th Refueling Outage complete the examinations required by the Second 10-Year Inservice Inspection Interval. Included in the 12th Refueling Outage was the automated examination of the Reactor Vessel. This examination was performed from the inside surface of the Reactor Vessel and was performed to the requirements of Supplements 4 and 6 to Appendix VIII of the 1995 Edition, 1996 Addenda of ASME Section XI. The manual ultrasonic examination of piping was performed using procedures and personnel qualified to Supplements 2 and 3 of Appendix VIII of the 1995 Edition, 1996 Addenda of ASME Section XI as qualified under the Performance Demonstration Initiative (PDI) Program. The remaining Class 1 and 2 examinations were performed to the 1986 Edition of ASME Section XI. The attached table summarizes the examination data for the ASME Class 1 and 2 components. No indications exceeding the acceptance standards of IWB-3000 or IWC-3000 were noted during the performance of any of these examinations. An entry in the CR Number column identifies the Condition Report used to evaluate and disposition indications.

Table IWB-2500-1, Examination Category B-L-2, Item B12.20 of the 1986 Edition of the ASME Boiler and Pressure Vessel Code requires that the internal surface of one Reactor Coolant Pump Casing receive a VT-3 visual examination during the 10-year inspection interval. FENOC requested relief from disassembling a Reactor Coolant Pump for the sole purpose of performing this VT-3 visual examination. This relief was granted provided that if one of the Reactor Coolant Pumps had not been disassembled during the 10-year inspection interval, this fact would be reported in the ISI Summary Report at the end of the interval. During the Davis-Besse Second 10-Year Inspection Interval, no Reactor Coolant Pumps were disassembled. Therefore, the VT-3 visual examination requirements of Code Item B12.20 were not performed.

10 CFR 50.55a(b)(2)(ix)(A) requires that for Class MC applications, the licensee shall evaluate the acceptability of inaccessible areas where conditions exist in accessible areas that could indicate the presence of or result in degradation to such inaccessible areas. For

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each inaccessible area identified, the licensee shall provide the following in the ISI Summary Report.

- (1) A description of the type and estimated extent of degradation, and the conditions that led to the degradation;
- (2) An evaluation of each area, and the result of the evaluation, and;
- (3) A description of necessary corrective actions.

During the 12th Refueling Outage, water seepage was noted through a Thiokol sealant barrier at the interface of the outside surface of the steel Containment Vessel and the concrete surface in the base of the Annulus. Access to the steel Containment Vessel below this point is not available as it is encased in concrete. This condition was documented in Davis-Besse Condition Report 2000-0709. The following evaluation was performed in accordance with 10 CFR 50.55a(b)(2)(ix)(A) to evaluate the acceptability of the inaccessible area of the Containment Vessel.

Type and Estimated Extent of Degradation

The moisture sealing properties of the Thiokol sealant have been reduced allowing water to seep out at the interface of the outside surface of the steel Containment Vessel and the concrete surface in the base of the annulus. This area is below the groundwater table and it is believed that the source of the water is groundwater. In order for the groundwater to arrive in the annulus, it must travel through a breach in the waterproof membrane that surrounds the containment foundation, travel through at least 3'-6" of concrete, and then travel along the concrete/Containment Vessel interface until it reaches the surface. This path reduces the hydrostatic head of the groundwater (at approximately 570 ft.), causing the water to appear as a very small seepage in a few locations at the annulus floor (elevation 562.5 ft.)

Evaluation

Wet corrosion occurs by a mechanism essentially electrochemical in nature. Electrochemical corrosion of metal takes place when an electrolyte and two electrodes, an anode and a cathode, are present. Water may serve as an electrolyte. At the anode, the metal in contact with the electrolyte changes into a positively charged particle, which goes into solution or forms an oxide film. The ease with which a metal changes to a metallic ion when it is in contact with water depends on its oxidation potential or solution pressure. If the hydrogen gas produced at the cathode is removed from the cathode by reaction with oxygen to produce water molecules or by water movement (depolarization), the corrosion process continues. Factors that influence the type and quantity of metallic corrosion are the presence of protective films, moving water, and alternate wetting and drying of the metal. Agitation or movement of water increases the corrosion rate of a metal because the oxygen supply rate to the cathode and the removal rate

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of metal ions from the anode are increased. Alternate wetting and drying tends to break up the rust or oxide film. This facilitates penetration of the film by the oxygen and water and leads to increased corrosion. Corrosion occurs at normal temperature only in the presence of both oxygen and water, both of which must be replenished continually. In a hermetically sealed container, corrosion of steel will continue only until either the oxygen or water, or both are exhausted.

The presence of water between the concrete foundation and the Containment Vessel in the inaccessible areas will not degrade the Containment Vessel any more than the exposed areas based on the following. The area that is most susceptible to corrosion is at the surface of the concrete and Containment Vessel where there is continuous moisture, moving or flowing water, and plenty of oxygen. A visual examination of this area noted that there was no noticeable differences between the area at the concrete to Containment Vessel interface and the areas immediately above the interface. This condition would bound any inaccessible areas embedded in concrete.

Microbiological Induced Corrosion (MIC) is not considered as a corrosion mechanism. Groundwater is not a likely source of MIC due to the lack of airborne or waterborne organisms. The location of the between the Containment Vessel and concrete foundation does not provide a ready food source for MIC to occur. Should MIC be occurring, it would be evident at the area of the seal where the water is open to the air. As the visual examination of this area revealed that there was no noticeable differences between the area at the concrete to Containment Vessel interface and the areas immediately above the interface, MIC does not appear to be occurring.

Description of necessary corrective actions

Based on the above, water in contact with the Containment Vessel will not degrade the Containment Vessel to an unacceptable condition. The interface between the Containment Vessel and the concrete will be injection grouted to reduce the leakage of ground water to the surface.

Detailed examination results are contained in the 2000 Inservice Inspection Report which is available for review in Davis-Besse's Records Management System.

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**Inservice Inspection Abstract of ASME Class 1 and Class 2 Examinations
12th Refueling Outage**

Exam Category	Code Item	Component / Description	NDE Method	Examination Acceptable	CR Number	Exam Date	Comments
Code Class: 1							
B-A PRESSURE RETAINING WELDS IN REACTOR VESSEL							
	B01.011	CIRCUMFERENTIAL SHELL WELDS					
		RC-RPV-WR-1-(LOWER)	UT	Yes		4/21/2000	
		LOWER CIRCUMFERENTIAL SHELL WELD (BELTLINE REGION) MK 170 TO MK 171					
		RC-RPV-WR-1-(UPPER)	UT	Yes		4/21/2000	
		UPPER CIRCUMFERENTIAL SHELL WELD (BELTLINE REGION) MK 169 TO MK 170					
		RC-RPV-WR-34	UT	Yes		4/21/2000	Relief Request RR-A19 is applicable.
		LOWER SHELL TO BOTTOM HEAD CIRCUMFERENTIAL WELD MK 171 TO MK 181					
	B01.021	CIRCUMFERENTIAL HEAD WELDS					
		RC-RPV-WR-35	UT	Yes		4/21/2000	Relief Request RR-A19 is applicable.
		BOTTOM HEAD CIRCUMFERENTIAL (DISC) WELD MK 181 TO MK 6					
	B01.030	SHELL-TO-FLANGE WELD					
		RC-RPV-WR-19	UT	Yes		4/21/2000	Relief Request RR-A20 is applicable.
		SHELL TO FLANGE WELD MK 169 TO MK 7					

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**Inservice Inspection Abstract of ASME Class 1 and Class 2 Examinations
12th Refueling Outage**

Exam Category	Code Item	Component / Description	NDE Method	Examination Acceptable	CR Number	Exam Date	Comments
Code Class: 1							
B-A	PRESSURE RETAINING WELDS IN REACTOR VESSEL						
	B01.040	HEAD-TO-FLANGE WELD RC-RPV-WH-7 CLOSURE HEAD TO FLANGE CIRCUMFERENTIAL WELD MK 24 TO MK 22	UT MT	Yes Yes		4/22/2000 4/22/2000	Relief Request RR-A2 is applicable.
B-B	PRESSURE RETAINING WELDS IN VESSELS OTHER THAN REACTOR VESSELS						
	B02.011	PRESSURIZER-CIRCUMFERENTIAL SHELL-TO-HEAD WELDS RC-PZR-WP-28 LOWER HEAD TO SHELL WELD MK 6 TO MK 4/40	UT	Yes		4/22/2000	
	B02.012	PRESSURIZER-LONGITUDINAL SHELL-TO-HEAD WELDS RC-PZR-MK-4-40-6-WP-7-Y-LU Y-AXIS HTR. BNDL-SHELL LONGSEAM WELD ADJ.TO WP-28 MK 4 TO MK 40	UT	Yes		4/22/2000	
B-D	FULL PENETRATION WELDS OF NOZZLES IN VESSELS (INSPECTION PROGRAM B)						
	B03.090	REACTOR VESSEL-NOZZLE-TO-VESSEL WELDS RC-RPV-WR-10/12-W/X 28 IN. W/X AXIS INLET NOZZLE TO SHELL WELD MK 182 TO MK 169	UT	Yes		4/21/2000	Relief Request RR-A5 is applicable.
		RC-RPV-WR-10/12-X/Y 28 IN. X/Y AXIS INLET NOZZLE TO SHELL WELD MK 182 TO MK 169	UT	Yes		4/21/2000	Relief Request RR-A5 is applicable.

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**Inservice Inspection Abstract of ASME Class 1 and Class 2 Examinations
12th Refueling Outage**

Exam Category	Code Item	Component / Description	NDE Method	Examination Acceptable	CR Number	Exam Date	Comments
Code Class: 1							
B-D	FULL PENETRATION WELDS OF NOZZLES IN VESSELS (INSPECTION PROGRAM B)						
	B03.090	REACTOR VESSEL-NOZZLE-TO-VESSEL WELDS					
		RC-RPV-WR-10/12-Y/Z	UT	Yes		4/21/2000	Relief Request RR-A5 is applicable.
		28 IN. Y/Z AXIS INLET NOZZLE TO SHELL WELD MK 182 TO MK 169					
		RC-RPV-WR-10/12-Z/W	UT	Yes		4/21/2000	Relief Request RR-A5 is applicable.
		28 IN. Z/W AXIS INLET NOZZLE TO SHELL WELD MK 182 TO MK 169					
		RC-RPV-WR-13/14/72-X	UT	Yes		4/21/2000	Relief Request RR-A5 and RR-A19 are applicable.
		36 IN. X AXIS OUTLET NOZZLE TO SHELL WELD MK 168 TO MK 169					
		RC-RPV-WR-13/14/72-Z	UT	Yes		4/21/2000	Relief Request RR-A5 and RR-A19 are applicable.
		36 IN Z AXIS OUTLET NOZZLE TO SHELL WELD MK 168 TO MK 169					
		RC-RPV-WR-54/55-W	UT	Yes		4/21/2000	Relief Request RR-A5 is applicable.
		14 IN. W AXIS CORE FLOODING NOZZLE TO SHELL WELD MK 176 TO MK 169					

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**Inservice Inspection Abstract of ASME Class 1 and Class 2 Examinations
12th Refueling Outage**

Exam Category	Code Item	Component / Description	NDE Method	Examination Acceptable	CR Number	Exam Date	Comments
Code Class: 1							
B-D FULL PENETRATION WELDS OF NOZZLES IN VESSELS (INSPECTION PROGRAM B)							
	B03.090	REACTOR VESSEL-NOZZLE-TO-VESSEL WELDS RC-RPV-WR-54/55-Y 14 IN. Y AXIS CORE FLOODING NOZZLE TO SHELL WELD MK 176 TO MK 169	UT	Yes		4/21/2000	Relief Request RR-A5 is applicable.
	B03.100	REACTOR VESSEL-NOZZLE INSIDE RADIUS SECTION RC-RPV-WR-10/12-W/X-IR 28 IN. W/X AXIS INLET NOZZLE INSIDE RADIUS MK 182	UT	Yes		4/21/2000	Relief Request RR-A5 is applicable.
		RC-RPV-WR-10/12-X/Y-IR 28 IN. XY AXIS INLET NOZZLE INSIDE RADIUS MK 182	UT	Yes		4/21/2000	Relief Request RR-A5 is applicable.
		RC-RPV-WR-10/12-Y/Z-IR 28 IN. Y/Z AXIS INLET NOZZLE INSIDE RADIUS MK 182	UT	Yes		4/21/2000	Relief Request RR-A5 is applicable.
		RC-RPV-WR-10/12-Z/W-IR 28 IN. Z/W AXIS INLET NOZZLE INSIDE RADIUS MK 182	UT	Yes		4/21/2000	Relief Request RR-A5 is applicable.

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**Inservice Inspection Abstract of ASME Class 1 and Class 2 Examinations
12th Refueling Outage**

Exam Category	Code Item	Component / Description	NDE Method	Examination Acceptable	CR Number	Exam Date	Comments
Code Class: 1							
B-D	FULL PENETRATION WELDS OF NOZZLES IN VESSELS (INSPECTION PROGRAM B)						
	B03.100	REACTOR VESSEL-NOZZLE INSIDE RADIUS SECTION					
		RC-RPV-WR-13/14/72-X-IR	UT	Yes		4/21/2000	Relief Request RR-A5 is applicable.
		36 IN. X AXIS OUTLET NOZZLE INSIDE RADIUS MK 168					
		RC-RPV-WR-13/14/72-Z-IR	UT	Yes		4/21/2000	Relief Request RR-A5 is applicable.
		36 IN. Z AXIS OUTLET NOZZLE INSIDE RADIUS MK 168					
		RC-RPV-WR-54/55-W-IR	UT	Yes		4/21/2000	Relief Request RR-A5 and RR-A19 are applicable.
		14 IN. W AXIS CORE FLOODING NOZZLE INSIDE RADIUS MK 176					
		RC-RPV-WR-54/55-Y-IR	UT	Yes		4/21/2000	Relief Request RR-A5 and RR-A19 are applicable.
		14 IN. Y AXIS CORE FLOODING NOZZLE INSIDE RADIUS MK 176					
	B03.110	PRESSURIZER-NOZZLE-TO-VESSEL WELDS					
		RC-PZR-WP-15	UT	Yes		4/15/2000	Relief Request RR-A11 is applicable.
		10 IN. SURGE NOZZLE TO LOWER HEAD WELD MK 8 TO MK 6					

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Code Class: 1							
B-D	FULL PENETRATION WELDS OF NOZZLES IN VESSELS (INSPECTION PROGRAM B)						
	B03.110	PRESSURIZER-NOZZLE-TO-VESSEL WELDS RC-PZR-WP-33-Y/Z 3 IN. Y/Z AXIS RELIEF NOZZLE TO UPPER HEAD WELD MK 124 TO MK 5	UT	Yes		4/10/2000	Relief Request RR-A11 is applicable.
	B03.120	PRESSURIZER-NOZZLE INSIDE RADIUS SECTION RC-PZR-WP-15-IR 10 IN. SURGE NOZZLE INSIDE RADIUS MK 8	UT	Yes		4/10/2000	
		RC-PZR-WP-33-W/X-IR 3 IN. W/X AXIS RELIEF NOZZLE INSIDE RADIUS MK 124	UT	Yes		4/8/2000	
		RC-PZR-WP-33-Y/Z-IR 3 IN. Y/Z AXIS RELIEF NOZZLE INSIDE RADIUS MK 124	UT	Yes		4/8/2000	
		RC-PZR-WP-33-Z/W-IR 2.5 IN. Z/W AXIS RELIEF NOZZLE INSIDE RADIUS MK 31	UT	Yes		4/8/2000	

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Code Class: 1							
B-D	FULL PENETRATION WELDS OF NOZZLES IN VESSELS (INSPECTION PROGRAM B)						
	B03.120	PRESSURIZER-NOZZLE INSIDE RADIUS SECTION RC-PZR-WP-34-IR 4 IN. SPRAY NOZZLE INSIDE RADIUS MK 9	UT	Yes		4/8/2000	
	B03.130	STEAM GENERATORS (PRIMARY SIDE)-NOZZLE-TO-VESSEL WELDS RC-SG-1-2-WG-25 36 IN. INLET NOZZLE TO UPPER HEAD WELD MK 70 TO MK 8	UT	Yes		4/28/2000	Relief Request RR-A11 is applicable.
	B03.140	STEAM GENERATORS (PRIMARY SIDE)-NOZZLE INSIDE RADIUS SECTION RC-SG-1-2-WG-25-IR 36 IN. INLET NOZZLE INSIDE RADIUS MK 70	UT	Yes		4/28/2000	
		RC-SG-1-2-WG-50-Y/Z-IR 28 IN. Y/Z AXIS OUTLET NOZZLE INSIDE RADIUS MK 65	UT	Yes		4/10/2000	
		RC-SG-1-2-WG-50-Z/W-IR 28 IN. Z/W AXIS OUTLET NOZZLE INSIDE RADIUS MK 65	UT	Yes		4/10/2000	

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Code Class: 1							
B-E PRESSURE RETAINING PARTIAL PENETRATION WELDS IN VESSELS							
	B04.012	CONTROL ROD DRIVE NOZZLES					
		RC-RPV-CRD-WH-26-B52-46	VT-2	Yes		5/16/2000	
		CRD NOZZLE TO CLOSURE HEAD WELD (ASSY NO. B52-46)					
		MK 52 TO MK 24					
		 RC-RPV-CRD-WH-26-B52-48					
		CRD NOZZLE TO CLOSURE HEAD WELD (ASSY NO. B52-48)	VT-2	Yes		5/16/2000	
		MK 52 TO MK 24					
		 RC-RPV-CRD-WH-26-B53-51					
		CRD NOZZLE TO CLOSURE HEAD WELD (ASSY NO. B53-51)	VT-2	Yes		5/16/2000	
		MK 53 TO MK 24					
		 RC-RPV-CRD-WH-26-B53-52					
		CRD NOZZLE TO CLOSURE HEAD WELD (ASSY NO. B53-52)	VT-2	Yes		5/16/2000	
		MK 53 TO MK 24					
		 RC-RPV-CRD-WH-26-B53-55					
		CRD NOZZLE TO CLOSURE HEAD WELD (ASSY NO. B53-55)	VT-2	Yes		5/16/2000	
		MK 53 TO MK 24					

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Code Class: 1							
B-E	B04.012	PRESSURE RETAINING PARTIAL PENETRATION WELDS IN VESSELS CONTROL ROD DRIVE NOZZLES					
		RC-RPV-CRD-WH-26-B53-56 CRD NOZZLE TO CLOSURE HEAD WELD (ASSY NO. B53-56) MK 53 TO MK 24	VT-2	Yes		5/16/2000	
		RC-RPV-CRD-WH-26-B54-58 CRD NOZZLE TO CLOSURE HEAD WELD (ASSY NO. B54-58) MK 54 TO MK 24	VT-2	Yes		5/16/2000	
		RC-RPV-CRD-WH-26-B54-59 CRD NOZZLE TO CLOSURE HEAD WELD (ASSY NO. B54-59) MK 54 TO MK 24	VT-2	Yes		5/16/2000	
		RC-RPV-CRD-WH-26-B54-60 CRD NOZZLE TO CLOSURE HEAD WELD (ASSY NO. B54-60) MK 54 TO MK 24	VT-2	Yes		5/16/2000	
		RC-RPV-CRD-WH-26-B54-61 CRD NOZZLE TO CLOSURE HEAD WELD (ASSY NO. B54-61) MK 54 TO MK 24	VT-2	Yes		5/16/2000	

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Code Class: 1							
B-E	B04.012	PRESSURE RETAINING PARTIAL PENETRATION WELDS IN VESSELS CONTROL ROD DRIVE NOZZLES					
		RC-RPV-CRD-WH-26-B55-62 CRD NOZZLE TO CLOSURE HEAD WELD (ASSY NO. B55-62) MK 55 TO MK 24	VT-2	Yes		5/16/2000	
		RC-RPV-CRD-WH-26-B55-63 CRD NOZZLE TO CLOSURE HEAD WELD (ASSY NO. B55-63) MK 55 TO MK 24	VT-2	Yes		5/16/2000	
		RC-RPV-CRD-WH-26-B55-64 CRD NOZZLE TO CLOSURE HEAD WELD (ASSY NO. B55-64) MK 55 TO MK 24	VT-2	Yes		5/16/2000	
		RC-RPV-CRD-WH-26-B55-65 CRD NOZZLE TO CLOSURE HEAD WELD (ASSY NO. B55-65) MK 55 TO MK 24	VT-2	Yes		5/16/2000	
		RC-RPV-CRD-WH-26-B55-66 CRD NOZZLE TO CLOSURE HEAD WELD (ASSY NO. B55-66) MK 55 TO MK 24	VT-2	Yes		5/16/2000	

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Code Class: 1							
B-E	PRESSURE RETAINING PARTIAL PENETRATION WELDS IN VESSELS						
	B04.012	CONTROL ROD DRIVE NOZZLES					
		RC-RPV-CRD-WH-26-B55-67	VT-2	Yes		5/16/2000	
		CRD NOZZLE TO CLOSURE HEAD WELD (ASSY NO. B55-67)					
		MK 55 TO MK 24					
		 RC-RPV-CRD-WH-26-B55-68					
		CRD NOZZLE TO CLOSURE HEAD WELD (ASSY NO. B55-68)	VT-2	Yes		5/16/2000	
		MK 55 TO MK 24					
		 RC-RPV-CRD-WH-26-B55-69					
		CRD NOZZLE TO CLOSURE HEAD WELD (ASSY NO. B55-69)	VT-2	Yes		5/16/2000	
		MK 55 TO MK 24					
	B04.013	INSTRUMENTATION NOZZLES					
		RC-RPV-INST-WR-46/77-22	VT-2	Yes		5/16/2000	
		INSTRUMENTATION NOZZLE TO LOWER HEAD WELD (HOLE NO. 22)					
		 RC-RPV-INST-WR-46/77-28					
		INSTRUMENTATION NOZZLE TO LOWER HEAD WELD (HOLE NO. 28)	VT-2	Yes		5/16/2000	

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Code Class: 1							
B-E	PRESSURE RETAINING PARTIAL PENETRATION WELDS IN VESSELS						
	B04.013	INSTRUMENTATION NOZZLES					
		RC-RPV-INST-WR-46/77-30	VT-2	Yes		5/16/2000	
		INSTRUMENTATION NOZZLE TO LOWER HEAD WELD (HOLE NO. 30)					
		RC-RPV-INST-WR-46/77-33	VT-2	Yes		5/16/2000	
		INSTRUMENTATION NOZZLE TO LOWER HEAD WELD (HOLE NO. 33)					
		RC-RPV-INST-WR-46/77-35	VT-2	Yes		5/16/2000	
		INSTRUMENTATION NOZZLE TO LOWER HEAD WELD (HOLE NO. 35)					
		RC-RPV-INST-WR-46/77-37	VT-2	Yes		5/16/2000	
		INSTRUMENTATION NOZZLE TO LOWER HEAD WELD (HOLE NO. 37)					
		RC-RPV-INST-WR-46/77-38	VT-2	Yes		5/16/2000	
		INSTRUMENTATION NOZZLE TO LOWER HEAD WELD (HOLE NO. 38)					

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Code Class: 1							
B-E	PRESSURE RETAINING PARTIAL PENETRATION WELDS IN VESSELS						
	B04.013	INSTRUMENTATION NOZZLES					
		RC-RPV-INST-WR-46/77-41	VT-2	Yes		5/16/2000	
		INSTRUMENTATION NOZZLE TO LOWER HEAD WELD (HOLE NO. 41)					
		RC-RPV-INST-WR-46/77-45	VT-2	Yes		5/16/2000	
		INSTRUMENTATION NOZZLE TO LOWER HEAD WELD (HOLE NO. 45)					
		RC-RPV-INST-WR-46/77-46	VT-2	Yes		5/16/2000	
		INSTRUMENTATION NOZZLE TO LOWER HEAD WELD (HOLE NO. 46)					
		RC-RPV-INST-WR-46/77-48	VT-2	Yes		5/16/2000	
		INSTRUMENTATION NOZZLE TO LOWER HEAD WELD (HOLE NO. 48)					
		RC-RPV-INST-WR-46/77-49	VT-2	Yes		5/16/2000	
		INSTRUMENTATION NOZZLE TO LOWER HEAD WELD (HOLE NO. 49)					

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Code Class: 1							
B-E	PRESSURE RETAINING PARTIAL PENETRATION WELDS IN VESSELS						
	B04.013	INSTRUMENTATION NOZZLES					
		RC-RPV-INST-WR-46/77-51	VT-2	Yes		5/16/2000	
		INSTRUMENTATION NOZZLE TO LOWER HEAD WELD (HOLE NO. 51)					
B-F	PRESSURE RETAINING DISSIMILAR METAL WELDS						
	B05.010	REACTOR VESSEL-NOZZLE-TO-SAFE END BUTT WELDS \geq 4 INCHES NOMINAL PIPE SIZE					
		RC-RPV-WR-53-W	UT	Yes		4/21/2000	Relief Request RR-A1 and RR-A22 are applicable.
		14 IN. W AXIS SE TO CF NOZZLE WELD MK 176 TO MK 89					
		RC-RPV-WR-53-Y	UT	Yes		4/21/2000	Relief Request RR-A1 and RR-A22 are applicable.
		14 IN. Y AXIS SAFE END TO NOZZLE WELD MK 176 TO MK 89					
	B05.040	PRESSURIZER-NOZZLE-TO-SAFE END BUTT WELDS \geq 4 INCHES NOMINAL PIPE SIZE					
		RC-PZR-WP-23	UT	Yes		4/27/2000	
		10 IN. SURGE NOZZLE TO SAFE END WELD MK 8 TO MK 37	PT	Yes		4/17/2000	
	B05.050	PRESSURIZER-NOZZLE-TO-SAFE END BUTT WELDS $<$ 4 INCHES NOMINAL PIPE SIZE					
		RC-PZR-WP-91-Y/Z	PT	Yes		4/8/2000	
		3 IN. Y/Z AXIS RELIEF NOZZLE TO SAFE END WELD MK 124 TO MK 125					

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Code Class: 1							
B-G-1 PRESSURE RETAINING BOLTING GREATER THAN 2 INCHES IN DIAMETER							
	B06.010	REACTOR VESSEL-CLOSURE HEAD NUTS					
		RC-RPV-NUT-051	MT	Yes		4/24/2000	
		CLOSURE HEAD NUT (HOLE NO. 51) MK 26					
		RC-RPV-NUT-052	MT	Yes		4/24/2000	
		CLOSURE HEAD NUT (HOLE NO. 52) MK 26					
		RC-RPV-NUT-053	MT	Yes		4/24/2000	
		CLOSURE HEAD NUT (HOLE NO. 53) MK 26					
		RC-RPV-NUT-054	MT	Yes		4/24/2000	
		CLOSURE HEAD NUT (HOLE NO. 54) MK 26					
		RC-RPV-NUT-055	MT	Yes		4/24/2000	
		CLOSURE HEAD NUT (HOLE NO. 55) MK 26					

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Code Class: 1							
B-G-1 PRESSURE RETAINING BOLTING GREATER THAN 2 INCHES IN DIAMETER							
	B06.010	REACTOR VESSEL-CLOSURE HEAD NUTS					
		RC-RPV-NUT-056	MT	Yes		4/24/2000	
		CLOSURE HEAD NUT (HOLE NO. 56) MK 26					
		RC-RPV-NUT-057	MT	Yes		4/24/2000	
		CLOSURE HEAD NUT (HOLE NO. 57) MK 26					
		RC-RPV-NUT-058	MT	Yes		4/24/2000	
		CLOSURE HEAD NUT (HOLE NO. 58) MK 26					
		RC-RPV-NUT-059	MT	Yes		4/24/2000	
		CLOSURE HEAD NUT (HOLE NO. 59) MK 26					
		RC-RPV-NUT-060	MT	Yes		4/24/2000	
		CLOSURE HEAD NUT (HOLE NO. 60) MK 26					

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Code Class: 1							
B-G-1	PRESSURE RETAINING BOLTING GREATER THAN 2 INCHES IN DIAMETER						
	B06.030	REACTOR VESSEL-CLOSURE STUDS, WHEN REMOVED					
		RC-RPV-STUD-051	UT	Yes		4/22/2000	
		CLOSURE HEAD STUD (HOLE NO. 51) MK 25	MT	Yes		4/24/2000	
		RC-RPV-STUD-052					
		CLOSURE HEAD STUD (HOLE NO. 52) MK 25	UT	Yes		4/22/2000	
			MT	Yes		4/24/2000	
		RC-RPV-STUD-053					
		CLOSURE HEAD STUD (HOLE NO. 53) MK 25	UT	Yes		4/22/2000	
			MT	Yes		4/24/2000	
		RC-RPV-STUD-054					
		CLOSURE HEAD STUD (HOLE NO. 54) MK 25	UT	Yes		4/22/2000	
			MT	Yes		4/24/2000	
		RC-RPV-STUD-055					
		CLOSURE HEAD STUD (HOLE NO. 55) MK 25	UT	Yes		4/22/2000	
			MT	Yes		4/24/2000	

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Code Class: 1							
B-G-1	PRESSURE RETAINING BOLTING GREATER THAN 2 INCHES IN DIAMETER						
	B06.030	REACTOR VESSEL-CLOSURE STUDS, WHEN REMOVED					
		RC-RPV-STUD-056	UT	Yes		4/22/2000	
		CLOSURE HEAD STUD (HOLE NO. 56) MK 25	MT	Yes		4/24/2000	
		RC-RPV-STUD-057					
		CLOSURE HEAD STUD (HOLE NO. 57) MK 25	UT	Yes		4/22/2000	
			MT	Yes		4/24/2000	
		RC-RPV-STUD-058					
		CLOSURE HEAD STUD (HOLE NO. 58) MK 25	UT	Yes		4/22/2000	
			MT	Yes		4/24/2000	
		RC-RPV-STUD-059					
		CLOSURE HEAD STUD (HOLE NO. 59) MK 25	UT	Yes		4/22/2000	
			MT	Yes		4/24/2000	
		RC-RPV-STUD-060					
		CLOSURE HEAD STUD (HOLE NO. 60) MK 25	UT	Yes		4/22/2000	
			MT	Yes		4/24/2000	

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Code Class: 1							
B-G-1	PRESSURE RETAINING BOLTING GREATER THAN 2 INCHES IN DIAMETER						
	B06.050	REACTOR VESSEL-CLOSURE WASHERS, BUSHINGS					
		RC-RPV-WASHERS-051	VT-1	Yes		4/18/2000	
		CLOSURE HEAD WASHERS (HOLE NO. 51) MK 14/MK 27					
		RC-RPV-WASHERS-052	VT-1	Yes		4/18/2000	
		CLOSURE HEAD WASHERS (HOLE NO. 52) MK 14/MK 27					
		RC-RPV-WASHERS-053	VT-1	Yes		4/18/2000	
		CLOSURE HEAD WASHERS (HOLE NO. 53) MK 14/MK 27					
		RC-RPV-WASHERS-054	VT-1	Yes		4/18/2000	
		CLOSURE HEAD WASHERS (HOLE NO. 54) MK 14/MK 27					
		RC-RPV-WASHERS-055	VT-1	Yes		4/18/2000	
		CLOSURE HEAD WASHERS (HOLE NO. 55) MK 14/MK 27					

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Code Class: 1							
B-G-1	PRESSURE RETAINING BOLTING GREATER THAN 2 INCHES IN DIAMETER						
	B06.050	REACTOR VESSEL-CLOSURE WASHERS, BUSHINGS					
		RC-RPV-WASHERS-056	VT-1	Yes		4/18/2000	
		CLOSURE HEAD WASHERS (HOLE NO. 56) MK 14/MK 27					
		RC-RPV-WASHERS-057	VT-1	Yes		4/18/2000	
		CLOSURE HEAD WASHERS (HOLE NO. 57) MK 14/MK 27					
		RC-RPV-WASHERS-058	VT-1	Yes		4/18/2000	
		CLOSURE HEAD WASHERS (HOLE NO. 58) MK 14/MK 27					
		RC-RPV-WASHERS-059	VT-1	Yes		4/18/2000	
		CLOSURE HEAD WASHERS (HOLE NO. 59) MK 14/MK 27					
		RC-RPV-WASHERS-060	VT-1	Yes		4/18/2000	
		CLOSURE HEAD WASHERS (HOLE NO. 60) MK 14/MK 27					

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Code Class: 1							
B-G-1	PRESSURE RETAINING BOLTING GREATER THAN 2 INCHES IN DIAMETER						
	B06.060	PRESSURIZER-BOLTS AND STUDS					
		RC-PZR-STUD-10	UT	Yes		4/10/2000	
		2.75 IN. 8UN-2A MANWAY STUD MK 67					
		RC-PZR-STUD-11	UT	Yes		4/10/2000	
		2.75 IN. 8UN-2A MANWAY STUD MK 67					
		RC-PZR-STUD-12	UT	Yes		4/10/2000	
		2.75 IN. 8UN-2A MANWAY STUD MK 67					
		RC-PZR-STUD-9	UT	Yes		4/8/2000	
		2.75 IN. 8UN-2A MANWAY STUD MK 67					
	B06.080	PRESSURIZER-NUTS, BUSHINGS, AND WASHERS					
		RC-PZR-NUT-10	VT-1	Yes		4/8/2000	
		2.75 IN. 8UN-2B MANWAY NUT MK 68					

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Davis-Besse Nuclear Power Station Unit #1, 5501 N. St. Rt. 2, Oak Harbor, Ohio 43449

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Exam Category	Code Item	Component / Description	NDE Method	Examination Acceptable	CR Number	Exam Date	Comments
Code Class: 1							
B-G-1	PRESSURE RETAINING BOLTING GREATER THAN 2 INCHES IN DIAMETER						
	B06.080	PRESSURIZER-NUTS, BUSHINGS, AND WASHERS					
		RC-PZR-NUT-11	VT-1	Yes		4/8/2000	
		2.75 IN. 8UN-2B MANWAY NUT MK 68					
		RC-PZR-NUT-12	VT-1	Yes		4/8/2000	
		2.75 IN. 8UN-2B MANWAY NUT MK 68					
		RC-PZR-NUT-9	VT-1	Yes		4/8/2000	
		2.75 IN. 8UN-2B MANWAY NUT MK 68					
B-G-2	PRESSURE RETAINING BOLTING, 2 INCHES AND LESS IN DIAMETER						
	B07.020	PRESSURIZER-BOLTS, STUDS, AND NUTS					
		RC-PZR-HTR-BNDL-3-BLTG	VT-1	Yes		4/10/2000	
		16 - 2 IN. HEATER BELT STUDS/NUTS MK 75, MK 76					
	B07.050	PIPING-BOLTS, STUDS, AND NUTS					
		RC-HEAD VENT-FLANGE BOLTING-1	VT-1	Yes		4/7/2000	
		8 - 1.25 IN. 12 UNF-2A BOLTS (FLANGE CONNECTION NO. 1)					

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Code Class: 1							
B-G-2	PRESSURE RETAINING BOLTING, 2 INCHES AND LESS IN DIAMETER						
	B07.050	PIPING-BOLTS, STUDS, AND NUTS RC-HEAD VENT-FLANGE BOLTING-4 8 - 1.125 IN. STUDS AND NUTS (FLANGE CONNECTION NO. 4)	VT-1	Yes		4/7/2000	
		RC-RC13B-BLTG-2 8 - 1.50 IN. STUDS AND NUTS (FLANGE CONNECTION NO. 2)	VT-1	Yes		4/5/2000	
	B07.070	VALVES-BOLTS, STUDS, AND NUTS DH-DH23-BLTG 16 - 1.375 IN. BONNET TO BODY BLTG	VT-1	Yes		4/17/2000	
B-J	PRESSURE RETAINING WELDS IN PIPING						
	B09.011	CIRCUMFERENTIAL PIPE WELDS \geq 4 IN. NOMINAL PIPE SIZE CF-33B-CCA-6-3-FW30 14 IN. PIPE TO SAFE END WELD	UT	Yes		4/21/2000	AUGMENTED EXAMINATION
		CF-33B-CCA-6-4-FW17 14 IN. PIPE TO SAFE END WELD	UT	Yes		4/21/2000	AUGMENTED EXAMINATION

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Code Class: 1							
B-J PRESSURE RETAINING WELDS IN PIPING							
	B09.011	CIRCUMFERENTIAL PIPE WELDS >= 4 IN. NOMINAL PIPE SIZE					
		DH-33A-CCA-4-4-SWB	UT	Yes		4/7/2000	
		8 IN. ELBOW TO PIPE WELD	PT	Yes		4/6/2000	
		 DH-33A-CCA-4-F6A-FW6					
		12 IN. PIPE TO VALVE DH11 WELD	UT	Yes		4/28/2000	Relief Request RR-A16 is applicable. EXAMINED PER RELIEF REQUEST RR-A16.
			PT	Yes		4/24/2000	
		 RC-MK-A-32-1-FW111B					
		28 IN. REACTOR VESSEL OUTLET NOZZLE TO PIPE WELD	UT	Yes		4/21/2000	Relief Request RR-A1 is applicable.
		MK 168 TO MK 119 (WJ-1-2)					
		 RC-MK-A-33-1-FW111A					
		36 IN. REACTOR VESSEL OUTLET NOZZLE TO PIPE WELD	UT	Yes		4/21/2000	Relief Request RR-A1 is applicable.
		MK 168 TO MK 119 (WJ-1-1)					
		 RC-MK-A-33-1-SW130A					
		36 IN. ELBOW TO PIPE WELD MK 31 TO MK 23	UT	Yes		4/21/2000	
			MT	Yes		4/15/2000	

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Code Class: 1							
B-J PRESSURE RETAINING WELDS IN PIPING							
	B09.011	CIRCUMFERENTIAL PIPE WELDS \geq 4 IN. NOMINAL PIPE SIZE					
		RC-MK-A-81-SW5	UT	Yes		4/27/2000	
		10 IN. ELBOW TO PIPE WELD MK 80 TO MK 84	PT	Yes		4/11/2000	
		 RC-MK-B-44-1-FW56B					
		28 IN. PIPE TO REACTOR VESSEL INLET NOZZLE WELD MK 120 TO MK 182 (WJ-4-4)	UT	Yes		4/21/2000	Relief Request RR-A1 is applicable.
		 RC-MK-B-56-1-FW113A					
		28 IN. PIPE TO REACTOR VESSEL INLET NOZZLE WELD MK 120 TO MK 182 (WJ-4-1)	UT	Yes		4/21/2000	Relief Request RR-A1 is applicable.
		 RC-MK-B-59-1-FW113B					
		28 IN. PIPE TO REACTOR VESSEL INLET NOZZLE WELD MK 120 TO MK 182 (WJ-4-3)	UT	Yes		4/21/2000	Relief Request RR-A1 is applicable.
		 RC-MK-B-61-1-FW56A					
		28 IN. PIPE TO REACTOR VESSEL INLET NOZZLE WELD MK 120 TO MK 182 (WJ-4-2)	UT	Yes		4/21/2000	Relief Request RR-A1 is applicable.

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Code Class: 1							
B-J PRESSURE RETAINING WELDS IN PIPING							
	B09.021	CIRCUMFERENTIAL PIPE WELDS < 4 IN. NOMINAL PIPE SIZE					
		HP-33C-CCA-2-45-FW34A	UT	Yes		4/20/2000	UT IS AUGMENTED EXAMINATION.
		2.5 IN. ELBOW TO ELBOW WELD	PT	Yes		4/13/2000	
		RC-30-CCA-8-3-FW8B					
		2.5 IN. FLANGE TO ELBOW WELD	PT	Yes		4/20/2000	
		RC-40-CCA-18-5-SWA					
		2.5 IN. ELBOW TO ELBOW WELD	PT	Yes		4/17/2000	
		RC-MK-A-112-SW6					
		2.5 IN. ELBOW TO PIPE WELD MK 98 TO MK 112	PT	Yes		4/17/2000	
		RC-MK-A-135-SW25					
		2.5 IN. ELBOW TO PIPE WELD MK 98 TO MK 135	PT	Yes		4/12/2000	

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Code Class: 1							
B-J PRESSURE RETAINING WELDS IN PIPING							
	B09.021	CIRCUMFERENTIAL PIPE WELDS < 4 IN. NOMINAL PIPE SIZE RC-MK-A-139-SW16 2.5 IN. ELBOW TO PIPE WELD MK 98 TO MK 139	PT	Yes		4/7/2000	
		RC-SK84-02-02-FW1B 3 IN. PIPE TO PIPE WELD	PT	Yes		4/6/2000	
		RC-SK84-02-09-FW6 2.5 IN. ELBOW TO FLANGE WELD	PT	Yes		4/7/2000	
	B09.031	BRANCH CONNECTION WELDS >= 4 IN. NOMINAL PIPE SIZE RC-MK-B-56-1-IN4 28 IN. PIPE TO 2.5 IN. BRANCH CONNECTION WELD MK 56 TO MK 46 (WJ-72)	UT MT	Yes Yes		4/22/2000 4/22/2000	Relief Request RR-A10 is applicable.
	B09.032	BRANCH CONNECTION WELDS < 4 IN. NOMINAL PIPE SIZE RC-MK-B-56-1-PS4 28 IN. PIPE TO 2.5 IN. BRANCH CONNECTION WELD MK 49 TO MK 51 (WJ-50)	UT PT	Yes Yes		4/28/2000 4/22/2000	UT IS AN AUGMENTED EXAMINATION

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Code Class: 1							
B-J	PRESSURE RETAINING WELDS IN PIPING						
	B09.040	SOCKET WELDS					
		RC-FSK-M-CCA-7-1-FW2	PT	Yes		4/17/2000	
		1.5 IN. VALVE RC51 TO PIPE WELD					
		 RC-FSK-M-CCA-7-1-FW34					
		1.5 IN. VALVE DH159 TO PIPE WELD	PT	Yes		4/13/2000	
B-L-1	PRESSURE RETAINING WELDS IN PUMP CASING						
	B12.010	PUMPS-PUMP CASING WELDS					
		RC-RCP-1-1-1-WELD	VT-2	Yes		5/16/2000	Relief Request RR-A8 is applicable.
		PUMP CASING WELD					
		 RC-RCP-1-1-2-WELD					
		PUMP CASING WELD	VT-1 VT-2	Yes Yes		4/13/2000 5/16/2000	Relief Request RR-A8 is applicable.
		 RC-RCP-1-2-1-WELD					
		PUMP CASING WELD	VT-2	Yes		5/16/2000	Relief Request RR-A8 is applicable.

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Code Class: 1							
B-L-1	PRESSURE RETAINING WELDS IN PUMP CASING						
	B12.010	PUMPS-PUMP CASING WELDS					
		RC-RCP-1-2-2-WELD	VT-2	Yes		5/16/2000	Relief Request RR-A8 is applicable.
		PUMP CASING WELD					
B-M-2	VALVE BODIES						
	B12.050	VALVES-VALVE BODIES EXCEEDING 4 INCHES NOMINAL PIPE SIZE					
		DH-DH23-SURFACES	VT-3	Yes		4/17/2000	
		8 IN. 1500# BB GATE VALVE INTERNAL SURFACES					
B-N-1	INTERIOR OF REACTOR VESSEL						
	B13.010	REACTOR VESSEL-VESSEL INTERIOR					
		RC-RPV-INTERIOR	VT-3	Yes		4/22/2000	
		VESSEL INTERIOR					
B-N-2	INTEGRALLY WELDED CORE SUPPORT STRUCTURES AND INTERIOR ATTACHMENTS TO REACTOR VESSEL						
	B13.060	REACTOR VESSEL (PWR)-INTERIOR ATTACHMENTS BEYOND BELTLINE REGION					
		RC-RPV-WR-3/49-1	VT-3	Yes		4/22/2000	
		INTERNAL GUIDE LUG TO SHELL ATTACHMENT WELD NO. 1					
		MK 16 TO MK 171					
		RC-RPV-WR-3/49-10					
		INTERNAL GUIDE LUG TO SHELL ATTACHMENT WELD NO.	VT-3	Yes		4/22/2000	
		10MK 16 TO MK 171					

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Code Class: 1							
B-N-2	INTEGRALLY WELDED CORE SUPPORT STRUCTURES AND INTERIOR ATTACHMENTS TO REACTOR VESSEL						
	B13.060	REACTOR VESSEL (PWR)-INTERIOR ATTACHMENTS BEYOND BELTLINE REGION					
		RC-RPV-WR-3/49-11	VT-3	Yes		4/22/2000	
		INTERNAL GUIDE LUG TO SHELL ATTACHMENT WELD NO. 11MK 16 TO MK 171					
		RC-RPV-WR-3/49-12	VT-3	Yes		4/22/2000	
		INTERNAL GUIDE LUG TO SHELL ATTACHMENT WELD NO. 12MK 16 TO MK 171					
		RC-RPV-WR-3/49-2	VT-3	Yes		4/22/2000	
		INTERNAL GUIDE LUG TO SHELL ATTACHMENT WELD NO. 2 MK 16 TO MK 171					
		RC-RPV-WR-3/49-3	VT-3	Yes		4/22/2000	
		INTERNAL GUIDE LUG TO SHELL ATTACHMENT WELD NO. 3 MK 16 TO MK 171					
		RC-RPV-WR-3/49-4	VT-3	Yes		4/22/2000	
		INTERNAL GUIDE LUG TO SHELL ATTACHMENT WELD NO. 4 MK 16 TO MK 171					

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Code Class: 1							
B-N-2	INTEGRALLY WELDED CORE SUPPORT STRUCTURES AND INTERIOR ATTACHMENTS TO REACTOR VESSEL						
	B13.060	REACTOR VESSEL (PWR)-INTERIOR ATTACHMENTS BEYOND BELTLINE REGION					
		RC-RPV-WR-3/49-5	VT-3	Yes		4/22/2000	
		INTERNAL GUIDE LUG TO SHELL ATTACHMENT WELD NO. 5 MK 16 TO MK 171					
		RC-RPV-WR-3/49-6	VT-3	Yes		4/22/2000	
		INTERNAL GUIDE LUG TO SHELL ATTACHMENT WELD NO. 6 MK 16 TO MK 171					
		RC-RPV-WR-3/49-7	VT-3	Yes		4/22/2000	
		INTERNAL GUIDE LUG TO SHELL ATTACHMENT WELD NO. 7 MK 16 TO MK 171					
		RC-RPV-WR-3/49-8	VT-3	Yes		4/22/2000	
		INTERNAL GUIDE LUG TO SHELL ATTACHMENT WELD NO. 8 MK 16 TO MK 171					
		RC-RPV-WR-3/49-9	VT-3	Yes		4/22/2000	
		INTERNAL GUIDE LUG TO SHELL ATTACHMENT WELD NO. 9 MK 16 TO MK 171					

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Code Class: 1							
B-N-3	REMOVABLE CORE SUPPORT STRUCTURES						
	B13.070	REACTOR VESSEL (PWR)-CORE SUPPORT STRUCTURE					
		RC-RPV-CORE SUPPORT ASSEMBLY	VT-3	Yes		4/22/2000	
		VESSEL CORE SUPPORT STRUCTURE					
Code Class: 2							
C-A	PRESSURE RETAINING WELDS IN PRESSURE VESSELS						
	C01.010	SHELL CIRCUMFERENTIAL WELDS					
		DH-COOLER-1-1-WELD G	UT	Yes		4/3/2000	Relief Request RR-B1 is applicable.
		SHELL TO FLANGE WELD					
		SP-SG-1-2-WG-8-3					
		NOZZLE BELT TO SHELL WELD MK 3 TO MK 2	UT	Yes		5/1/2000	
		SP-SG-1-2-WG-8-4					
		SHELL TO SHELL WELD MK 5 TO MK 6	UT	Yes		5/1/2000	
C-B	PRESSURE RETAINING NOZZLE WELDS IN VESSELS						
	C02.021	NOZZLE-TO-SHELL (OR HEAD) WELD > 1/2 IN. NOMINAL THICKNESS WITHOUT REINFORCING PLATE					
		SP-SG-1-2-WG-23-W/X	UT	Yes		4/28/2000	
		24 IN. W/X AXIS STEAM OUTLET NOZZLE TO SHELL WELD	MT	Yes		4/27/2000	
		MK 14 TO MK 3					

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Code Class: 2							
C-B	PRESSURE RETAINING NOZZLE WELDS IN VESSELS						
	C02.021	NOZZLE-TO-SHELL (OR HEAD) WELD > 1/2 IN. NOMINAL THICKNESS WITHOUT REINFORCING PLATE					
		SP-SG-1-2-WG-23-XY	UT	Yes		4/28/2000	
		24 IN. X/Y AXIS STEAM OUTLET NOZZLE TO SHELL WELD	MT	Yes		4/27/2000	
		MK 14 TO MK 3					
	C02.022	NOZZLE INSIDE RADIUS SECTION > 1/2 IN. NOMINAL THICKNESS WITHOUT REINFORCING PLATE					
		SP-SG-1-2-WG-23-W/X-IR	UT	Yes		4/28/2000	
		24 IN. W/X AXIS STEAM OUTLET NOZZLE INSIDE RADIUS MK 14					
		SP-SG-1-2-WG-23-XY-IR					
		24 IN. X/Y AXIS STEAM OUTLET NOZZLE INSIDE RADIUS MK 14	UT	Yes		4/28/2000	
C-C	INTEGRAL ATTACHMENTS FOR VESSELS, PIPING, PUMPS, AND VALVES						
	C03.020	PIPING-INTEGRALLY WELDED ATTACHMENTS					
		FW-7-EBB-3-9-SR24A-AW	MT	Yes		4/24/2000	Relief Request RR-A14 is applicable.
		14 IN. PIPE SUPPORT ATTACHMENT WELD					
		FW-7-EBB-3-SG2-SP8-AW					
		14 IN. PIPE SUPPORT ATTACHMENT WELD TO MK 121	MT	Yes		4/27/2000	Relief Request RR-A14 is applicable.
		MK 130					

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Code Class: 2							
C-C INTEGRAL ATTACHMENTS FOR VESSELS, PIPING, PUMPS, AND VALVES							
	C03.020	PIPING-INTEGRALLY WELDED ATTACHMENTS					
		HP-33C-CCB-2-H18-AW	PT	Yes		4/5/2000	Relief Request RR-A14 is applicable.
		4 IN. PIPE SUPPORT ATTACHMENT WELD					
		MS-3A-EBB-1-28-SR19-AW	MT	Yes		4/6/2000	Relief Request RR-A14 is applicable.
		36 IN. PIPE SUPPORT ATTACHMENT WELD					
	C03.20A	THIN WALL PIPING INTEGRALLY WELDED ATTACHMENTS					
		CS-34-GCB-5-H19-AW	PT	Yes		4/1/2000	Relief Request RR-A14 is applicable.
		8 IN. PIPE SUPPORT ATTACHMENT WELD					
		DH-33A-GCB-8-H13-AW	PT	Yes		4/12/2000	Relief Request RR-A14 is applicable.
		12 IN. PIPE SUPPORT ATTACHMENT WELD					
		DH-33B-GCB-1-H25-AW	PT	Yes		4/1/2000	Relief Request RR-A14 is applicable.
		10 IN. PIPE SUPPORT ATTACHMENT WELD					

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Code Class: 2							
C-C	INTEGRAL ATTACHMENTS FOR VESSELS, PIPING, PUMPS, AND VALVES						
	C03.20A	THIN WALL PIPING INTEGRALLY WELDED ATTACHMENTS					
		HP-33A-GCB-4-H6-AW	PT	Yes		4/5/2000	Relief Request RR-A14 is applicable.
		6 IN. PIPE SUPPORT ATTACHMENT WELD					
C-D	PRESSURE RETAINING BOLTING GREATER THAN 2 INCHES IN DIAMETER						
	C04.030	PUMPS-BOLTS AND STUDS					
		HP-PUMP-1-2-STUD-001	UT	Yes		3/8/2000	
		2.25 IN. DIA. PUMP COVER TO CASE STUD AT HOLE NO. 1					
		HP-PUMP-1-2-STUD-002	UT	Yes		3/8/2000	
		2.25 IN. DIA. PUMP COVER TO CASE STUD AT HOLE NO. 2					
		HP-PUMP-1-2-STUD-003	UT	Yes		3/8/2000	
		2.25 IN. DIA. PUMP COVER TO CASE STUD AT HOLE NO. 3					
		HP-PUMP-1-2-STUD-004	UT	Yes		3/8/2000	
		2.25 IN. DIA. PUMP COVER TO CASE STUD AT HOLE NO. 4					

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Code Class: 2							
C-D PRESSURE RETAINING BOLTING GREATER THAN 2 INCHES IN DIAMETER							
	C04.030	PUMPS-BOLTS AND STUDS					
		HP-PUMP-1-2-STUD-005	UT	Yes		3/8/2000	
		2.25 IN. DIA. PUMP COVER TO CASE STUD AT HOLE NO. 5					
		HP-PUMP-1-2-STUD-006	UT	Yes		3/8/2000	
		2.25 IN. DIA. PUMP COVER TO CASE STUD AT HOLE NO. 6					
		HP-PUMP-1-2-STUD-007	UT	Yes		3/8/2000	
		2.25 IN. DIA. PUMP COVER TO CASE STUD AT HOLE NO. 7					
		HP-PUMP-1-2-STUD-008	UT	Yes		3/8/2000	
		2.25 IN. DIA. PUMP COVER TO CASE STUD AT HOLE NO. 8					
		HP-PUMP-1-2-STUD-009	UT	Yes		3/8/2000	
		2.25 IN. DIA. PUMP COVER TO CASE STUD AT HOLE NO. 9					

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Code Class: 2							
C-D							
	C04.030	PRESSURE RETAINING BOLTING GREATER THAN 2 INCHES IN DIAMETER					
		PUMPS-BOLTS AND STUDS					
		HP-PUMP-1-2-STUD-010	UT	Yes		3/8/2000	
		2.25 IN. DIA. PUMP COVER TO CASE STUD AT HOLE NO. 10					
		HP-PUMP-1-2-STUD-011	UT	Yes		3/8/2000	
		2.25 IN. DIA. PUMP COVER TO CASE STUD AT HOLE NO. 11					
		HP-PUMP-1-2-STUD-012	UT	Yes		3/8/2000	
		2.25 IN. DIA. PUMP COVER TO CASE STUD AT HOLE NO. 12					
		HP-PUMP-1-2-STUD-013	UT	Yes		3/8/2000	
		2.25 IN. DIA. PUMP COVER TO CASE STUD AT HOLE NO. 13					
		HP-PUMP-1-2-STUD-014	UT	Yes		3/8/2000	
		2.25 IN. DIA. PUMP COVER TO CASE STUD AT HOLE NO. 14					

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Exam Category	Code Item	Component / Description	NDE Method	Examination Acceptable	CR Number	Exam Date	Comments
Code Class: 2							
C-D							
	C04.030	PRESSURE RETAINING BOLTING GREATER THAN 2 INCHES IN DIAMETER PUMPS-BOLTS AND STUDS					
		HP-PUMP-1-2-STUD-015 2.25 IN. DIA. PUMP COVER TO CASE STUD AT HOLE NO. 15	UT	Yes		3/8/2000	
		HP-PUMP-1-2-STUD-016 2.25 IN. DIA. PUMP COVER TO CASE STUD AT HOLE NO. 16	UT	Yes		3/8/2000	
		HP-PUMP-1-2-STUD-017 2.25 IN. DIA. PUMP COVER TO CASE STUD AT HOLE NO. 17	UT	Yes		3/8/2000	
		HP-PUMP-1-2-STUD-018 2.25 IN. DIA. PUMP COVER TO CASE STUD AT HOLE NO. 18	UT	Yes		3/8/2000	
		HP-PUMP-1-2-STUD-019 2.25 IN. DIA. PUMP COVER TO CASE STUD AT HOLE NO. 19	UT	Yes		3/8/2000	

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Exam Category	Code Item	Component / Description	NDE Method	Examination Acceptable	CR Number	Exam Date	Comments
Code Class: 2							
C-D	PRESSURE RETAINING BOLTING GREATER THAN 2 INCHES IN DIAMETER						
	C04.030	PUMPS-BOLTS AND STUDS					
		HP-PUMP-1-2-STUD-020	UT	Yes		3/8/2000	
		2.25 IN. DIA. PUMP COVER TO CASE STUD AT HOLE NO. 20					
C-F-1	PRESSURE RETAINING WELDS IN AUSTENITIC STAINLESS STEEL OR HIGH ALLOY PIPING						
	C05.011	CIRCUMFERENTIAL PIPE WELDS \geq 3/8 IN. NOMINAL WALL THICKNESS					
		DH-33B-CCB-6-2-FW2	UT	Yes		4/27/2000	Relief Request RR-B4 is applicable.
		10 IN. ELBOW TO ELBOW WELD	PT	Yes		4/22/2000	
	C05.021	CIRCUMFERENTIAL PIPE WELDS $>$ 1/5 IN. NOMINAL WALL THICKNESS					
		HP-33C-CCB-2-12-SWC	UT	Yes		4/20/2000	Relief Request RR-B4 is applicable.
		4 IN. PIPE TO ELBOW WELD	PT	Yes		3/31/2000	
		HP-33C-CCB-2-21-SWC					
		2.5 IN. ELBOW TO PIPE WELD	UT PT	Yes Yes		4/23/2000 4/22/2000	Relief Request RR-B4 is applicable.
		HP-33C-CCB-2-25-SWA					
		2.5 IN. PIPE TO ELBOW WELD	UT PT	Yes Yes		4/21/2000 4/11/2000	Relief Request RR-B4 is applicable.

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Exam Category	Code Item	Component / Description	NDE Method	Examination Acceptable	CR Number	Exam Date	Comments
Code Class: 2							
C-F-1	PRESSURE RETAINING WELDS IN AUSTENITIC STAINLESS STEEL OR HIGH ALLOY PIPING						
	C05.021	CIRCUMFERENTIAL PIPE WELDS > 1/5 IN. NOMINAL WALL THICKNESS					
		HP-4-CCB-2-FW161	UT	Yes		4/22/2000	Relief Request RR-B4 is applicable. PRESERVICE EXAMINATION
		2.5 IN. PIPE TO ELBOW WELD	PT	Yes		4/20/2000	
	C05.041	CIRCUMFERENTIAL PIPE BRANCH CONNECTION WELDS > 2 IN. NOMINAL BRANCH PIPE SIZE					
		HP-33C-CCB-2-14-SWB	PT	Yes		4/3/2000	
		4 IN. PIPE TO 4 X 2 SOCKOLET WELD					
C-F-2	PRESSURE RETAINING WELDS IN CARBON OR LOW ALLOY STEEL PIPING						
	C05.051	CIRCUMFERENTIAL PIPE WELDS \geq 3/8 IN. NOMINAL WALL THICKNESS FOR PIPING >NPS 4					
		AF-6C-EBB-4-7-FW52	UT	Yes		4/6/2000	
		6 IN. ELBOW TO PIPE WELD	MT	Yes		4/3/2000	
		FW-7-EBB-3-4-SWH					
		14 IN. PIPE TO ELBOW WELD	UT MT	Yes Yes		4/27/2000 4/22/2000	
		MS-3A-EBB-1-14-SWB					
		26 IN. PIPE TO ELBOW WELD	UT MT	Yes Yes		4/27/2000 4/24/2000	

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Code Class: 2							
C-F-1 PRESSURE RETAINING WELDS IN AUSTENITIC STAINLESS STEEL OR HIGH ALLOY PIPING							
	C05.11A	CIRCUMFERENTIAL PIPE WELD < 3/8 IN. & > 1/5 IN. NOMINAL WALL THICKNESS FOR PIPING > NPS 4					
		CS-34-GCB-5-18-SWA	UT	Yes		4/3/2000	Relief Request RR-B4 is
		8 IN. TEE TO ELBOW WELD	PT	Yes		4/5/2000	applicable.
		CS-34-GCB-5-8-SWC					
		8 IN. ELBOW TO PIPE WELD	UT	Yes		4/10/2000	Relief Request RR-B4 is
			PT	Yes		3/31/2000	applicable.
		DH-33A-GCB-8-3-SWB					
		18 IN. ELBOW TO 18 X 12 CONCENTRIC REDUCER WELD	UT	Yes		3/31/2000	Relief Request RR-B4 is
			PT	Yes		3/29/2000	applicable.
		DH-33A-GCB-8-6A-SWF					
		12 IN. PIPE TO FLANGE WELD	UT	Yes		4/22/2000	Relief Request RR-B4 is
			PT	Yes		4/5/2000	applicable.
		DH-33B-GCB-1-3-SWB					
		10 IN. ELBOW TO ELBOW WELD	UT	Yes		4/5/2000	Relief Request RR-B4 is
			PT	Yes		3/29/2000	applicable.

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Code Class: 2							
C-F-1 PRESSURE RETAINING WELDS IN AUSTENITIC STAINLESS STEEL OR HIGH ALLOY PIPING							
	C05.11A	CIRCUMFERENTIAL PIPE WELD < 3/8 IN. & > 1/5 IN. NOMINAL WALL THICKNESS FOR PIPING > NPS 4					
		DH-33B-GCB-10-1-SWA	UT	Yes		4/5/2000	Relief Request RR-B4 is applicable.
		10 IN. FLANGE TO ELBOW WELD	PT	Yes		3/30/2000	
		DH-33B-GCB-10-19-SWD					
		10 IN. ELBOW TO PIPE WELD	UT	Yes		4/13/2000	Relief Request RR-B4 is applicable.
			PT	Yes		3/31/2000	
PRESSURE RETAINING WELDS IN AUSTENITIC STAINLESS STEEL							
	C05.11B	CIRCUMFERENTIAL PIPE WELD <= 1/5 IN. NOMINAL WALL THICKNESS FOR PIPING > NPS 4					
		HP-33A-GCB-4-4-FW32	PT	Yes		3/29/2000	Relief Request RR-B4/ is applicable.
		6 IN. ELBOW TO HP PUMP INLET NOZZLE WELD					
PRESSURE RETAINING WELDS IN AUSTENITIC STAINLESS STEEL OR HIGH ALLOY PIPING							
	C05.21A	CIRCUMFERENTIAL PIPE WELD < 1/5 IN. NOMINAL WALL THICKNESS FOR PIPING >= NPS 2 AND <= NPS 4					
		HP-35-HCC-21-5-SWB	PT	Yes		4/25/2000	Relief Request RR-B4 is applicable.
		PIPE TO ELBOW WELD					
		HP-4-GCB-11-FW76					
		4 IN. PIPE TO FLANGE WELD	PT	Yes		4/8/2000	Relief Request RR-B4 is applicable.

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Code Class: 2							
C-F-2	PRESSURE RETAINING WELDS IN CARBON OR LOW ALLOY STEEL PIPING						
	C05.51A	CIRCUMFERENTIAL PIPE WELDS < 3/8 IN. NOMINAL WALL THICKNESS FOR PIPING > NPS 4					
		MS-3A-EBB-2-1-SWB	UT	Yes		4/7/2000	Relief Request RR-B4 is applicable.
		6 IN. ELBOW TO PIPE WELD	MT	Yes		4/7/2000	
		MS-3A-EBB-2-7-SWA					
		6 IN. PIPE TO ELBOW WELD	MT	Yes		4/6/2000	
C-G	PRESSURE RETAINING WELDS IN PUMPS AND VALVES						
	C06.020	VALVES-VALVE BODY WELDS					
		MS-PV-ICS11A-WELD H	PT	Yes		3/5/2000	PRESERVICE EXAMINATION
		VALVE ICS11A BODY WELD					
		MS-PV-ICS11A-WELD I					
		VALVE ICS11A BODY WELD	PT	Yes		3/5/2000	PRESERVICE EXAMINATION
		MS-PV-ICS11A-WELD J					
		VALVE ICS11A BODY WELD	PT	Yes		3/5/2000	PRESERVICE EXAMINATION

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Code Class: 2							
C-G							
	PRESSURE RETAINING WELDS IN PUMPS AND VALVES						
	C06.020	VALVES-VALVE BODY WELDS					
		MS-PV-ICS11A-WELD K	PT	Yes		3/5/2000	PRESERVICE EXAMINATION
		VALVE ICS11A BODY WELD					
		MS-PV-ICS11B-WELD H	PT	Yes		3/4/2000	PRESERVICE EXAMINATION
		VALVE ICS11A BODY WELD					
		MS-PV-ICS11B-WELD I	PT	Yes		3/4/2000	PRESERVICE EXAMINATION
		VALVE ICS11A BODY WELD					
		MS-PV-ICS11B-WELD J	PT	Yes		3/4/2000	PRESERVICE EXAMINATION
		VALVE ICS11A BODY WELD					
		MS-PV-ICS11B-WELD K	PT	Yes		3/4/2000	PRESERVICE EXAMINATION
		VALVE ICS11A BODY WELD					

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Exam Category	Code Item	Component / Description	NDE Method	Examination Acceptable	CR Number	Exam Date	Comments
Code Class: 1							
F-A	SUPPORTS						
	F01.010	CLASS 1 PIPING SUPPORTS					
		DH-33C-CCA-4-H10	VT-3	Yes		4/22/2000	Relief Request RR-A9 is applicable. PRESERVICE EXAMINATION
		NONWELDED HYDRAULIC SNUBBER SUPPORT					
		RC-30-CCA-8-H4	VT-3	Yes		5/8/2000	Relief Request RR-A9 is applicable. PRESERVICE EXAMINATION
		HYDRAULIC SNUBBER SUPPORT					
		RC-40-CCA-18-H1	VT-3	Yes		4/10/2000	Relief Request RR-A9 is applicable.
		RIGID SUPPORT					
		RC-FSK-M-CCA-7-1-H1	VT-3	Yes		4/13/2000	Relief Request RR-A9 is applicable.
		RIGID SUPPORT					
		RC-FSK-M-CCA-7-1-H4	VT-3	Yes		4/13/2000	Relief Request RR-A9 is applicable.
		RIGID SUPPORT					

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Exam Category	Code Item	Component / Description	NDE Method	Examination Acceptable	CR Number	Exam Date	Comments
Code Class: 1							
F-A	SUPPORTS						
	F01.010	CLASS 1 PIPING SUPPORTS					
		RC-FSK-M-CCA-7-1-PS-H25	VT-3	Yes		4/10/2000	Relief Request RR-A9 is applicable.
		HYDRAULIC SNUBBER SUPPORT					
		RC-M1170-H2	VT-3	Yes		5/6/1999	Relief Request RR-A9 is applicable. PRESERVICE EXAMINATION
		HYDRAULIC SNUBBER SUPPORT	VT-3	Yes		5/4/2000	
		RC-M1170-H3	VT-3	Yes		5/4/2000	Relief Request RR-A9 is applicable. PRESERVICE EXAMINATION
		HYDRAULIC SNUBBER SUPPORT					
		RC-M1170-H4	VT-3	Yes		5/4/2000	Relief Request RR-A9 is applicable. PRESERVICE EXAMINATION
		RIGID SUPPORT/HYDRAULIC SNUBBER					
		RC-MK-A-103-PS-H37	VT-3	Yes		5/4/2000	Relief Request RR-A9 is applicable. PRESERVICE EXAMINATION
		HYDRAULIC SNUBBER SUPPORT					

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Code Class: 1							
F-A	SUPPORTS						
	F01.010	CLASS 1 PIPING SUPPORTS					
		RC-MK-A-135-PS-H36	VT-3	Yes		5/6/1999	Relief Request RR-A9 is applicable. PRESERVICE EXAMINATION
		HYDRAULIC SNUBBER SUPPORT	VT-3	Yes		5/4/2000	
		RC-MK-A-83-PSU-R1					
		HYDRAULIC SNUBBER SUPPORT	VT-3	Yes		4/22/2000	Relief Request RR-A9 is applicable. PRESERVICE EXAMINATION
		RC-MK-A-83-PSU-R2					
		HYDRAULIC SNUBBER SUPPORT	VT-3	Yes		4/22/2000	Relief Request RR-A9 is applicable. PRESERVICE EXAMINATION
		RC-MK-A-83-PSU-R3					
		HYDRAULIC SNUBBER SUPPORT	VT-3	Yes		4/22/2000	Relief Request RR-A9 is applicable. PRESERVICE EXAMINATION
Code Class: 2							
	F01.020	CLASS 2 PIPING SUPPORTS					
		AF-6C-EBB-4-H15	VT-3	Yes		4/22/2000	Relief Request RR-A9 is applicable. PRESERVICE EXAMINATION
		HYDRAULIC SNUBBER SUPPORT					

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Code Class: 2							
F-A	SUPPORTS						
	F01.020	CLASS 2 PIPING SUPPORTS					
		DH-33B-CCB-6-H12	VT-3	Yes		9/22/1998	Relief Request RR-A9 is applicable. PRESERVICE EXAMINATION
		HYDRAULIC SNUBBER SUPPORT					
		DH-33B-CCB-6-H2	VT-3	Yes		9/22/1998	Relief Request RR-A9 is applicable. PRESERVICE EXAMINATION
		HYDRAULIC SNUBBER SUPPORT	VT-3	Yes		4/15/2000	
		FW-7-EBB-3-2-SR23	VT-3	Yes		4/22/2000	Relief Request RR-A9 is applicable. PRESERVICE EXAMINATION
		HYDRAULIC SNUBBER SUPPORT					
		FW-7-EBB-3-5-SR20	VT-3	Yes		4/22/2000	Relief Request RR-A9 is applicable. PRESERVICE EXAMINATION
		HYDRAULIC SNUBBER SUPPORT					
		FW-7-EBB-3-5-SR21	VT-3	Yes		7/1/1998	Relief Request RR-A9 is applicable. PRESERVICE EXAMINATION
		HYDRAULIC SNUBBER SUPPORT					

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Code Class: 2							
F-A	SUPPORTS						
	F01.020	CLASS 2 PIPING SUPPORTS					
		FW-7-EBB-3-8-SR25D	VT-3	Yes		4/22/2000	Relief Request RR-A9 is applicable. PRESERVICE EXAMINATION
		HYDRAULIC SNUBBER SUPPORT					
		FW-7-EBB-3-9-SR24A	VT-3	Yes		4/22/2000	Relief Request RR-A9 is applicable. PRESERVICE EXAMINATION
		HYDRAULIC SNUBBER SUPPORT					
		FW-7-EBB-3-9-SR25A	VT-3	Yes		4/19/2000	Relief Request RR-A9 is applicable. SNUBBER RE-INSPECTED BY DAVIS-BESSE QC AND FOUND SATISFACTORY.
		HYDRAULIC SNUBBER SUPPORT					
		FW-7-EBB-3-SG2-SP8	VT-3	Yes		4/24/2000	Relief Request RR-A9 is applicable.
		RIGID SUPPORT					
		HP-33C-CCB-2-H18	VT-3	Yes		4/1/2000	Relief Request RR-A9 is applicable.
		RIGID SUPPORT					

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Code Class: 2							
F-A	SUPPORTS						
	F01.020	CLASS 2 PIPING SUPPORTS					
		HP-33C-CCB-2-H30	VT-3	Yes		4/5/2000	Relief Request RR-A9 is applicable.
		RIGID SUPPORT					
		HP-33C-CCB-2-H47	VT-3	Yes		4/28/2000	Relief Request RR-A9 is applicable. PRESERVICE EXAMINATION
		HYDRAULIC SNUBBER SUPPORT					
		HP-ANCHOR-A056	VT-3	Yes		4/26/2000	Relief Request RR-A9 is applicable. PRESERVICE EXAMINATION
		ANCHOR SUPPORT					
		HP-M2518-H1	VT-3	Yes		5/1/2000	Relief Request RR-A9 is applicable. PRESERVICE EXAMINATION
		NON-WELDED RIGID SUPPORT					
		HP-M2518-H2	VT-3	Yes		4/26/2000	Relief Request RR-A9 is applicable. PRESERVICE EXAMINATION
		NON-WELDED RIGID SUPPORT					

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Code Class: 2							
F-A	SUPPORTS						
	F01.020	CLASS 2 PIPING SUPPORTS					
		HP-M2518-H3	VT-3	Yes		4/26/2000	Relief Request RR-A9 is applicable. PRESERVICE EXAMINATION
		NON-WELDED RIGID SUPPORT					
		HP-M2518-H4	VT-3	Yes		4/26/2000	Relief Request RR-A9 is applicable. PRESERVICE EXAMINATION
		NON-WELDED RIGID SUPPORT					
		HP-M2518-H5	VT-3	Yes		4/26/2000	Relief Request RR-A9 is applicable. PRESERVICE EXAMINATION
		NON-WELDED RIGID SUPPORT					
		HP-M2518-H6	VT-3	Yes		5/9/2000	Relief Request RR-A9 is applicable. PRESERVICE EXAMINATION
		NON-WELDED RIGID SUPPORT					
		MS-3A-EBB-1-11-SR2	VT-3	Yes		4/24/2000	Relief Request RR-A9 is applicable.
		RIGID SUPPORT					

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Code Class: 2							
F-A	SUPPORTS						
	F01.020	CLASS 2 PIPING SUPPORTS					
		MS-3A-EBB-1-15-SR4	VT-3	Yes		4/22/2000	Relief Request RR-A9 is applicable. PRESERVICE EXAMINATION
		HYDRAULIC SNUBBER SUPPORT					
		MS-3A-EBB-1-17-H5	VT-3	Yes		4/17/2000	Relief Request RR-A9 is applicable.
		VARIABLE SPRING SUPPORT					
		MS-3A-EBB-1-17-SR8	VT-3	Yes		4/29/2000	Relief Request RR-A9 is applicable. PRESERVICE EXAMINATION
		HYDRAULIC SNUBBER SUPPORT					
		MS-3A-EBB-1-20-SR11	VT-3	Yes		3/31/1999	Relief Request RR-A9 is applicable. PRESERVICE EXAMINATION
		HYDRAULIC SNUBBER SUPPORT	VT-3	Yes		4/28/2000	
		MS-3A-EBB-1-28-SR19	VT-3	Yes		4/5/2000	Relief Request RR-A9 is applicable.
		HYDRAULIC SNUBBER SUPPORT					

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Code Class: 2							
F-A	SUPPORTS						
	F01.020	CLASS 2 PIPING SUPPORTS					
		MS-3A-EBB-1-6-SR4	VT-3	Yes		4/29/2000	Relief Request RR-A9 is applicable. PRESERVICE EXAMINATION
		HYDRAULIC SNUBBER SUPPORT					
		MS-3A-EBB-1-8-SR8	VT-3	Yes		4/29/2000	Relief Request RR-A9 is applicable. PRESERVICE EXAMINATION
		HYDRAULIC SNUBBER SUPPORT					
		MS-3A-EBB-1-H1	VT-3	Yes		4/28/2000	Relief Request RR-A9 is applicable. PRESERVICE EXAMINATION
		STRUT/HYDRAULIC SNUBBER SUPPORT ON VALVE ICS11A					
		MS-3A-EBB-1-H3	VT-3	Yes		4/28/2000	Relief Request RR-A9 is applicable. PRESERVICE EXAMINATION
		STRUT/HYDRAULIC SNUBBER SUPPORT					
Code Class: 1							
	F01.040	COMPONENT SUPPORTS					
		RC-RCP-1-2-1-SNUBBERS	VT-3	Yes		4/29/2000	Relief Request RR-A9 is applicable. PRESERVICE EXAMINATION
		REACTOR COOLANT PUMP 1-2-1 SNUBBERS					

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First Energy Nuclear Operating, 76 South Main St., Akron, Ohio 44308

Davis-Besse Nuclear Power Station Unit #1, 5501 N. St. Rt. 2, Oak Harbor, Ohio 43449

Commercial Service Date: 11/27/77

**Inservice Inspection Abstract of ASME Class 1 and Class 2 Examinations
12th Refueling Outage**

Exam Category	Code Item	Component / Description	NDE Method	Examination Acceptable	CR Number	Exam Date	Comments
Code Class: 1							
F-A	SUPPORTS						
	F01.040	COMPONENT SUPPORTS					
		RC-RCP-1-2-2-SNUBBERS	VT-3	Yes		4/29/2000	Relief Request RR-A9 is applicable. PRESERVICE EXAMINATION
		REACTOR COOLANT PUMP 1-2-2 SNUBBERS					
		 RC-RPV-X/Y AXIS SUPPORT					
		REACTOR VESSEL X/Y AXIS SUPPORT	VT-3	Yes		4/5/2000	Relief Request RR-A9 is applicable.
Code Class: 2							
	F01.20A	THIN WALL PIPING SUPPORTS					
		CS-33A-HCB-3-H15	VT-3	Yes		3/13/2000	Relief Request RR-A9 is applicable. PRESERVICE EXAMINATION
		HYDRAULIC SNUBBER SUPPORT					
		 CS-34-GCB-5-H32					
		HYDRAULIC SNUBBER SUPPORT	VT-3 VT-3	Yes Yes		3/10/2000 4/28/2000	Relief Request RR-A9 is applicable. PRESERVICE EXAMINATION
		 DH-33A-GCB-8-H11					
		HYDRAULIC SNUBBER SUPPORT	VT-3 VT-3	Yes Yes		4/1/1999 4/15/2000	Relief Request RR-A9 is applicable. PRESERVICE EXAMINATION

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Commercial Service Date: 11/27/77

**Inservice Inspection Abstract of ASME Class 1 and Class 2 Examinations
12th Refueling Outage**

Exam Category	Code Item	Component / Description	NDE Method	Examination Acceptable	CR Number	Exam Date	Comments
Code Class: 2							
F-A SUPPORTS							
	F01.20A	THIN WALL PIPING SUPPORTS					
		DH-33A-GCB-8-H13	VT-3	Yes		3/29/2000	Relief Request RR-A9 is applicable.
		VARIABLE SPRING SUPPORT					
		DH-33A-GCB-8-H4	VT-3	Yes		8/20/1998	Relief Request RR-A9 is applicable. PRESERVICE EXAMINATION
		HYDRAULIC SNUBBER SUPPORT					
		DH-33B-GCB-1-H25	VT-3	Yes		3/29/2000	Relief Request RR-A9 is applicable.
		WELDED SWAY STRUT SUPPORT					
		DH-33B-GCB-1-H32	VT-3	Yes		3/29/2000	Relief Request RR-A9 is applicable.
		NONWELDED RIGID SUPPORT					
		DH-33B-GCB-1-H4	VT-3	Yes		9/23/1998	Relief Request RR-A9 is applicable. PRESERVICE EXAMINATION
		HYDRAULIC SNUBBER SUPPORT					

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Davis-Besse Nuclear Power Station Unit #1, 5501 N. St. Rt. 2, Oak Harbor, Ohio 43449

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**Inservice Inspection Abstract of ASME Class 1 and Class 2 Examinations
12th Refueling Outage**

Exam Category	Code Item	Component / Description	NDE Method	Examination Acceptable	CR Number	Exam Date	Comments
Code Class: 2							
F-A	SUPPORTS						
	F01.20A	THIN WALL PIPING SUPPORTS					
		DH-33B-GCB-10-H13	VT-3	Yes		4/1/1999	Relief Request RR-A9 is applicable. PRESERVICE EXAMINATION
		WELDED HYDRAULIC SNUBBER SUPPORT					
		DH-33B-GCB-10-H29	VT-3	Yes		9/15/1998	Relief Request RR-A9 is applicable. PRESERVICE EXAMINATION
		WELDED HYDRAULIC SNUBBER SUPPORT					
		DH-33B-GCB-10-H32	VT-3	Yes		2/25/1999	Relief Request RR-A9 is applicable. PRESERVICE EXAMINATION
		WELDED HYDRAULIC SNUBBER SUPPORT					
		DH-33C-GCB-7-H3	VT-3	Yes		3/30/2000	Relief Request RR-A9 is applicable.
		HYDRAULIC SNUBBER SUPPORT					
		DH-ANCHOR-A059	VT-3	Yes		3/30/2000	Relief Request RR-A9 is applicable. POST HEATUP EXAMINATION
		ANCHOR SUPPORT					

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First Energy Nuclear Operating, 76 South Main St., Akron, Ohio 44308

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Inservice Inspection Abstract of ASME Class 1 and Class 2 Examinations
12th Refueling Outage

Exam Category	Code Item	Component / Description	NDE Method	Examination Acceptable	CR Number	Exam Date	Comments
Code Class: 2							
F-A	SUPPORTS						
	F01.20A	THIN WALL PIPING SUPPORTS					
		DH-ANCHOR-A060	VT-3	Yes		3/30/2000	Relief Request RR-A9 is applicable. POST HEATUP EXAMINATION
		ANCHOR					
		HP-33A-GCB-4-H6	VT-3	Yes		3/30/2000	Relief Request RR-A9 is applicable.
		RIGID SUPPORT					
		HP-35-HCC-21-H15	VT-3	Yes		4/5/2000	Relief Request RR-A9 is applicable.
		NONWELDED RIGID SUPPORT					
		MS-3A-EBB-2-H31	VT-3	Yes		9/18/1998	Relief Request RR-A9 is applicable. PRESERVICE EXAMINATION
		HYDRAULIC SNUBBER SUPPORT					
		MS-ANCHOR-A004	VT-3	Yes		4/5/2000	Relief Request RR-A9 is applicable.
		ANCHOR					

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First Energy Nuclear Operating, 76 South Main St., Akron, Ohio 44308

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Commercial Service Date: 11/27/77

Inservice Inspection Abstract of ASME Class 1 and Class 2 Examinations
12th Refuelling Outage

Exam Category	Code Item	Component / Description	NDE Method	Examination Acceptable	CR Number	Exam Date	Comments
Code Class: 2							
F-A	SUPPORTS						
	F01.20A	THIN WALL PIPING SUPPORTS					
		SW-41-HBB-7-H1	VT-3	Yes		4/5/2000	Relief Request RR-A9 is applicable.
		WELDED SWAY STRUT SUPPORT					

1. Owner FIRST ENERGY CORPORATION, 46 S. MAIN ST., AKRON, OHIO 44308
(Name and Address of Owner)

2. Plant DAVIS BESSE NUCLEAR POWER STATION, OAK HARBOR, OHIO 43449
(Name and Address of Plant)

3. Unit #1	4. Owner Certificate of Authorization (if required)	N/A
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5. Commercial Service Date 11/21/77 6. National Board Number for Unit N/A

7. Components Inspected

SYSTEM PRESSURE TESTS

[illegible]

Note: Supplemental sheets in form of lists, sketches, or drawings may be used provided (1) size is 8 1/2 in. x 11 in., (2) information in items 1 through 6 on this data report is included on each sheet, and (3) each sheet is numbered and the number of sheets recorded at the top of this form.

First Energy Nuclear Operating, 76 South Main St., Akron, Ohio 44308

Davis-Besse Nuclear Power Station Unit #1, 5501 N. St. Rt. 2, Oak Harbor, Ohio 43449

Commercial Service Date: 11/27/77

FORM NIS-1 (back)

8. Examination Dates MAY 1998 to MAY 2000 9. Inspection Interval from SEPT 1990 to SEPT 2000

10. Applicable Editions of Section XI 1986 Addenda None

11. Abstract of Examinations. Include a list of examinations and a statement concerning status of work required for current interval.

SEE ATTACHED PAGES

12. Abstract of Conditions Noted.

ALL CORRECTIVE MEASURES ARE IDENTIFIED IN THE SPECIFIC TEST PACKAGES AND ARE AVAILABLE FOR REVIEW IN DAVIS-BESSE'S RECORDS MANAGEMENT SYSTEM.

13. Abstract of Corrective Measures Recommended and Taken

ALL CORRECTIVE MEASURES ARE IDENTIFIED IN THE SPECIFIC TEST PACKAGES AND ARE AVAILABLE FOR REVIEW IN DAVIS -BESSE'S RECORDS MANAGEMENT SYSTEM.

Certificate of Authorization No. (if applicable) N/AExpiration Date N/A

We certify that the statements made in this report are correct and the examinations and corrective measures taken conform to the rules of the ASME Code, Section XI.

Date 8-10-00 Signed **FIRST ENERGY NUCLEAR**
OPERATING COMPANY

By Michael D. Shepley

CERTIFICATE OF INSERVICE INSPECTION

I, the undersigned, holding a valid commission issued by the National Board of Boiler and Pressure Vessel Inspectors and/or the State or Province of OHIO and employed by Factory Mutual Ins. Co. of Johnston, Rhode Island have inspected the components described in this Owners' Data Report during the period MAY 1998 to MAY 2000, and state that to the best of my knowledge and belief, the Owner has performed examinations and taken corrective measures described in this Owners' Data Report in accordance with the requirements of the ASME Code, Section XI.

By signing this certificate neither the Inspector nor his employer makes any warranty, expressed or implied, concerning the examinations and corrective measures described in this Owner's Date Report. Furthermore, neither the Inspector nor his employer shall be liable in any manner for any personal injury or property damage or a loss of any kind arising from or connected with this inspection.

Date AUGUST 10, 2000

Thomas A. Pagn
Inspector's Signature

Commissions

NB9330 "N I A" "OHIO"
National Board, State, Province and No. COMM.

SYSTEM PRESSURE TESTS

The following pressure tests were performed for Class 1 and 2 components from June, 1998, during the 12th Operating Cycle through the 12th Refueling Outage which was completed on May 18, 2000. Code Case N-498-1 (authorized by Relief Request RR-A15) and Code Case N-522 (authorized by Relief Request RR-B8) were utilized in the performance of these tests. These tests were performed in accordance with ASME Section XI and the Inservice Inspection Program and were performed on the dates given below. These tests do not include those associated with the repairs/replacements addressed in the Repair and Replacement Section of this report.

During the System Hydrostatic Test of the Reactor Coolant System, a incore/cable closure (flowerpot) located in the Incore Tank was observed to be leaking approximately 4 drops per minute (Condition Report 2000-1456). The top cover was re-torqued and the leakage was reduced to approximately 1 drop every 90 seconds. This leakage was found acceptable for continued service in accordance with the provisions of IWB-3142.4. The inclusion of this analysis in the ISI Summary Report fulfills the requirement of IWB-3144 that the evaluation analyses required by IWB-3142.4 be submitted to the regulatory authority having jurisdiction at the plant site.

The 1 drop every 90 seconds leakage at the incore/cable closure will contribute to the overall Reactor Coolant System leakage limits that are described in the Davis-Besse Technical Specifications. However, this leakage is so minor that it will not be noticeable in the leakage calculation required by the Technical Specifications. The leakage will be contained in the Incore Tank which has an open drain line to the Containment Sump. Leakage has occurred from the flowerpots during at least one previous occasion. No adverse affects resulted from this leakage. All of the components associated with the incore flowerpots are materials that are resistant to boric acid corrosion and further degradation. Leakage will drip into the Incore Tank which is stainless steel and also resistant to corrosion. The temperature of the leakage is not conducive to boric acid corrosion. Based on the minimal leakage and that the flowerpot and surrounding materials are resistant to boric acid corrosion, the flowerpot leakage noted is acceptable.

Test Zone	Zone Description	Test Type	Date Completed	Class
CV001	CTMT Vessel Leak Test Line (CTMT P17)	Hydrostatic	4/11/00	2
CS001	CS Pump 1-1 Suction	Hydrostatic	5/12/99	2
CS002	CS Pump 1-1 Discharge	Hydrostatic	5/12/99	2
CS004	CS Pump 1-2 Suction	Hydrostatic	11/12/98	2
CS005	CS Pump 1-2 Discharge	Hydrostatic	11/12/98	2
CV008	CTMT Purge Exhaust (CTMT P34)	Hydrostatic	6/3/98	2
CV009	CTMT Purge Supply (CTMT P33)	Hydrostatic	6/3/98	2
CV010	H2 Dilution System Exhaust (CTMT P51)	Hydrostatic	4/7/00	2
CV011	H2 Dilution Supply Train 1 (CTMT P67)	Hydrostatic	4/16/00	2

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Test Zone	Zone Description	Test Type	Date Completed	Class
CV012	H2 Dilution Supply Train 2 (CTMT P69)	Hydrostatic	4/16/00	2
DR001	Station Drainage (CTMT P13)	Hydrostatic	6/3/98	2
CF003	Common Cft Vent Line (CTMT P47b)	Hydrostatic	5/23/00	2
CF004	Cft 1-2 Fill Line (CTMT P44a)	Hydrostatic	6/3/98	2
CF005	Common Cft Bleed Line (CTMT P47a)	Hydrostatic	6/3/98	2
CF006	Cft 1-1 Fill Line (CTMT P71c)	Hydrostatic	6/3/98	2
CF007	Cft 1-1 Discharge To RCS	Hydrostatic	5/26/98	2
CF008	Cft 1-2 Discharge To RCS	Hydrostatic	5/26/98	2
CF009	Cf And Dh Train 1 Injection Lines To R/X Vessel	Hydrostatic	6/5/00	1
CF010	Cf And Dh Train 2 Injection Lines To R/X Vessel	Hydrostatic	5/19/00	1
DH002	Bwst Common Supply To EcCS Trains 1 & 2 Suction	Hydrostatic	1/11/00	2
DH003	Bwst Supply To EcCS #1 Suction	Hydrostatic	1/11/00	2
DH007	Bwst Supply To EcCS #2 Suction	Hydrostatic	10/5/99	2
DH008	RCS Supply To DHP 1-2	Functional	4/1/00	2
DH009	DHP 1-2 Discharge	Functional	4/9/00	2
DH012	Refuel Canal Fill (CTMT P49)	Functional	6/3/98	2
DH014	RCS To DHP Suction	Hydrostatic	6/5/00	1
DH015	RCS TO DHP SUCTION (Pressurize DH23 To DH21)	Hydrostatic	4/24/00	1
DH016	Pressurizer Spray Line	Hydrostatic	5/19/00	1
DW001	Demineralized Water Supply To CTMT (CTMT P21)	Hydrostatic	6/3/98	2
HP001	HPI Pump 1-1 Suction	Functional	10/23/98	2
HP002	HPI Pump 1-1 Discharge	Hydrostatic	10/23/98	2
MS001B	OTSG 1-1 And Ms Lines Inside Shield Bldg (Train1)	Hydrostatic	5/26/98	2
MS002B	OTSG 1-2 And Ms Lines Inside Shield Bldg (Train 2)	Hydrostatic	5/26/98	2
MS003	SG Chem Cleaning (CTMT P59)	Hydrostatic	6/3/98	2
MU001	Makeup (CTMT P14) From Letdown Coolers	Hydrostatic	6/3/98	2
MU002	Makeup (CTMT P54) From Seal Inj Filters To RCP 1-1-1	Hydrostatic	6/3/98	2

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Davis-Besse Nuclear Power Station Unit #1, 5501 N. St. Rt. 2, Oak Harbor, Ohio 43449

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Test Zone	Zone Description	Test Type	Date Completed	Class
MU003	Makeup (CTMT P55) From Seal Inj Filters To RCP 1-1-2	Hydrostatic	6/3/98	2
MU004	Makeup (CTMT P52) From Seal Inj Filters To RCP 1-2-1	Hydrostatic	6/3/98	2
MU005	Makeup (CTMT P53) From Seal Inj Filters To RCP 1-2-2	Hydrostatic	6/3/98	2
MU006	Makeup (CTMT P56) From RCP Seal Clg To Makeup	Hydrostatic	6/3/98	2
NN001	Nitrogen Supply (CTMT P44b)	Hydrostatic	4/3/00	2
RC001H	Primary Reactor Coolant System Class 1 Pipe For Hydrostatic Test	Hydrostatic	6/5/00	1
RC001L	Primary Reactor Coolant System Class 1 Pipe For Leakage Test	Leakage	6/5/00	1
RC001N	R/X Vessel Nozzles And Penetration Welds	Hydrostatic	6/5/00	1
RC003	Sample From RCS, (CTMT P1)	Hydrostatic	6/3/98	2
RC004	From RCP Seal Cavities To Gas Radwaste, (CTMT P16)	Hydrostatic	6/3/98	2
RC005	Pressurizer Quench Tank To Circ Pump, (CTMT P48)	Hydrostatic	6/3/98	2
RC006	Quench Tank Pump To Quench Tank, (CTMT P41)	Hydrostatic	6/3/98	2
RC007	Drains To Rc Drain Tank, (CTMT P32)	Hydrostatic	6/3/98	2
RC008	Vapor Space Of Quench Tank To Sample System, (CTMT P68a)	Hydrostatic	5/23/00	2
RC009	RCP 1-1-1 Gasket Drain	Inservice	4/26/00	2
RC010	RCP 1-1-2 Gasket Drain	Inservice	4/26/00	2
RC011	RCP 1-2-1 Gasket Drain	Inservice	4/26/00	2
RC012	RCP 1-2-2 Gasket Drain	Inservice	4/26/00	2
SF001	Fuel Transfer Tube 1-1	Hydrostatic	5/9/00	2
SF002	Fuel Transfer Tube 1-2	Hydrostatic	5/9/00	2
IA001	Instrument Air (CTMT P43a)	Hydrostatic	4/3/00	2
SA001	Station Air (CTMT P42a)	Hydrostatic	4/10/00	2
SA002	SA Supply To Test CS Lower Ring Hdr	Hydrostatic	5/20/99	2
SA003	SA Supply To Test CS Upper Ring Hdr	Hydrostatic	8/13/99	2

REPAIRS AND REPLACEMENTS

The following repairs and replacements on ASME Class 1 and 2 components were performed on Davis Besse Nuclear Power Station Unit #1 since May, 1998 which marked the completion of the 11th Refueling Outage and completion of the 12th Refueling Outage in May, 2000.

The appropriate Code Data Form is attached for each of these repairs. Complete documentation for these repairs and replacements is available for review in Toledo Edison's Records Management System.

WORK ORDER #	SYSTEM	DESCRIPTION
1-97-0626-09	Main Steam	Replaced Hanger 3A-EBB-2-H31 Snubber with Like for Like Replacement.
1-97-0626-14	Decay Heat	Replaced Hanger 33B-GCB-10-H29 Snubber with Like for Like Replacement.
1-97-0626-15	Decay Heat	Replaced Hanger 33B-CCB-6-H12 Snubber with Like for Like Replacement.
1-97-0626-16	Decay Heat	Replaced Hanger 33B-GCB-1-H4 Snubber with Like for Like Replacement.
1-97-0626-17	Decay Heat	Replaced Hanger 33B-GCB-1-H38 Snubber with Like for Like Replacement.
1-97-0626-18	Decay Heat	Replaced Hanger 33A-GCB-8-H4 Snubber with Like for Like Replacement.
1-98-0463-00	Reactor Coolant	Modified Hanger PS-H36.
1-98-0558-00	Reactor Coolant	Replaced Body to Bonnet nuts on Valve RC 2.
1-98-1155-00	Reactor Coolant	Replaced Body to Bonnet nuts on Valve RC 2.
98-000039-000, 98-000423-000, & 98-000423-001	Reactor Coolant	Rebuilt Spare Power Relief Valve and installed as Valve RC 2A.
98-000135-000	Main Steam	Replaced Hanger EBB-1-SR 11 (East) Snubber with Like for Like Replacement.
98-000139-000	Decay Heat	Replaced Hanger 33A-GCB-8-H11 Snubber with Like for Like Replacement.

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WORK ORDER #	SYSTEM	DESCRIPTION
98-000140-000	Decay Heat	Replaced Hanger 33A-GCB-10-H13 Snubber with Like for Like Replacement.
98-000141-000	Decay Heat	Replaced Hanger 33A-GCB-10-H32 Snubber with Like for Like Replacement.
98-000142-000	Decay Heat	Replaced Hanger 33B-GCB-10-H32 Snubber with Like for Like Replacement.
98-000373-000, 98-000373-003, 99-001196-000	Reactor Coolant	Replaced Bonnet, Disc, and Body to Bonnet Studs on Valve RC 2.
98-000456-002	Decay Heat	Replaced Flange Bolting on RO 1515.
98-000718-000	Decay Heat	Replaced Safety Valve DH 4849 with Spare Valve.
98-000913-001	Main Steam	Replaced Atmospheric Vent Valve ICS 11A with New Valve and installed new Valve to Flange Bolting.
98-000914-001	Main Steam	Replaced Atmospheric Vent Valve ICS 11B with New Valve and installed new Valve to Flange Bolting.
98-001201-000	Reactor Coolant	Rebuilt Spare Pressurizer Code Safety Valve.
98-001213-000	Reactor Coolant	Replaced Pressurizer Code Safety Valve RC 13A with a Spare Valve.
98-001214-000	Reactor Coolant	Replaced Pressurizer Code Safety Valve RC 13B with a Spare Valve.
98-001710-000	Containment	Replaced Containment Vacuum Relief Valve CV 5076 with a New Valve.
99-000269-000, 99-000269-003, & 99-000269-004	Makeup & Purification	Replaced Bonnet and Body to Bonnet Studs on Valve MU 1A.
99-000403-000	Core Flood	Replace Stem on Valve CF 1A.
99-001162-000	Main Steam	Replaced Main Steam Safety Valves SP 17B4, SP 17B6, and SP 17B7 with re-built valves and replaced Flange Bolting on Valve SP-17B4.
99-001365-000 & 99-003279-002	Main Steam	Fabricated and Replaced Two Load Pins on Hanger EBB-1-SR4 Snubber C-224.
99-001365-000 & 99-003279-002	Main Steam	Fabricated and Replaced Two Load Pins on Hanger EBB-1-SR4 Snubber C-225.

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WORK ORDER #	SYSTEM	DESCRIPTION
99-001791-001	High Pressure Injection	Replaced Body to Bonnet Bolting on Valve HP 57.
99-002640-000 & 99-002763-007	Reactor Coolant	Replaced Body to Bonnet Bolting on Valve RC 33.
99-002705-000	Reactor Coolant	Replaced Body to Bonnet Bolting on Valve RC 38.
99-002965-00	Reactor Coolant	Replaced Body to Bonnet Bolting on Valve RC 262.
99-003278-000	Main Steam	Replaced Hanger EBB-1-SR4 Snubber with Like for Like Replacement.
99-003279-000	Main Steam	Replaced Hanger EBB-1-SR4 West Bottom Snubber with Like for Like Replacement.
99-003333-000	Main Steam	Replaced Main Steam Safety Valves SP 17A5, SP 17A7, and SP 17A9 with re-built valves and replaced Flange Bolting on Valve SP-17A9.
99-003334-000	Main Steam	Replaced Main Steam Safety Valves SP 17B3, SP 17B6, and SP 17B7 with re-built valves and replaced Flange Bolting.
99-003339-000	Main Steam	Replaced Hanger EBB-1-SR4 Snubber with Like for Like Replacement.
99-003340-000	Main Steam	Replaced Hanger EBB-1-SR4 Snubber with Like for Like Replacement.
99-003398-000	Reactor Coolant	Replaced Pressurizer Code Safety Valve RC 13A with a Spare Valve.
99-003399-000	Reactor Coolant	Replaced Pressurizer Code Safety Valve RC 13B with a Spare Valve.
99-003666-000	Reactor Coolant	Replace Reactor Coolant Pump 1-2 Seal Cartridge.
99-004337-008	Decay Heat	Installed new piping and four valves to tie the High Pressure Injection System to the Decay Heat System.
99-004337-009	Decay Heat	Installed supports on new piping which tied the High Pressure Injection System to the Decay Heat System.
99-005123-000	Reactor Coolant	Rebuilt Spare Pressurizer Safety Relief Valve.

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WORK ORDER #	SYSTEM	DESCRIPTION
99-005147-056	Containment Spray	Replaced Hanger 33A-HCB-3-H15 Snubber with Like for Like Replacement.
99-005148-002	Auxiliary Feedwater	Replaced Hanger 6C-EBB-4-H15 Snubber with Like for Like Replacement.
99-005148-006	Reactor Coolant	Replaced Hanger PSU-R1Snubber with Like for Like Replacement.
99-005148-010	Decay Heat	Replaced Hanger 33B-CCB-6-H2 Snubber with Like for Like Replacement.
99-005148-012	Reactor Coolant	Replaced Reactor Coolant Pump Snubber RCP 3-5 with Like for Like Replacement.
99-005148-014	Reactor Coolant	Replaced Reactor Coolant Pump Snubber RCP 3-6 with Like for Like Replacement.
99-005148-016	Main Steam	Replaced Hanger EBB-1-SR8 West Snubber with Like for Like Replacement.
99-005148-018	Main Steam	Replaced Hanger EBB-1-SR8 Snubber with Like for Like Replacement.
99-005148-020	Steam Generator	Replaced Hanger EBB-5-12-H2 Snubber with Like for Like Replacement.
99-005148-026	Steam Generator	Replaced Hanger EBB-5-17-H1 Snubber with Like for Like Replacement.
99-005148-034	Main Steam	Replaced Hanger 3A-EBB-1-H1 (X) Snubber with Like for Like Replacement.
99-005148-036	Main Steam	Replaced Hanger 3A-EBB-1-H1 (Z) Snubber with Like for Like Replacement.
99-005148-038	Containment Spray	Replaced Hanger 34-GCB-5-H32 Snubber with Like for Like Replacement.
99-005148-046	Reactor Coolant	Replaced Hanger 30-CCA-8-H4 Snubber with Like for Like Replacement.
99-005149-004	Main Feedwater	Replaced Hanger EBB-3-SR24A Snubber with Like for Like Replacement.

WORK ORDER #	SYSTEM	DESCRIPTION
99-005149-006	Main Feedwater	Replaced Hanger EBB-3-SR25A Snubber with Like for Like Replacement.
99-005149-008	Main Feedwater	Replaced Hanger EBB-3-SR20 Snubber with Like for Like Replacement.
99-005149-014	Steam Generator	Replaced Hanger EBB-5-9-H1Snubber with Like for Like Replacement.
99-005149-024	Main Steam	Replaced Hanger EBB-3-SR11 West Snubber with Like for Like Replacement.
99-005152-000	Reactor Coolant	Rebuilt Spare Pressurizer Safety Relief Valve.
99-006520-001	Radiation Monitoring	Installed new line to facilitate draining of RE 4597AA.
00-000328-000	Auxiliary Feedwater	Replaced nut on Auxiliary Feedwater Header Riser #4.
00-001712-000	Main Steam	Replaced Main Steam Line #2 Warmup Isolation Valve MS 700.
Framatone Technologies Inc.	Control Rod Drives	Replaced flange bolting at cooling water manifolds for Control Rod Drives.
Framatone Technologies Inc.	Reactor Coolant	Replaced hold down bolts and split rings on Control Rod Drive to Reactor Vessel at Locations C11, D10, F8, F10 & G9.
Framatone Technologies Inc.	Steam Generator	Repaired Steam Generator #1 by rolled plugs.
Framatone Technologies Inc.	Steam Generator	Repaired Steam Generator #2 by rolled plugs.

First Energy Nuclear Operating, 76 South Main St., Akron, Ohio 44308

Davis-Besse Nuclear Power Station Unit #1, 5501 N. St. Rt. 2, Oak Harbor, Ohio 43449

Commercial Service Date: 11/27/77

FORM NIS-2 / NR-1 / NVR-1 OWNER'S REPORT FOR REPAIRS OR REPLACEMENTS
 As Required by the Provisions of the ASME Code Section XI

1. Owner Toledo Edison Company Date 8/23/98
Name
300 Madison Ave., Toledo, OH 43652
Address
2. Plant Davis-Besse Nuclear Power Station Unit # 1
Name
5501 N. SR 2, Oak Harbor, OH 43449
Address MWO# 1-97-0626-09
Repair Organization P.O. No., Job No., etc.
3. Work Performed By Toledo Edison Company Type Code Stamp NR
Name
5501 N. SR 2, Oak Harbor, OH 43449
Address Authorization NO. 20
 Expiration Date 7-17-01
4. Identification of Systems Main Steam to Aux. Feed Pumps (3A-EBB-2-H31)
5. (a) Applicable Construction Code ANSI B31.1 Edition 19 67 Addendum n/a Code Case n/a
 (b) Applicable Edition of Section XI Utilized for Repairs or Replacements 19 86
 (c) Design Responsibilities ITT Grinnell

6. Identification of Components Repaired or Replaced and Replacement Components

Name of Component	Name of Manufacturer	Manufacturer's Serial NO.	National Board NO.	Other Identification/Jurisdictional NO.	Year Built	Repaired Replaced or Replacement	ASME Code Stamped
Snubber	ITT Grinnell	17883	n/a	A-52	n/a	Replaced	No
Snubber	ITT Grinnell	12085	n/a	A-52	n/a	Replacement	No

7. Description of Work

ITT Grinnell snubber S/N 17883 was removed and replaced with like-for-like snubber S/N 12085. Replacement snubber S/N 12085 was rebuilt under MWO# 1-97-0515-16 and was functionally tested satisfactorily.

8. Tests Conducted: Hydrostatic (NO) Pneumatic (NO) Nominal Operating Pressure (NO) Other (NO)
 Pressure n/a psi Test Temperature n/a °F
- Pressure Relief Valves:
 Service n/a Size n/a
 Opening Pressure n/a Blowdown (if applicable) n/a
 Set Pressure and Blowdown Adjustments Made Using n/a
(Test Medium)
 at n/a
(Location)

NOTE: Supplemental sheets in the form of lists, sketches, or drawings may be used, provided (1) size is 8 1/2 x 11 in. (2) Information in items 1 through 6 on this report is included on each sheet, and (3) each sheet is numbered and the number of sheets is recorded at the top of this form.

First Energy Nuclear Operating, 76 South Main St., Akron, Ohio 44308

Davis-Besse Nuclear Power Station Unit #1, 5501 N. St. Rt. 2, Oak Harbor, Ohio 43449

Commercial Service Date: 11/27/77

FORM NIS-2 / NR-1 / NVR-4	
8. REMARKS	Applicable Manufacturers Data Reports to be Attached
N/A	
<p style="text-align: center;">CERTIFICATE OF COMPLIANCE</p> <p>We certify that the statements made in the report are correct and this <u>replacement</u> conforms to the rules of the ASME Code Section XI.</p> <p>Type Code Symbol <u>NR</u> Certificate of Authorization No. <u>20</u> Expiration Date <u>7-17-01</u></p> <p>Signed <u>D.W.S. Lamm, Jr.</u> Nuclear Safety & Inspection Manager Date <u>9/23</u> 19<u>98</u></p> <p style="text-align: center; font-size: small;">Owner or Owner's Designee, Title</p>	
<p style="text-align: center;">CERTIFICATE OF INSERVICE INSPECTION</p> <p>I, the undersigned, holding a valid commission issued by the National Board of Boiler and Pressure Vessel Inspectors in the State or Province of <u>Ohio</u> and employed by <u>Protection Mutual</u> of <u>Park Ridge, Illinois</u> have inspected the components described in this Owner's Report during the period <u>11/4/97</u> to <u>10/1/98</u>, and state that to the best of my knowledge and belief, the Owner has performed examinations and taken corrective measures described in this Owner's Report in accordance with the requirements of the ASME Code, Section XI.</p> <p>By signing this certificate, neither the Inspector nor his employer makes any warranty, expressed or implied, concerning the examinations and corrective measures described in the Owner's Report. Furthermore, neither the Inspector nor his employer shall be liable in any manner for any personal injury or property damage or a loss of any kind arising from or connected with this inspection.</p> <p>Inspector's Signature <u>Thomas J. Lamm</u> Date <u>10/1/98</u> Commissions <u>National Board, State, Province and Endorsements</u></p>	
<p style="text-align: center;">CERTIFICATE OF COMPLIANCE</p> <p>We certify that the statements made in this report are correct and the repair, modification, or replacement of the pressure relief device described above conforms to Section XI and Section III of the ASME Code and the National Board rules as defined in publications NB-65 and NB-102, current editions.</p> <p>Certificate of Authorization No. <u>316</u> to use the "VR" stamp expires <u>Dec. 13</u> 19<u>98</u>.</p> <p>Certificate of Authorization No. <u>20</u> to use the "NR" stamp expires <u>July 17</u> 2001</p> <p>Date _____, 19____, Signed <u>Toledo Edison</u> Nuclear Safety & Inspection Manager</p> <p style="text-align: center; font-size: small;">Repair Organization Authorized Representative Title</p>	
<p style="text-align: center;">CERTIFICATE OF INSPECTION</p> <p>I, the undersigned, holding a valid commission issued by the National Board of Boiler and Pressure Vessel Inspectors, and certificate of competency issued by the State or Province of <u>Ohio</u> and employed by <u>Protection Mutual</u> of <u>Park Ridge, Illinois</u> have inspected the repair, modification, or replacement described in this report on _____, 19____ and state that to the best of my knowledge and belief, this repair, modification, or replacement has been made or constructed in accordance with Section XI and Section III of the ASME Code and the National Board rules as defined in the publication NB-65 and NB-102, current editions.</p> <p>By signing this certificate, neither the Inspector nor his employer makes any warranty, expressed or implied, concerning the repair, modification or replacement described in this report. Furthermore, neither the Inspector nor his employer shall be liable in any manner for any personal injury or property damage or a loss of any kind arising from or connected with this inspection.</p> <p>Authorized Inspector's Signature _____ Date _____ Commissions <u>National Board, State, Province and Endorsements</u></p> <p style="text-align: center;">* Factory Mutual Engineering Association</p>	

First Energy Nuclear Operating, 76 South Main St., Akron, Ohio 44308

Davis-Besse Nuclear Power Station Unit #1, 5501 N. St. Rt. 2, Oak Harbor, Ohio 43449

Commercial Service Date: 11/27/77

FORM NIS-2 / NR-1 / NVR-1 OWNER'S REPORT FOR REPAIRS OR REPLACEMENTS As Required by the Provisions of the ASME Code Section XI

1. Owner Toledo Edison Company Date 10/1/98
300 Madison Ave., Toledo, OH 43652 Sheet 1 of 1
 2. Plant Davis-Besse Nuclear Power Station Unit # 1
5501 N. SR 2, Oak Harbor, OH 43449 MWO# 1-97-0626-14
 3. Work Performed By Toledo Edison Company Type Code Stamp NR
5501 N. SR 2, Oak Harbor, OH 43449 Authorization NO. 20
 Expiration Date 7-17-01

4. Identification of Systems DH REMOVAL COOLER 1-1 TO RCS 33B-GCB-10-H29 (A-77)

5. (a) Applicable Construction Code ANSI B31.1 Edition 19 87 Addendum N/A Code Case N/A
 (b) Applicable Edition of Section XI Utilized for Repairs or Replacements 19 86
 (c) Design Responsibilities ITT GRINNELL

6. Identification of Components Repaired or Replaced and Replacement Components

Name of Component	Name of Manufacturer	Manufacturer's Serial NO.	National Board NO.	Other Identification/Jurisdictional NO.	Year Built	Repaired Replaced or Replacement	ASME Code Stamped
SNUBBER	ITT GRINNELL	12036	N/A	A-77	N/A	REPLACED	No
SNUBBER	ITTGRINNELL	12186	N/A	A-77	N/A	REPLACEMENT	No

7. Description of Work

ITT GRINNELL SNUBBER S/N 12036 WAS REMOVED AND REPLACED WITH LIKE-FOR-LIKE SNUBBER S/N 12186. REPLACEMENT
 SNUBBER S/N 12186 WAS REBUILT UNDER MWO# 1-97-0515-15 AND WAS FUNCTIONALLY TESTED SATISFACTORILY.

8. Tests Conducted: Hydrostatic (NO) Pneumatic (NO) Nominal Operating Pressure (NO) Other (NO)
 Pressure N/A psi Test Temperature N/A °F

Pressure Relief Valves:
 Service N/A Size N/A
 Opening Pressure N/A Blowdown (if applicable) N/A
 Set Pressure and Blowdown Adjustments Made Using N/A
 at N/A

NOTE: Supplemental sheets in the form of lists, sketches, or drawings may be used, provided (1) size is 8 1/2 x 11 in. (2) information in items 1 through 6 on this report is included on each sheet, and (3) each sheet is numbered and the number of sheets is recorded at the top of this form.

First Energy Nuclear Operating, 76 South Main St., Akron, Ohio 44308

Davis-Besse Nuclear Power Station Unit #1, 5501 N. St. Rt. 2, Oak Harbor, Ohio 43449

Commercial Service Date: 11/27/77

FORM NIS-2 / NR-1 / NVR-1

9. REMARKS

Applicable Manufacturers Data Reports to be Attached

N/A

CERTIFICATE OF COMPLIANCE

We certify that the statements made in the report are correct and this replacement conforms to the rules of the ASME Code Section XI.

Type Code Symbol NR Certificate of Authorization No. 20 Expiration Date 7-17-01

Signed J. W. S. [Signature] Nuclear Safety & Inspection Manager Date 10/1 1998

Owner or Owner's Designee, Title

CERTIFICATE OF INSERVICE INSPECTION

I, the undersigned, holding a valid commission issued by the National Board of Boiler and Pressure Vessel Inspectors in the State or Province of Ohio and employed by Protection Mutual of Park Ridge, Illinois have inspected the components described in this Owner's Report during the period 11/18/97 to 10/6/98, and state that to the best of my knowledge and belief, the Owner has performed examinations and taken corrective measures described in this Owner's Report in accordance with the requirements of the ASME Code, Section XI.

By signing this certificate, neither the Inspector nor his employer makes any warranty, expressed or implied, concerning the examinations and corrective measures described in the Owner's Report. Furthermore, neither the Inspector nor his employer shall be liable in any manner for any personal injury or property damage or a loss of any kind arising from or connected with this inspection.

Thomas J. [Signature]
Inspector's Signature

10/6/98
Date

Commissions NB 9330 N I A OHIO
National Board, State, Province and Endorsements COMM.

CERTIFICATE OF COMPLIANCE

We certify that the statements made in this report are correct and the repair, modification, or replacement of the pressure relief device described above conforms to Section XI and Section III of the ASME Code and the National Board rules as defined in publications NB-65 and NB-102, current editions.

Certificate of Authorization No. 316 to use the "VR" stamp expires Dec. 13, 1998.

Certificate of Authorization No. 20 to use the "NR" stamp expires July 17, 2001

Date _____, 19____, Signed Toledo Edison Nuclear Safety & Inspection Manager
Repair Organization Authorized Representative Title

CERTIFICATE OF INSPECTION

I, the undersigned, holding a valid commission issued by the National Board of Boiler and Pressure Vessel Inspectors, and certificate of competency issued by the State or Province of Ohio and employed by Protection Mutual

of Park Ridge, Illinois have inspected the repair, modification, or replacement described in this report

on _____, 19____ and state that to the best of my knowledge and belief, this repair, modification, or replacement has been made or constructed in accordance with Section XI and Section III of the ASME Code and the National Board rules as defined in the publication NB-65 and NB-102, current editions.

By signing this certificate, neither the Inspector nor his employer makes any warranty, expressed or implied, concerning the repair, modification or replacement described in this report. Furthermore, neither the Inspector nor his employer shall be liable in any manner for any personal injury or property damage or a loss of any kind arising from or connected with this inspection.

Authorized Inspector's Signature _____ Date _____

Commissions _____
National Board, State, Province and Endorsements

* Factory Mutual Engineering Association

First Energy Nuclear Operating, 76 South Main St., Akron, Ohio 44308

Davis-Besse Nuclear Power Station Unit #1, 5501 N. St. Rt. 2, Oak Harbor, Ohio 43449

Commercial Service Date: 11/27/77

FORM NIS-2 / NR-1 / NVR-4 OWNER'S REPORT FOR REPAIRS OR REPLACEMENTS As Required by the Provisions of the ASME Code Section XI

1. Owner Toledo Edison Company Date 10/8/98
Name
300 Madison Ave., Toledo, OH 43652
Address
 Sheet 1 of 1
2. Plant Davis-Besse Nuclear Power Station Unit # 1
Name
5501 N. SR 2, Oak Harbor, OH 43449
Address
MWO # 1-97-0626-15
Repair Organization P.O. No., Job No., etc.
3. Work Performed By Toledo Edison Company Type Code Stamp NR
Name
5501 N. SR 2, Oak Harbor, OH 43449
Address
 Authorization NO. 20
 Expiration Date 7-17-01
4. Identification of Systems LP1 LOOP 2 33B-CCB-6-H12 (A-80)
5. (a) Applicable Construction Code ANSI B31.1 Edition 19 67 Addendum N/A Code Case N/A
 (b) Applicable Edition of Section XI Utilized for Repairs or Replacements 19 86
 (c) Design Responsibilities ITT GRINNELL

6. Identification of Components Repaired or Replaced and Replacement Components

Name of Component	Name of Manufacturer	Manufacturer's Serial NO.	National Board NO.	Other Identification/Jurisdictional NO.	Year Built	Repaired Replaced or Replacement	ASME Code Stamped
SNUBBER	ITT GRINNELL	24077	N/A	A-80	N/A	REPLACED	No
SNUBBER	ITT GRINNELL	15995	N/A	A-80	N/A	REPLACEMENT	No

7. Description of Work

ITT GRINNELL SNUBBER S/N 24077 WAS REMOVED AND REPLACED WITH LIKE-FOR-LIKE SNUBBER S/N 15995. REPLACEMENT
 SNUBBER S/N 15995 WAS REBUILT UNDER MWO# 1-97-0515-11 AND WAS FUNCTIONALLY TESTED SATISFACTORILY.

8. Tests Conducted: Hydrostatic (NO) Pneumatic (NO) Nominal Operating Pressure (NO) Other (NO)
 Pressure N/A psi Test Temperature N/A °F
- Pressure Relief Valves:
 Service N/A Size N/A
 Opening Pressure N/A Blowdown (if applicable) N/A
 Set Pressure and Blowdown Adjustments Made Using N/A
(Test Medium)
 at N/A
(Location)

NOTE: Supplemental sheets in the form of lists, sketches, or drawings may be used, provided (1) size is 8 1/2 x 11 in. (2) information in items 1 through 6 on this report is included on each sheet, and (3) each sheet is numbered and the number of sheets is recorded at the top of this form.

First Energy Nuclear Operating, 76 South Main St., Akron, Ohio 44308

Davis-Besse Nuclear Power Station Unit #1, 5501 N. St. Rt. 2, Oak Harbor, Ohio 43449

Commercial Service Date: 11/27/77

FORM NIS-2 /NR-1 /NVR-1	
8. REMARKS N/A	<small>Applicable Manufacturers Data Reports to be Attached</small>
<p style="text-align: center;">CERTIFICATE OF COMPLIANCE</p> <p>We certify that the statements made in the report are correct and this <u>replacement</u> conforms to the rules of the ASME Code Section XI.</p> <p>Type Code Symbol <u>NR</u> Certificate of Authorization No. <u>20</u> Expiration Date <u>7-17-01</u></p> <p>Signed <u>W. W. Stearns</u> Nuclear Safety & Inspection Manager Date <u>10/8</u> 19 <u>98</u> <small>Owner or Owner's Designee, Title</small></p>	
<p style="text-align: center;">CERTIFICATE OF INSERVICE INSPECTION</p> <p>I, the undersigned, holding a valid commission issued by the National Board of Boiler and Pressure Vessel Inspectors in the State or Province of <u>Ohio</u> and employed by <u>Protection Mutual *</u> of <u>Park Ridge, Illinois</u>, have inspected the components described in this Owner's Report during the period <u>11/18/97</u> to <u>10/8/98</u>, and state that to the best of my knowledge and belief, the Owner has performed examinations and taken corrective measures described in this Owner's Report in accordance with the requirements of the ASME Code, Section XI.</p> <p>By signing this certificate, neither the Inspector nor his employer makes any warranty, expressed or implied, concerning the examinations and corrective measures described in the Owner's Report. Furthermore, neither the Inspector nor his employer shall be liable in any manner for any personal injury or property damage or a loss of any kind arising from or connected with this inspection.</p> <p>Inspector's Signature <u>Thomas H. Laps</u> Date <u>10/8/98</u> Commissions <u>NB9330</u> <u>N I A</u> <u>OHIO</u> <small>National Board, State, Province and Endorsements</small></p>	
<p style="text-align: center;">CERTIFICATE OF COMPLIANCE</p> <p>We certify that the statements made in this report are correct and the repair, modification, or replacement of the pressure relief device described above conforms to Section XI and Section III of the ASME Code and the National Board rules as defined in publications NB-65 and NB-102, current editions.</p> <p>Certificate of Authorization No. <u>316</u> to use the "VR" stamp expires <u>Dec. 13</u>, 19 <u>98</u>.</p> <p>Certificate of Authorization No. <u>20</u> to use the "NR" stamp expires <u>July 17</u>, 20<u>01</u></p> <p>Date <u> </u>, 19 <u> </u>, Signed <u>Toledo Edison</u> <u> </u> <u> </u> <small>Repair Organization Authorized Representative Title</small> <u>Nuclear Safety & Inspection Manager</u></p>	
<p style="text-align: center;">CERTIFICATE OF INSPECTION</p> <p>I, the undersigned, holding a valid commission issued by the National Board of Boiler and Pressure Vessel Inspectors, and certificate of competency issued by the State or Province of <u>Ohio</u> and employed by <u>Protection Mutual *</u> of <u>Park Ridge, Illinois</u> have inspected the repair, modification, or replacement described in this report on <u> </u>, 19 <u> </u> and state that to the best of my knowledge and belief, this repair, modification, or replacement has been made or constructed in accordance with Section XI and Section III of the ASME Code and the National Board rules as defined in the publication NB-65 and NB-102, current editions.</p> <p>By signing this certificate, neither the Inspector nor his employer makes any warranty, expressed or implied, concerning the repair, modification or replacement described in this report. Furthermore, neither the Inspector nor his employer shall be liable in any manner for any personal injury or property damage or a loss of any kind arising from or connected with this inspection.</p> <p>Authorized Inspector's Signature <u> </u> Date <u> </u> Commissions <u> </u> <small>National Board, State, Province and Endorsements</small></p> <p style="text-align: right;">* Factory Mutual Engineering Association</p>	

First Energy Nuclear Operating, 76 South Main St., Akron, Ohio 44308

Davis-Besse Nuclear Power Station Unit #1, 5501 N. St. Rt. 2, Oak Harbor, Ohio 43449

Commercial Service Date: 11/27/77

FORM NIS-2 / NR-1 / NVR-1-OWNER'S REPORT FOR REPAIRS OR REPLACEMENTS
 As Required by the Provisions of the ASME Code Section XI

1. Owner Toledo Edison Company Date 9/28/98
300 Madison Ave., Toledo, OH 43652 Sheet 1 of 1

2. Plant Davis-Besse Nuclear Power Station Unit #1
5501 N. SR 2, Oak Harbor, OH 43449 MWO# 1-97-0626-16

3. Work Performed By Toledo Edison Company Type Code Stamp NR
5501 N. SR 2, Oak Harbor, OH 43449 Authorization NO. 20
 Expiration Date 7-17-01

4. Identification of Systems DH PUMP1-2 DISCHARGE 33B-GCB-1-H4 (A-87)

5. (a) Applicable Construction Code ANSI B31.1 Edition 19 87 Addendum N/A Code Case N/A
 (b) Applicable Edition of Section XI Utilized for Repairs or Replacements 19 86
 (c) Design Responsibilities ITT GRINNELL

6. Identification of Components Repaired or Replaced and Replacement Components

Name of Component	Name of Manufacturer	Manufacturer's Serial NO.	National Board NO.	Other Identification/Jurisdictional NO.	Year Built	Repaired Replaced or Replacement	ASME Code Stamped
SNUBBER	ITT GRINNELL	14200	N/A	A-87	N/A	REPLACED	No
SNUBBER	ITT GRINNELL	9441	N/A	A-87	N/A	REPLACEMENT	No

7. Description of Work

ITT GRINNELL SNUBBER S/N 14200 WAS REMOVED AND REPLACED WITH LIKE-FOR-LIKE SNUBBER S/N 9441. REPLACEMENT
 SNUBBER S/N 9441 WAS REBUILT UNDER MWO# 1-97-0515-15 AND WAS FUNCTIONALLY TESTED SATISFACTORILY.

8. Tests Conducted: Hydrostatic (NO) Pneumatic (NO) Nominal Operating Pressure (NO) Other (NO)
 Pressure N/A psi Test Temperature N/A °F

Pressure Relief Valves:
 Service N/A Size N/A
 Opening Pressure N/A Blowdown (if applicable) N/A
 Set Pressure and Blowdown Adjustments Made Using N/A
 at N/A

NOTE: Supplemental sheets in the form of lists, sketches, or drawings may be used, provided (1) size is 8 1/2 x 11 in.
 (2) Information in items 1 through 6 on this report is included on each sheet, and (3) each sheet is numbered and the number of sheets is recorded at the top of this form.

12 RFO Inservice Inspection Summary

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First Energy Nuclear Operating, 76 South Main St., Akron, Ohio 44308

Davis-Besse Nuclear Power Station Unit #1, 5501 N. St. Rt. 2, Oak Harbor, Ohio 43449

Commercial Service Date: 11/27/77

FORM NIS-2 / NR-1 / NVR-1	
B. REMARKS	Applicable Manufacturers Data Reports to be Attached
WA	
CERTIFICATE OF COMPLIANCE	
<p>We certify that the statements made in the report are correct and this <u>replacement</u> conforms to the rules of the ASME Code Section XI.</p> <p>Type Code Symbol <u>NR</u> Certificate of Authorization No. <u>20</u> Expiration Date <u>7-17-01</u></p> <p>Signed <u>J. W. Starnes</u> Nuclear Safety & Inspection Manager Date <u>9/28</u> 19<u>98</u></p> <p style="font-size: x-small; text-align: center;">Owner or Owner's Designee, Title</p>	
CERTIFICATE OF INSERVICE INSPECTION	
<p>I, the undersigned, holding a valid commission issued by the National Board of Boiler and Pressure Vessel Inspectors in the State or Province of <u>Ohio</u> and employed by <u>Protection Mutual</u> of <u>Park Ridge, Illinois</u> have inspected the components described in this Owner's Report during the period <u>11/27/77</u> to <u>9/28/98</u>, and state that to the best of my knowledge and belief, the Owner has performed examinations and taken corrective measures described in this Owner's Report in accordance with the requirements of the ASME Code, Section XI.</p> <p>By signing this certificate, neither the inspector nor his employer makes any warranty, expressed or implied, concerning the examinations and corrective measures described in the Owner's Report. Furthermore, neither the inspector nor his employer shall be liable in any manner for any personal injury or property damage or a loss of any kind arising from or connected with this inspection.</p> <p>Inspector's Signature <u>Thomas H. Lipe</u> Date <u>9/28/98</u> Commissions <u>NB 9330</u> "N I A" <u>Ohio</u></p> <p style="font-size: x-small; text-align: center;">National Board, State, Province and Endorsements</p>	
CERTIFICATE OF COMPLIANCE	
<p>We certify that the statements made in this report are correct and the repair, modification, or replacement of the pressure relief device described above conforms to Section XI and Section III of the ASME Code and the National Board rules as defined in publications NB-65 and NB-102, current editions.</p> <p>Certificate of Authorization No. <u>318</u> to use the "VR" stamp expires <u>Dec. 13</u>, 19<u>98</u>.</p> <p>Certificate of Authorization No. <u>20</u> to use the "NR" stamp expires <u>July 17</u>, 2001</p> <p>Date _____, 19____, Signed <u>Toledo Edison</u> Nuclear Safety & Inspection Manager</p> <p style="font-size: x-small; text-align: center;">Repair Organization Authorized Representative Title</p>	
CERTIFICATE OF INSPECTION	
<p>I, the undersigned, holding a valid commission issued by the National Board of Boiler and Pressure Vessel Inspectors, and certificate of competency issued by the State or Province of <u>Ohio</u> and employed by <u>Protection Mutual</u> of <u>Park Ridge, Illinois</u> have inspected the repair, modification, or replacement described in this report on _____, 19____ and state that to the best of my knowledge and belief, this repair, modification, or replacement has been made or constructed in accordance with Section XI and Section III of the ASME Code and the National Board rules as defined in the publication NB-65 and NB-102, current editions.</p> <p>By signing this certificate, neither the inspector nor his employer makes any warranty, expressed or implied, concerning the repair, modification or replacement described in this report. Furthermore, neither the inspector nor his employer shall be liable in any manner for any personal injury or property damage or a loss of any kind arising from or connected with this inspection.</p> <p>Authorized Inspector's Signature _____ Date _____ Commissions _____</p> <p style="text-align: center; font-size: small;">National Board, State, Province and Endorsements</p> <p style="text-align: center;">* Factory Mutual Engineering Association</p>	

First Energy Nuclear Operating, 76 South Main St., Akron, Ohio 44308

Davis-Besse Nuclear Power Station Unit #1, 5501 N. St. Rt. 2, Oak Harbor, Ohio 43449

Commercial Service Date: 11/27/77

FORM NIS-2 / NR-1 / NVR-1 OWNER'S REPORT FOR REPAIRS OR REPLACEMENTS As Required by the Provisions of the ASME Code Section XI							
1. Owner <u>Toledo Edison Company</u> <small>Name</small> <u>300 Madison Ave., Toledo, OH 43652</u> <small>Address</small>				Date <u>10/1/88</u> Sheet 1 of 1			
2. Plant <u>Davis-Besse Nuclear Power Station</u> <small>Name</small> <u>5501 N. SR 2, Oak Harbor, OH 43449</u> <small>Address</small>				Unit <u>#1</u> <u>MWO 1-87-0626-17</u> <small>Repair Organization P.O. No., Job No., etc.</small>			
3. Work Performed By <u>Toledo Edison Company</u> <small>Name</small> <u>5501 N. SR 2, Oak Harbor, OH 43449</u> <small>Address</small>				Type Code Stamp <u>NR</u> Authorization NO. <u>20</u> Expiration Date <u>7-17-01</u>			
4. Identification of Systems <u>Decay Heat Pump 1-2 Discharge Line 33B-GCB-H38 (Subsys 049-02)</u>							
5. (a) Applicable Construction Code <u>ANSI B31.1</u> Edition 19 <u>67</u> Addendum <u>N/A</u> Code Case <u>N/A</u> (b) Applicable Edition of Section XI Utilized for Repairs or Replacements <u>19 86</u> (c) Design Responsibilities <u>ITT Grinnell</u>							
6. Identification of Components Repaired or Replaced and Replacement Components							
Name of Component	Name of Manufacturer	Manufacturer's Serial NO.	National Board NO.	Other Identification/Jurisdictional NO.	Year Built	Repaired Replaced or Replacement	ASME Code Stamped
Snubber	ITT Grinnell	14501	N/A	A-81	N/A	Replaced	No
Snubber	ITT Grinnell	17656	N/A	A-81	N/A	Replacement	No
7. Description of Work							
ITT Grinnell snubber S/N 14501 was removed and replaced with a like for like snubber S/N 17656. Replacement snubber S/N 17656 was rebuilt under MWO 1-87-0516-11 and was functionally tested satisfactory prior to installation.							
8. Tests Conducted: Hydrostatic (NO) Pneumatic (NO) Nominal Operating Pressure (NO) Other (NO) Pressure <u>N/A</u> psi Test Temperature <u>N/A</u> °F Pressure Relief Valves: Service <u>N/A</u> Size <u>N/A</u> Opening Pressure <u>N/A</u> Blowdown (if applicable) <u>N/A</u> Set Pressure and Blowdown Adjustments Made Using <u>N/A</u> at <u>N/A</u>							
<small>NOTE: Supplemental sheets in the form of lists, sketches, or drawings may be used, provided (1) size is 8 1/2 x 11 in. (2) Information in items 1 through 6 on this report is included on each sheet, and (3) each sheet is numbered and the number of sheets is recorded at the top of this form.</small>							

First Energy Nuclear Operating, 76 South Main St., Akron, Ohio 44308

Davis-Besse Nuclear Power Station Unit #1, 5501 N. St. Rt. 2, Oak Harbor, Ohio 43449

Commercial Service Date: 11/27/77

FORM NIS-2 /NR-1 /NVR-4	
8. REMARKS	Applicable Manufacturers Data Reports to be Attached
N/A	
<p style="text-align: center;">CERTIFICATE OF COMPLIANCE</p> <p>We certify that the statements made in the report are correct and this <u>replacement</u> conforms to the rules of the ASME Code Section XI.</p> <p>Type Code Symbol <u>NR</u> Certificate of Authorization No. <u>20</u> Expiration Date <u>7-17-01</u></p> <p>Signed <u>W. H. Starn, Jr.</u> Nuclear Safety & Inspection Manager Date <u>10/1</u> 19 <u>98</u></p> <p style="text-align: center; font-size: small;">Owner or Owner's Designee, Title</p>	
<p style="text-align: center;">CERTIFICATE OF INSERVICE INSPECTION</p> <p>I, the undersigned, holding a valid commission issued by the National Board of Boiler and Pressure Vessel Inspectors in the State or Province of <u>Ohio</u> and employed by <u>Protection Mutual *</u> of <u>Park Ridge, Illinois</u> have inspected the components described in this Owner's Report during the period <u>11/18/97</u> to <u>10/6/98</u>, and state that to the best of my knowledge and belief, the Owner has performed examinations and taken corrective measures described in this Owner's Report in accordance with the requirements of the ASME Code, Section XI.</p> <p>By signing this certificate, neither the Inspector nor his employer makes any warranty, expressed or implied, concerning the examinations and corrective measures described in the Owner's Report. Furthermore, neither the Inspector nor his employer shall be liable in any manner for any personal injury or property damage or a loss of any kind arising from or connected with this inspection.</p> <p>Inspector's Signature <u>Thomas E. Lopez</u> Date <u>10/6/98</u> Commissions <u>NB9330</u> "N I A" <u>OHIO COMM.</u></p> <p style="text-align: center; font-size: small;">National Board, State, Province and Endorsements</p>	
<p style="text-align: center;">CERTIFICATE OF COMPLIANCE</p> <p>We certify that the statements made in this report are correct and the repair, modification, or replacement of the pressure relief device described above conforms to Section XI and Section III of the ASME Code and the National Board rules as defined in publications NB-65 and NB-102, current editions.</p> <p>Certificate of Authorization No. <u>316</u> to use the "VR" stamp expires <u>Dec. 13</u>, 19 <u>98</u>.</p> <p>Certificate of Authorization No. <u>20</u> to use the "NR" stamp expires <u>July 17</u>, 2001</p> <p>Date _____, 19____, Signed <u>Toledo Edison</u> Nuclear Safety & Inspection Manager</p> <p style="text-align: center; font-size: small;">Repair Organization Authorized Representative Title</p>	
<p style="text-align: center;">CERTIFICATE OF INSPECTION</p> <p>I, the undersigned, holding a valid commission issued by the National Board of Boiler and Pressure Vessel Inspectors, and certificate of competency issued by the State or Province of <u>Ohio</u> and employed by <u>Protection Mutual *</u> of <u>Park Ridge, Illinois</u> have inspected the repair, modification, or replacement described in this report on _____, 19____ and state that to the best of my knowledge and belief, this repair, modification, or replacement has been made or constructed in accordance with Section XI and Section III of the ASME Code and the National Board rules as defined in the publication NB-65 and NB-102, current editions.</p> <p>By signing this certificate, neither the Inspector nor his employer makes any warranty, expressed or implied, concerning the repair, modification or replacement described in this report. Furthermore, neither the Inspector nor his employer shall be liable in any manner for any personal injury or property damage or a loss of any kind arising from or connected with this inspection.</p> <p>Authorized Inspector's Signature _____ Date _____ Commissions _____</p> <p style="text-align: center; font-size: small;">National Board, State, Province and Endorsements</p> <p style="text-align: right;">* Factory Mutual Engineering Association</p>	

First Energy Nuclear Operating, 76 South Main St., Akron, Ohio 44308

Davis-Besse Nuclear Power Station Unit #1, 5501 N. St. Rt. 2, Oak Harbor, Ohio 43449

Commercial Service Date: 11/27/77

FORM NIS-2 / NR-1 / NVR-1 OWNER'S REPORT FOR REPAIRS OR REPLACEMENTS
 As Required by the Provisions of the ASME Code Section XI

1. Owner Toledo Edison Company Date 8/26/98
300 Madison Ave., Toledo, OH 43652 Sheet 1 of 1

2. Plant Davis-Besse Nuclear Power Station Unit #1
5501 N. SR 2, Oak Harbor, OH 43449 MWO# 1-97-0626-18
 Repair Organization P.O. No., Job No., etc.

3. Work Performed By Toledo Edison Company Type Code Stamp NR
5501 N. SR 2, Oak Harbor, OH 43449 Authorization NO. 20
 Expiration Date 7-17-01

4. Identification of Systems DH PUMP 1-1 SUCTION LINE 33A-GCB-8-H4 (A-113)

5. (a) Applicable Construction Code ANSI B31.1 Edition 19 67 Addendum N/A Code Case N/A
 (b) Applicable Edition of Section XI Utilized for Repairs or Replacements 19 86
 (c) Design Responsibilities ITT GRINNEL

6. Identification of Components Repaired or Replaced and Replacement Components

Name of Component	Name of Manufacturer	Manufacturer's Serial NO.	National Board NO.	Other Identification/Jurisdictional NO.	Year Built	Repaired Replaced or Replacement	ASME Code Stamped
Snubber	ITT Grinnel	8906	n/a	A-113	n/a	Replaced	No
Snubber	ITT Grinnel	10151	n/a	A-113	n/a	Replacement	No

7. Description of Work

ITT Grinnel snubber S/N 8906 was removed and replaced with a like for like snubber S/N 10151.

Replacement snubber was rebuilt under MWO# 1-97-0515-15 and functionally tested satisfactory prior to installation.

8. Tests Conducted: Hydrostatic (NO) Pneumatic (NO) Nominal Operating Pressure (NO) Other (NO)
 Pressure n/a psi Test Temperature n/a °F

Pressure Relief Valves:
 Service n/a Size n/a
 Opening Pressure n/a Blowdown (if applicable) n/a
 Set Pressure and Blowdown Adjustments Made Using n/a
at n/a (See Section 1)

NOTE: Supplemental sheets in the form of lists, sketches, or drawings may be used, provided (1) size is 8 1/2 x 11 in. (2) information in items 1 through 6 on this report is included on each sheet, and (3) each sheet is numbered and the number of sheets is recorded at the top of this form.

First Energy Nuclear Operating, 76 South Main St., Akron, Ohio 44308

Davis-Besse Nuclear Power Station Unit #1, 5501 N. St. Rt. 2, Oak Harbor, Ohio 43449

Commercial Service Date: 11/27/77

FORM NIS-2 /NR-1 /NVR-4	
8. REMARKS	Applicable Manufacturers Data Reports to be Attached
N/A	
<p style="text-align: center;">CERTIFICATE OF COMPLIANCE</p> <p>We certify that the statements made in the report are correct and this <u>replacement</u> conforms to the rules of the ASME Code Section XI.</p> <p>Type Code Symbol <u>NB</u> Certificate of Authorization No. <u>20</u> Expiration Date <u>7-17-01</u></p> <p>Signed <u>J.W. Starnes</u> Nuclear Safety & Inspection Manager Date <u>8/26</u> 19<u>98</u></p> <p style="text-align: center; font-size: x-small;">Owner or Owner's Designee, Title</p>	
<p style="text-align: center;">CERTIFICATE OF INSERVICE INSPECTION</p> <p>I, the undersigned, holding a valid commission issued by the National Board of Boiler and Pressure Vessel Inspectors in the State or Province of <u>Ohio</u> and employed by <u>Protection Mutual</u> of <u>Park Ridge, Illinois</u>, have inspected the components described in this Owner's Report during the period <u>11/27/97</u> to <u>8/27/98</u>, and state that to the best of my knowledge and belief, the Owner has performed examinations and taken corrective measures described in this Owner's Report in accordance with the requirements of the ASME Code, Section XI.</p> <p>By signing this certificate, neither the Inspector nor his employer makes any warranty, expressed or implied, concerning the examinations and corrective measures described in the Owner's Report. Furthermore, neither the Inspector nor his employer shall be liable in any manner for any personal injury or property damage or a loss of any kind arising from or connected with this inspection.</p> <p>Inspector's Signature <u>Thomas H. Pope</u> Date <u>8/27/98</u> Commissions <u>NB9330</u> <u>N I A</u> <u>OHIO COMM.</u></p> <p style="text-align: center; font-size: x-small;">National Board, State, Province and Endorsements</p>	
<p style="text-align: center;">CERTIFICATE OF COMPLIANCE</p> <p>We certify that the statements made in this report are correct and the repair, modification, or replacement of the pressure relief device described above conforms to Section XI and Section III of the ASME Code and the National Board rules as defined in publications NB-65 and NB-102, current editions.</p> <p>Certificate of Authorization No. <u>316</u> to use the "VR" stamp expires <u>Dec. 13</u> 19<u>99</u>.</p> <p>Certificate of Authorization No. <u>20</u> to use the "NR" stamp expires <u>July 17</u> 2001</p> <p>Date _____, 19____, Signed <u>Toledo Edison</u> Authorized Representative Nuclear Safety & Inspection Manager</p> <p style="text-align: center; font-size: x-small;">Repair Organization Title</p>	
<p style="text-align: center;">CERTIFICATE OF INSPECTION</p> <p>I, the undersigned, holding a valid commission issued by the National Board of Boiler and Pressure Vessel Inspectors, and certificate of competency issued by the State or Province of <u>Ohio</u> and employed by <u>Protection Mutual</u> of <u>Park Ridge, Illinois</u> have inspected the repair, modification, or replacement described in this report on _____, 19____ and state that to the best of my knowledge and belief, this repair, modification, or replacement has been made or constructed in accordance with Section XI and Section III of the ASME Code and the National Board rules as defined in the publication NB-65 and NB-102, current editions.</p> <p>By signing this certificate, neither the Inspector nor his employer makes any warranty, expressed or implied, concerning the repair, modification or replacement described in this report. Furthermore, neither the Inspector nor his employer shall be liable in any manner for any personal injury or property damage or a loss of any kind arising from or connected with this inspection.</p> <p>Authorized Inspector's Signature _____ Date _____ Commissions _____</p> <p style="text-align: center; font-size: x-small;">National Board, State, Province and Endorsements</p> <p style="text-align: center;">* Factory Mutual Engineering Association</p>	

First Energy Nuclear Operating, 76 South Main St., Akron, Ohio 44308

Davis-Besse Nuclear Power Station Unit #1, 5501 N. St. Rt. 2, Oak Harbor, Ohio 43449

Commercial Service Date: 11/27/77

FORM NIS-2 / NR-1 / NVR-1 OWNER'S REPORT FOR REPAIRS OR REPLACEMENTS

As Required by the Provisions of the ASME Code Section XI

1. Owner <u>Toledo Edison Company</u> <small>Name</small> <u>300 Madison Ave., Toledo, OH 43652</u> <small>Address</small> 2. Plant <u>Davis-Besse Nuclear Power Station</u> <small>Name</small> <u>5501 N. SR 2, Oak Harbor, OH 43449</u> <small>Address</small> 3. Work Performed By <u>Toledo Edison Company</u> <small>Name</small> <u>5501 N. SR 2, Oak Harbor, OH 43449</u> <small>Address</small>	Date <u>3/8/99</u> Sheet 1 of 1 Unit <u>#1</u> MWO# <u>1-98-0453-00</u> <small>Repair Organization P.O. No., Job No., etc.</small> Type Code Stamp <u>NR</u> Authorization NO. <u>20</u> Expiration Date <u>7-17-01</u>
4. Identification of Systems <u>Pipe Hanger PS-H36 (RC-2) SUS 054-04</u>	
5. (a) Applicable Construction Code <u>ANSI B31.1</u> Edition 19 <u>67</u> Addendum <u>n/a</u> Code Case <u>n/a</u> (b) Applicable Edition of Section XI Utilized for Repairs or Replacements 19 <u>86</u> (c) Design Responsibilities <u>Toledo Edison</u>	

6. Identification of Components Repaired or Replaced and Replacement Components							
Name of Component	Name of Manufacturer	Manufacturer's Serial NO.	National Board NO.	Other Identification/ Jurisdictional NO.	Year Built	Repaired or Replacement	ASME Code Stamped
1/2"x 3" A36 plate	UNK	N/A	N/A	HT# 302244	N/A	REPLACEMENT	N/A

7. Description of Work

Installed new yoke on valve RC-2 which required modification of pipe support PS-H36 per VDWG M-190-HBW-135. The old pipe support was removed and modified per DCN # M-190-HBW-135-4 and rewelded using two pcs. of 1/2"x3"x4" A36 plate. Snubbers were reinstalled under MWO# 3-88-4398-01 and a VT-S performed satisfactory.

8. Tests Conducted:	Hydrostatic (NO) Pressure <u>n/a</u> psi	Pneumatic (NO) Test Temperature <u>n/a</u> °F	Nominal Operating Pressure (NO) Other (NO)
Pressure Relief Valves:			
Service <u>n/a</u>		Size <u>n/a</u>	
Opening Pressure <u>n/a</u>		Blowdown (if applicable) <u>n/a</u>	
Set Pressure and Blowdown Adjustments Made Using <u>n/a</u> at <u>n/a</u>			

NOTE: Supplemental sheets in the form of lists, sketches, or drawings may be used, provided (1) size is 8 1/2 x 11 in. (2) Information in items 1 through 6 on this report is included on each sheet, and (3) each sheet is numbered and the number of sheets is recorded at the top of this form.

12 RFO Inservice Inspection Summary

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First Energy Nuclear Operating, 76 South Main St., Akron, Ohio 44308

Davis-Besse Nuclear Power Station Unit #1, 5501 N. St. Rt. 2, Oak Harbor, Ohio 43449

Commercial Service Date: 11/27/77

FORM NIS-2 /NR-1 /NVR-1	
8. REMARKS <div style="border: 1px solid black; height: 40px; margin-top: 5px;"></div>	<small>Applicable Manufacturer Data Reports to be Attached</small> <div style="border: 1px solid black; height: 40px; margin-top: 5px;"></div>
<p style="text-align: center;">CERTIFICATE OF COMPLIANCE</p> <p>We certify that the statements made in the report are correct and this <u>replacement</u> conforms to the rules of the ASME Code Section XI.</p> <p>Type Code Symbol <u>NR</u> Certificate of Authorization No. <u>20</u> Expiration Date <u>7-17-01</u></p> <p>Signed <u>J. W. Starnes, Jr.</u> Nuclear Safety & Inspection Manager Date <u>3/8</u> 19 <u>97</u></p> <p style="text-align: center; font-size: small;">Owner or Owner's Designee, Title</p>	
<p style="text-align: center;">CERTIFICATE OF INSERVICE INSPECTION</p> <p>I, the undersigned, holding a valid commission issued by the National Board of Boiler and Pressure Vessel Inspectors in the State or Province of <u>Ohio</u> and employed by <u>Protection Mutual</u> of <u>Park Ridge, Illinois</u> have inspected the components described in this Owner's Report during the period <u>5/2/96</u> to <u>3/12/99</u>, and state that to the best of my knowledge and belief, the Owner has performed examinations and taken corrective measures described in this Owner's Report in accordance with the requirements of the ASME Code, Section XI.</p> <p>By signing this certificate, neither the Inspector nor his employer makes any warranty, expressed or implied, concerning the examinations and corrective measures described in the Owner's Report. Furthermore, neither the Inspector nor his employer shall be liable in any manner for any personal injury or property damage or a loss of any kind arising from or connected with this inspection.</p> <p>Inspector's Signature <u>Thomas H. Page</u> Date <u>3/12/99</u> Commissions <u>NB 4330 "N I A"</u> <u>OHIO</u></p> <p style="text-align: center; font-size: small;">National Board, State, Province and Endorsements</p>	
<p style="text-align: center;">CERTIFICATE OF COMPLIANCE</p> <p>We certify that the statements made in this report are correct and the repair, modification, or replacement of the pressure relief device described above conforms to Section XI and Section III of the ASME Code and the National Board rules as defined in publications NB-65 and NB-102, current editions.</p> <p>Certificate of Authorization No. <u>316</u> to use the "VR" stamp expires <u>Dec. 13</u>, <u>2001</u></p> <p>Certificate of Authorization No. <u>20</u> to use the "NR" stamp expires <u>July 17</u>, <u>2001</u></p> <p>Date _____, 19____, Signed <u>Toledo Edison</u> Nuclear Safety & Inspection Manager</p> <p style="text-align: center; font-size: small;">Repair Organization Authorized Representative Title</p>	
<p style="text-align: center;">CERTIFICATE OF INSPECTION</p> <p>I, the undersigned, holding a valid commission issued by the National Board of Boiler and Pressure Vessel Inspectors, and certificate of competency issued by the State or Province of <u>Ohio</u> and employed by <u>Protection Mutual</u> of <u>Park Ridge, Illinois</u> have inspected the repair, modification, or replacement described in this report on _____, 19____ and state that to the best of my knowledge and belief, this repair, modification, or replacement has been made or constructed in accordance with Section XI and Section III of the ASME Code and the National Board rules as defined in the publication NB-65 and NB-102, current editions.</p> <p>By signing this certificate, neither the Inspector nor his employer makes any warranty, expressed or implied, concerning the repair, modification or replacement described in this report. Furthermore, neither the Inspector nor his employer shall be liable in any manner for any personal injury or property damage or a loss of any kind arising from or connected with this inspection.</p> <p>Authorized Inspector's Signature _____ Date _____ Commissions _____</p> <p style="text-align: center; font-size: small;">National Board, State, Province and Endorsements</p> <p style="text-align: center;">* Factory Mutual Engineering Association</p>	

First Energy Nuclear Operating, 76 South Main St., Akron, Ohio 44308

Davis-Besse Nuclear Power Station Unit #1, 5501 N. St. Rt. 2, Oak Harbor, Ohio 43449

Commercial Service Date: 11/27/77

FORM NIS-2 / NR-1 / NVR-4 OWNER'S REPORT FOR REPAIRS OR REPLACEMENTS
 As Required by the Provisions of the ASME Code Section XI

1. Owner Toledo Edison Company Date 3/8/99
300 Madison Ave., Toledo, OH 43652 Sheet 1 of 1
 Address

2. Plant Davis-Besse Nuclear Power Station Unit # 1
5501 N. St. Rt. 2, Oak Harbor, OH 43449 MWO # 1-98-0558-00
 Address Repair Organization P.O. No., Job No., etc.

3. Work Performed By Toledo Edison Company Type Code Stamp NR
5501 N. St. Rt. 2, Oak Harbor, OH 43449 Authorization NO. 20
 Address Expiration Date 7-17-01

4. Identification of Systems Pressurizer Spray Valve (RC-2) SUS 054-04

5. (a) Applicable Construction Code ASME III CL1 Edition 19 71 Addendum n/a Code Case n/a
 (b) Applicable Edition of Section XI Utilized for Repairs or Replacements 19 86
 (c) Design Responsibilities Velan Inc.

6. Identification of Components Repaired or Replaced and Replacement Components

Name of Component	Name of Manufacturer	Manufacturer's Serial NO.	National Board NO.	Other Identification/Jurisdictional NO.	Year Built	Repaired Replaced or Replacement	ASME Code Stamped
Nuts 5/8"-11	Nova	n/a	n/a	HT# 43038	n/a	Replacement	No

7. Description of Work

Two nuts were missing from the body to bonnet flange for RC-2. Replaced with two new 5/8"-11, SA-194 Grade 8H, HT#43038 nuts. Performed a VT-2 upon completion of assembly.

8. Tests Conducted: Hydrostatic (NO) Pneumatic (NO) Nominal Operating Pressure (YES) Other (NO)
 Pressure 2102 psi Test Temperature 581 °F

Pressure Relief Valves:
 Service n/a Size n/a
 Opening Pressure n/a Blowdown (if applicable) n/a
 Set Pressure and Blowdown Adjustments Made Using n/a
 at n/a (Test Station)

NOTE: Supplemental sheets in the form of lists, sketches, or drawings may be used, provided (1) size is 8 1/2 x 11 in. (2) Information in items 1 through 6 on this report is included on each sheet, and (3) each sheet is numbered and the number of sheets is recorded at the top of this form.

FORM NIS-2 / NR-1 / NVR-1	
8. REMARKS <div style="border: 1px solid black; height: 40px; margin-top: 5px;"></div>	<small>Applicable Manufacturer Data Reports to be Attached</small> <div style="border: 1px solid black; height: 40px; margin-top: 5px;"></div>
<div style="text-align: center; border-top: 1px solid black; border-bottom: 1px solid black;"> CERTIFICATE OF COMPLIANCE </div> <p style="font-size: small;">We certify that the statements made in the report are correct and this <u>replacement</u> conforms to the rules of the ASME Code Section XI.</p> <p style="font-size: small;">Type Code Symbol <u>NR</u> Certificate of Authorization No. <u>20</u> Expiration Date <u>7-17-81</u></p> <p style="font-size: small;">Signed <u>A. W. S. [Signature]</u> Nuclear Safety & Inspection Manager Date <u>3/8</u> 19 <u>77</u></p> <p style="font-size: x-small; text-align: center;">Owner or Owner's Designee, Title</p>	
<div style="text-align: center; border-top: 1px solid black; border-bottom: 1px solid black;"> CERTIFICATE OF INSERVICE INSPECTION </div> <p style="font-size: small;">I, the undersigned, holding a valid commission issued by the National Board of Boiler and Pressure Vessel Inspectors in the State or Province of <u>Ohio</u> and employed by <u>Protection Mutual</u> of <u>Park Ridge, Illinois</u> have inspected the components described in this Owner's Report during the period <u>5/21/76</u> to <u>3/12/79</u>, and state that to the best of my knowledge and belief, the Owner has performed examinations and taken corrective measures described in this Owner's Report in accordance with the requirements of the ASME Code, Section XI.</p> <p style="font-size: x-small;">By signing this certificate, neither the Inspector nor his employer makes any warranty, expressed or implied, concerning the examinations and corrective measures described in the Owner's Report. Furthermore, neither the Inspector nor his employer shall be liable in any manner for any personal injury or property damage or a loss of any kind arising from or connected with this inspection.</p> <div style="display: flex; justify-content: space-between; align-items: flex-end;"> <div style="width: 45%;"> <p style="font-size: small;">Inspector's Signature <u>Thomas [Signature]</u> Date <u>3/12/79</u></p> </div> <div style="width: 50%; text-align: right;"> <p style="font-size: small;">Commissions <u>NB 4330</u> <u>N I A</u> <u>OHIO</u> <u>CONNA.</u></p> <p style="font-size: x-small;">National Board, State, Province and Endorsements</p> </div> </div>	
<div style="text-align: center; border-top: 1px solid black; border-bottom: 1px solid black;"> CERTIFICATE OF COMPLIANCE </div> <p style="font-size: small;">We certify that the statements made in this report are correct and the repair, modification, or replacement of the pressure relief device described above conforms to Section XI and Section III of the ASME Code and the National Board rules as defined in publications NB-65 and NB-102, current editions.</p> <p style="font-size: small;">Certificate of Authorization No. <u>316</u> to use the "VR" stamp expires <u>Dec. 13</u> <u>2001</u></p> <p style="font-size: small;">Certificate of Authorization No. <u>20</u> to use the "NR" stamp expires <u>July 17</u> <u>2001</u></p> <p style="font-size: small;">Date _____, 19____, Signed <u>Toledo Edison</u> Nuclear Safety & Inspection Manager</p> <p style="font-size: x-small; text-align: center;">Repair Organization Authorized Representative Title</p>	
<div style="text-align: center; border-top: 1px solid black; border-bottom: 1px solid black;"> CERTIFICATE OF INSPECTION </div> <p style="font-size: small;">I, the undersigned, holding a valid commission issued by the National Board of Boiler and Pressure Vessel Inspectors, and certificate of competency issued by the State or Province of <u>Ohio</u> and employed by <u>Protection Mutual</u> of <u>Park Ridge, Illinois</u> have inspected the repair, modification, or replacement described in this report on _____, 19____ and state that to the best of my knowledge and belief, this repair, modification, or replacement has been made or constructed in accordance with Section XI and Section III of the ASME Code and the National Board rules as defined in the publication NB-65 and NB-102, current editions.</p> <p style="font-size: x-small;">By signing this certificate, neither the Inspector nor his employer makes any warranty, expressed or implied, concerning the repair, modification or replacement described in this report. Furthermore, neither the Inspector nor his employer shall be liable in any manner for any personal injury or property damage or a loss of any kind arising from or connected with this inspection.</p> <div style="display: flex; justify-content: space-between; align-items: flex-end;"> <div style="width: 45%;"> <p style="font-size: small;">Authorized Inspector's Signature _____ Date _____</p> </div> <div style="width: 50%; text-align: right;"> <p style="font-size: small;">Commissions _____</p> <p style="font-size: x-small;">National Board, State, Province and Endorsements</p> <p style="font-size: small;">* Factory Mutual Engineering Association</p> </div> </div>	

First Energy Nuclear Operating, 76 South Main St., Akron, Ohio 44308

Davis-Besse Nuclear Power Station Unit #1, 5501 N. St. Rt. 2, Oak Harbor, Ohio 43449

Commercial Service Date: 11/27/77

FORM NIS-2 / NR-1 / NYR-1 OWNER'S REPORT FOR REPAIRS OR REPLACEMENTS

As Required by the Provisions of the ASME Code Section XI

1. Owner Toledo Edison Company Date 2/12/99
300 Madison Ave., Toledo, OH 43652 Sheet 1 of 1
 2. Plant Davis-Besse Nuclear Power Station Unk # 1
5501 N. SR 2, Oak Harbor, OH 43449 MWO# 1-98-1155-00
 3. Work Performed By Toledo Edison Company Type Code Stamp NR
5501 N. SR 2, Oak Harbor, OH 43449 Authorization NO. 20
 Expiration Date 7-17-01

4. Identification of Systems Pressurizer Spray Valve (RC-2)

5. (a) Applicable Construction Code ASME NCL1 Edition 19 71 Addendum n/a Code Case n/a
 (b) Applicable Edition of Section XI Utilized for Repairs or Replacements 19 86
 (c) Design Responsibilities Velan Inc.

6. Identification of Components Repaired or Replaced and Replacement Components							
Name of Component	Name of Manufacturer	Manufacturer's Serial NO.	National Board NO.	Other Identification/ Jurisdictional NO.	Year Built	Repaired Replaced or Replacement	ASME Code Stamped
Nut 5/8"-11	Velan	n/a	n/a	n/a	unk	Replaced	No
Nut 5/8"- 11	Velan	n/a	n/a	Ht# A17623/GHD	1990	Replacement	No

7. Description of Work
 Replaced eight 5/8"-11 UNC nuts SA194 Gr 8M, heat # A17623/GHD on bonnet to body for valve RC-2. Performed a VT-2 leakage test upon completion of assembly.

8. Tests Conducted: Hydrostatic (NO) Pneumatic (NO) Nominal Operating Pressure (YES) Other (NO)
 Pressure 2148 psi Test Temperature 534 °F
 Pressure Relief Valves:
 Service n/a Size n/a
 Opening Pressure n/a Blowdown (if applicable) n/a
 Set Pressure and Blowdown Adjustments Made Using n/a
 at n/a

NOTE: Supplemental sheets in the form of lists, sketches, or drawings may be used, provided (1) size is 8 1/2 x 11 in. (2) Information in items 1 through 6 on this report is included on each sheet, and (3) each sheet is numbered and the number of sheets is recorded at the top of this form.

First Energy Nuclear Operating, 76 South Main St., Akron, Ohio 44308

Davis-Besse Nuclear Power Station Unit #1, 5501 N. St. Rt. 2, Oak Harbor, Ohio 43449

Commercial Service Date: 11/27/77

FORM NIS-2 / NR-1 / NVR-1	
9. REMARKS	Applicable Manufacturer Data Reports to be Attached
<div style="border: 1px solid black; padding: 5px; margin-bottom: 10px;"> <p style="text-align: center; margin: 0;">CERTIFICATE OF COMPLIANCE</p> <p>We certify that the statements made in the report are correct and this <u>replacement</u> conforms to the rules of the ASME Code Section XI.</p> <p>Type Code Symbol <u>NR</u> Certificate of Authorization No. <u>20</u> Expiration Date <u>7-17-01</u></p> <p>Signed <u>[Signature]</u> Nuclear Safety & Inspection Manager Date <u>2/13</u> 19 <u>99</u></p> <p style="font-size: small; text-align: center;">Officer or Owner's Designee, Title</p> </div> <div style="border: 1px solid black; padding: 5px; margin-bottom: 10px;"> <p style="text-align: center; margin: 0;">CERTIFICATE OF INSERVICE INSPECTION</p> <p>I, the undersigned, holding a valid commission issued by the National Board of Boiler and Pressure Vessel Inspectors in the State or Province of <u>Ohio</u> and employed by <u>Protection Mutual</u> of <u>Park Ridge, Illinois</u> have inspected the components described in this Owner's Report during the period <u>10/16/98</u> to <u>2/25/99</u>, and state that to the best of my knowledge and belief, the Owner has performed examinations and taken corrective measures described in this Owner's Report in accordance with the requirements of the ASME Code, Section XI.</p> <p>By signing this certificate, neither the Inspector nor his employer makes any warranty, expressed or implied, concerning the examinations and corrective measures described in the Owner's Report. Furthermore, neither the Inspector nor his employer shall be liable in any manner for any personal injury or property damage or a loss of any kind arising from or connected with this inspection.</p> <p>Inspector's Signature <u>Thomas J. Lipe</u> Date <u>2/25/99</u> Commissions <u>NB9330</u> <u>N E A</u> <u>OHIO</u> <small>Inspector's Signature Date National Board, State, Province and Endorsements Comm.</small></p> </div> <div style="border: 1px solid black; padding: 5px; margin-bottom: 10px;"> <p style="text-align: center; margin: 0;">CERTIFICATE OF COMPLIANCE</p> <p>We certify that the statements made in this report are correct and the repair, modification, or replacement of the pressure relief device described above conforms to Section XI and Section III of the ASME Code and the National Board rules as defined in publications NB-65 and NB-102, current editions.</p> <p>Certificate of Authorization No. <u>316</u> to use the "VR" stamp expires <u>Dec. 19</u> <u>2001</u></p> <p>Certificate of Authorization No. <u>20</u> to use the "NR" stamp expires <u>July 17</u> <u>2001</u></p> <p>Date _____, 19____, Signed <u>Toledo Edison</u> Nuclear Safety & Inspection Manager <small>Repair Organization Authorized Representative Title</small></p> </div> <div style="border: 1px solid black; padding: 5px;"> <p style="text-align: center; margin: 0;">CERTIFICATE OF INSPECTION</p> <p>I, the undersigned, holding a valid commission issued by the National Board of Boiler and Pressure Vessel Inspectors, and certificate of competency issued by the State or Province of <u>Ohio</u> and employed by <u>Protection Mutual</u> of <u>Park Ridge, Illinois</u> have inspected the repair, modification, or replacement described in this report on _____, 19____ and state that to the best of my knowledge and belief, this repair, modification, or replacement has been made or constructed in accordance with Section XI and Section III of the ASME Code and the National Board rules as defined in the publication NB-65 and NB-102, current editions.</p> <p>By signing this certificate, neither the Inspector nor his employer makes any warranty, expressed or implied, concerning the repair, modification or replacement described in this report. Furthermore, neither the Inspector nor his employer shall be liable in any manner for any personal injury or property damage or a loss of any kind arising from or connected with this inspection.</p> <p>Authorized Inspector's Signature _____ Date _____ Commissions _____ <small>Authorized Inspector's Signature Date National Board, State, Province and Endorsements</small></p> <p style="text-align: right; font-size: small;">* Factory Mutual Engineering Association</p> </div>	

First Energy Nuclear Operating, 76 South Main St., Akron, Ohio 44308

Davis-Besse Nuclear Power Station Unit #1, 5501 N. St. Rt. 2, Oak Harbor, Ohio 43449

Commercial Service Date: 11/27/77

FORM NIS-2 / NR-1 / NVR-1 OWNER'S REPORT FOR REPAIRS OR REPLACEMENTS <small>As Required by the Provisions of the ASME Code Section XI</small>							
1. Owner <u>Toledo Edison Company</u> <small>Name</small> <u>300 Madison Ave., Toledo, OH 43652</u> <small>Address</small>				Date <u>5/17/89</u> Sheet 1 of 1			
2. Plant <u>Davis-Besse Nuclear Power Station</u> <small>Name</small> <u>5501 N. St. Rt. 2, Oak Harbor, OH 43449</u> <small>Address</small>				Unit <u># 1</u> W.O. 98-039-00, 98-423-00 & -01 <small>Repair Organization P.O. No., Job No., etc.</small>			
3. Work Performed By <u>Toledo Edison Company</u> <small>Name</small> <u>5501 N. St. Rt. 2, Oak Harbor, OH 43449</u> <small>Address</small>				Type Code Stamp <u>NR / VR</u> Authorization NO. <u>20 / 316</u> Expiration Date <u>7-17-01 / 12-13-01</u>			
4. Identification of Systems <u>RC2A PRESSURIZER POWER RELIEF VALVE</u>							
5. (a) Applicable Construction Code <u>ASME MCCLJ</u> Edition 19 <u>Z1</u> Addendum <u>W71</u> Code Case <u>1501</u> (b) Applicable Edition of Section XI Utilized for Repairs or Replacements 19 <u>86</u> (c) Design Responsibilities <u>CROSBY VALVE & GAGE CO.</u>							
6. Identification of Components Repaired or Replaced and Replacement Components							
Name of Component	Name of Manufacturer	Manufacturer's Serial NO.	National Board NO.	Other Identification/Jurisdictional NO.	Year Built	Repaired or Replacement	ASME Code Stamped
RELIEF VLV	CROSBY VALVE	N56969-00-0001	NA	NA	1977	REPLACED	Yes
RELIEF VLV	CROSBY VALVE	N56969-00-0002	NA	NA	1977	REPLACEMENT	Yes
NOZZLE MN.	CROSBY VALVE	N95208-32-0004	NA	NA	NA	REPLACEMENT	No
DISC MN.	CROSBY VALVE	N95209-34-0005	NA	NA	NA	REPLACEMENT	No
NOZZLE PILOT	CROSBY VALVE	N9093200380015	NA	NA	NA	REPLACEMENT	No
DISC PILOT	CROSBY VALVE	N90931-33-0005	NA	NA	NA	REPLACEMENT	No
7. Description of Work Rebuilt spare valve S/N N56969-00-0002 with new nozzles (main valve and pilot valve) and new discs (main valve and pilot valve). Removed existing valve S/N N56969-00-0001 and replaced with like-for-like rebuilt valve S/N N56969-00-0002. Performed a VT-2 leakage test upon completion.							
8. Tests Conducted: Hydrostatic (NO) Pneumatic (NO) Nominal Operating Pressure (YES) Other (NO) Pressure <u>2128</u> psi Test Temperature <u>536</u> °F Pressure Relief Valves: Service <u>steam</u> Size <u>2 1/2 x 4"</u> Opening Pressure <u>2435 psig</u> Blowdown (if applicable) <u>na</u> Set Pressure and Blowdown Adjustments Made Using <u>na</u> at <u>na</u>							
<small>NOTE: Supplemental sheets in the form of lists, sketches, or drawings may be used, provided (1) size is 8 1/2 x 11 in. (2) Information in items 1 through 6 on this report is included on each sheet, and (3) each sheet is numbered and the number of sheets is recorded at the top of this form.</small>							

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First Energy Nuclear Operating, 76 South Main St., Akron, Ohio 44308

Davis-Besse Nuclear Power Station Unit #1, 5501 N. St. Rt. 2, Oak Harbor, Ohio 43449

Commercial Service Date: 11/27/77

FORM NIS-2 / NR-1 / NVR-1	
8. REMARKS	Applicable Manufacturers Data Reports to be Attached
N/A	
CERTIFICATE OF COMPLIANCE	
<p>We certify that the statements made in the report are correct and this replacement conforms to the rules of the ASME Code Section XI.</p> <p>Type Code Symbol <u>NR</u> Certificate of Authorization No. <u>20</u> Expiration Date <u>7-17-01</u></p> <p>Signed <u>D.W. Starnes</u> Nuclear Safety & Inspection Manager Date <u>5/17</u> 19 <u>99</u></p> <p style="text-align: center; font-size: x-small;">Owner or Owner's Designee, Title</p>	
CERTIFICATE OF INSERVICE INSPECTION	
<p>I, the undersigned, holding a valid commission issued by the National Board of Boiler and Pressure Vessel Inspectors in the State or Province of <u>Ohio</u> and employed by <u>Protection Mutual</u> of <u>Park Ridge, Illinois</u> have inspected the components described in this Owner's Report during the period <u>2/24/99</u> to <u>5/20/99</u>, and state that to the best of my knowledge and belief, the Owner has performed examinations and taken corrective measures described in this Owner's Report in accordance with the requirements of the ASME Code, Section XI.</p> <p>By signing this certificate, neither the inspector nor his employer makes any warranty, expressed or implied, concerning the examinations and corrective measures described in the Owner's Report. Furthermore, neither the inspector nor his employer shall be liable in any manner for any personal injury or property damage or a loss of any kind arising from or connected with this inspection.</p> <p>Inspector's Signature <u>Thomas H. Laps</u> Date <u>5/20/99</u> Commissions <u>NB 9330 "N I A"</u> <u>OHIO COMM.</u></p> <p style="text-align: center; font-size: x-small;">National Board, State, Province and Endorsements</p>	
CERTIFICATE OF COMPLIANCE	
<p>We certify that the statements made in this report are correct and the repair, modification, or replacement of the pressure relief device described above conforms to Section XI and Section III of the ASME Code and the National Board rules as defined in publications NB-65 and NB-102, current editions.</p> <p>Certificate of Authorization No. <u>316</u> to use the "VR" stamp expires <u>Dec. 13</u>, 2001</p> <p>Certificate of Authorization No. <u>20</u> to use the "NR" stamp expires <u>July 17</u>, 2001</p> <p>Date <u>5/17</u>, 19 <u>99</u>, Signed <u>Toledo Edison</u> <u>D.W. Starnes</u> Nuclear Safety & Inspection Manager</p> <p style="text-align: center; font-size: x-small;">Repair Organization Authorized Representative Title</p>	
CERTIFICATE OF INSPECTION	
<p>I, the undersigned, holding a valid commission issued by the National Board of Boiler and Pressure Vessel Inspectors, and certificate of competency issued by the State or Province of <u>Ohio</u> and employed by <u>Protection Mutual</u> of <u>Park Ridge, Illinois</u> have inspected the repair, modification, or replacement described in this report on <u>MAY 20</u>, 19 <u>99</u> and state that to the best of my knowledge and belief, this repair, modification, or replacement has been made or constructed in accordance with Section XI and Section III of the ASME Code and the National Board rules as defined in the publication NB-65 and NB-102, current editions.</p> <p>By signing this certificate, neither the inspector nor his employer makes any warranty, expressed or implied, concerning the repair, modification or replacement described in this report. Furthermore, neither the inspector nor his employer shall be liable in any manner for any personal injury or property damage or a loss of any kind arising from or connected with this inspection.</p> <p>Authorized Inspector's Signature <u>Thomas H. Laps</u> Date <u>5/20/99</u> Commissions <u>NB 9330 "N I A"</u> <u>OHIO COMM.</u></p> <p style="text-align: center; font-size: x-small;">National Board, State, Province and Endorsements</p> <p style="text-align: center;">* Factory Mutual Engineering Association</p>	

* FACTORY MUTUAL INS. CO.

First Energy Nuclear Operating, 76 South Main St., Akron, Ohio 44308

Davis-Besse Nuclear Power Station Unit #1, 5501 N. St. Rt. 2, Oak Harbor, Ohio 43449

Commercial Service Date: 11/27/77

FORM NIS-2 / NR-1 / NVR-4 OWNER'S REPORT FOR REPAIRS OR REPLACEMENTS As Required by the Provisions of the ASME Code Section XI

1. Owner Toledo Edison Company Date 4/6/99
Name
300 Madison Ave., Toledo, OH 43652
Address
 Sheet 1 of 1

2. Plant Davis-Besse Nuclear Power Station Unit #1
Name
5501 N. SR 2, Oak Harbor, OH 43449
Address
WO# 98-000135-000
Repair Organization P.O. No., Job No., etc.

3. Work Performed By Toledo Edison Company Type Code Stamp NR
Name
5501 N. SR 2, Oak Harbor, OH 43449
Address
 Authorization NO. 20
 Expiration Date 7-17-01

4. Identification of Systems MAIN STEAM FORM STEAM GENERATOR 1-1 EBB1SR11 EAST (A-41)

5. (a) Applicable Construction Code ANSI B31.1 Edition 19 67 Addendum N/A Code Case N/A
 (b) Applicable Edition of Section XI Utilized for Repairs or Replacements 19 86
 (c) Design Responsibilities ITT GRINNELL

6. Identification of Components Repaired or Replaced and Replacement Components

Name of Component	Name of Manufacturer	Manufacturer's Serial NO.	National Board NO.	Other Identification/Jurisdictional NO.	Year Built	Repaired or Replacement	ASME Code Stamped
SNUBBER	ITT GRINNELL	11892	N/A	A-41	N/A	REPLACED	No
SNUBBER	ITT GRINNELL	12449	N/A	A-41	N/A	REPLACEMENT	No

7. Description of Work

ITT GRINNELL SNUBBER S/N 11892 WAS REMOVED AND REPLACED WITH LIKE-FOR-LIKE SNUBBER S/N 12449. REPLACEMENT
 SNUBBER S/N 12449 WAS REBUILT UNDER WO# 98-000740-000 AND WAS FUNCTIONALLY TESTED SATISFACTORY. A VT-3 WAS PERFORMED
 SATISFACTORY AFTER INSTALLATION.

8. Tests Conducted: Hydrostatic (NO) Pneumatic (NO) Nominal Operating Pressure (NO) Other (NO)
 Pressure N/A psi Test Temperature N/A °F

Pressure Relief Valves:
 Service N/A Size N/A
 Opening Pressure N/A Blowdown (if applicable) N/A
 Set Pressure and Blowdown Adjustments Made Using N/A
 at N/A (Test Medium)

NOTE: Supplemental sheets in the form of lists, sketches, or drawings may be used, provided (1) size is 8 1/2 x 11 in. (2) Information in items 1 through 6 on this report is included on each sheet, and (3) each sheet is numbered and the number of sheets is recorded at the top of this form.

First Energy Nuclear Operating, 76 South Main St., Akron, Ohio 44308

Davis-Besse Nuclear Power Station Unit #1, 5501 N. St. Rt. 2, Oak Harbor, Ohio 43449

Commercial Service Date: 11/27/77

FORM NIS-2 / NR-1 / NVR-1

8. REMARKS

Applicable Manufacturers Data Reports to be Attached

N/A

CERTIFICATE OF COMPLIANCE

We certify that the statements made in the report are correct and this replacement conforms to the rules of the ASME Code Section XI.

Type Code Symbol NR Certificate of Authorization No. 20 Expiration Date 7-17-01

Signed [Signature] Nuclear Safety & Inspection Manager Date 4/6 19 99

Owner or Owner's Designee, Title

CERTIFICATE OF INSERVICE INSPECTION

I, the undersigned, holding a valid commission issued by the National Board of Boiler and Pressure Vessel Inspectors in the State or Province of Ohio and employed by Protection Mutual * of Park Ridge, Illinois have inspected the components described in this Owner's Report during the period 11-4-92 to 4-6-92, and state that to the best of my knowledge and belief, the Owner has performed examinations and taken corrective measures described in this Owner's Report in accordance with the requirements of the ASME Code, Section XI.

By signing this certificate, neither the Inspector nor his employer makes any warranty, expressed or implied, concerning the examinations and corrective measures described in the Owner's Report. Furthermore, neither the Inspector nor his employer shall be liable in any manner for any personal injury or property damage or a loss of any kind arising from or connected with this inspection.

Inspector's Signature

Date 4-6-99

Commissions NB9801 N.T.A. Ohio Comm
National Board, State, Province and Endorsements

CERTIFICATE OF COMPLIANCE

We certify that the statements made in this report are correct and the repair, modification, or replacement of the pressure relief device described above conforms to Section XI and Section III of the ASME Code and the National Board rules as defined in publications NB-65 and NB-102, current editions.

Certificate of Authorization No. 316 to use the "VR" stamp expires Dec. 13, 2001

Certificate of Authorization No. 20 to use the "NR" stamp expires July 17, 2001

Date _____, 19____, Signed Toledo Edison Nuclear Safety & Inspection Manager
Repair Organization Authorized Representative Title

CERTIFICATE OF INSPECTION

I, the undersigned, holding a valid commission issued by the National Board of Boiler and Pressure Vessel Inspectors, and certificate of competency issued by the State or Province of Ohio and employed by Protection Mutual *

of Park Ridge, Illinois have inspected the repair, modification, or replacement described in this report

on _____, 19____ and state that to the best of my knowledge and belief, this repair, modification, or replacement has been made or constructed in accordance with Section XI and Section III of the ASME Code and the National Board rules as defined in the publication NB-65 and NB-102, current editions.

By signing this certificate, neither the Inspector nor his employer makes any warranty, expressed or implied, concerning the repair, modification or replacement described in this report. Furthermore, neither the Inspector nor his employer shall be liable in any manner for any personal injury or property damage or a loss of any kind arising from or connected with this inspection.

Authorized Inspector's Signature

Date

Commissions _____
National Board, State, Province and Endorsements

* Factory Mutual Engineering Association

First Energy Nuclear Operating, 76 South Main St., Akron, Ohio 44308

Davis-Besse Nuclear Power Station Unit #1, 5501 N. St. Rt. 2, Oak Harbor, Ohio 43449

Commercial Service Date: 11/27/77

FORM NIS-2 / NR-1 / NVR-4 OWNER'S REPORT FOR REPAIRS OR REPLACEMENTS
 As Required by the Provisions of the ASME Code Section XI

1. Owner Toledo Edison Company Date 7/8/99
Name
300 Madison Ave., Toledo, OH 43652
Address
 Sheet 1 of 1

2. Plant Davis-Besse Nuclear Power Station Unit # 1
Name
5501 N. SR 2, Oak Harbor, OH 43449
Address
WO# 98-000139-000
Repair Organization P.O. No., Job No., etc.

3. Work Performed By Toledo Edison Company Type Code Stamp NR
Name
5501 N. SR 2, Oak Harbor, OH 43449
Address
 Authorization NO. 20
 Expiration Date 7-17-01

4. Identification of Systems DH PUMP 1-1 SUCTION LINE 33A-GCB-8-H11 A114

5. (a) Applicable Construction Code ANSI B31.1 Edition 19 67 Addendum N/A Code Case N/A
 (b) Applicable Edition of Section XI Utilized for Repairs or Replacements 19 85
 (c) Design Responsibilities ITT GRINNELL

6. Identification of Components Repaired or Replaced and Replacement Components

Name of Component	Name of Manufacturer	Manufacturer's Serial NO.	National Board NO.	Other Identification/Jurisdictional NO.	Year Built	Repaired Replaced or Replacement	ASME Code Stamped
SNUBBER	ITT GRINNELL	16826	N/A	A114	N/A	REPLACED	No
SNUBBER	ITT GRINNELL	21306	N/A	A114	N/A	REPLACEMENT	No

7. Description of Work

ITT GRINNELL SNUBBER S/N 16826 WAS REMOVED AND REPLACED WITH LIKE-FOR-LIKE SNUBBER S/N 21306. REPLACEMENT SNUBBER S/N 21306 WAS REBUILT UNDER MWO# 1-87-0515-05 AND WAS FUNCTIONALLY TESTED SATISFACTORILY.

8. Tests Conducted: Hydrostatic (NO) Pneumatic (NO) Nominal Operating Pressure (NO) Other (NO)
 Pressure N/A psi Test Temperature N/A °F

Pressure Relief Valves:
 Service N/A Size N/A
 Opening Pressure N/A Blowdown (if applicable) N/A
 Set Pressure and Blowdown Adjustments Made Using N/A
 at N/A (Test Medium)

NOTE: Supplemental sheets in the form of lists, sketches, or drawings may be used, provided (1) size is 8 1/2 x 11 in. (2) Information in items 1 through 6 on this report is included on each sheet, and (3) each sheet is numbered and the number of sheets is recorded at the top of this form.

First Energy Nuclear Operating, 76 South Main St., Akron, Ohio 44308

Davis-Besse Nuclear Power Station Unit #1, 5501 N. St. Rt. 2, Oak Harbor, Ohio 43449

Commercial Service Date: 11/27/77

FORM NIS-2 /NR-1 /NVR-1

B. REMARKS

Applicable Manufacturers Data Reports to be Attached

N/A

CERTIFICATE OF COMPLIANCE

We certify that the statements made in the report are correct and this replacement conforms to the rules of the ASME Code Section XI.

Type Code Symbol NR Certificate of Authorization No. 20 Expiration Date 7-17-01

Signed H. W. Stevens/99 Nuclear Safety & Inspection Manager Date 7/9/- 1999
Owner or Owner's Designee, Title

CERTIFICATE OF INSERVICE INSPECTION

I, the undersigned, holding a valid commission issued by the National Board of Boiler and Pressure Vessel Inspectors in the State or Province of Ohio and employed by Protection Mutual of Park Ridge, Illinois have inspected the components described in this Owner's Report during the period 11/18/97 to 7/15/99, and state that to the best of my knowledge and belief, the Owner has performed examinations and taken corrective measures described in this Owner's Report in accordance with the requirements of the ASME Code, Section XI.

By signing this certificate, neither the Inspector nor his employer makes any warranty, expressed or implied, concerning the examinations and corrective measures described in the Owner's Report. Furthermore, neither the Inspector nor his employer shall be liable in any manner for any personal injury or property damage or a loss of any kind arising from or connected with this inspection.

Thomas A. Lynn 7/15/99
 Inspector's Signature Date

Commissions NB 9330 "N I A"
 National Board, State, Province and Endorsements

"OHIO"
 Comm.

CERTIFICATE OF COMPLIANCE

We certify that the statements made in this report are correct and the repair, modification, or replacement of the pressure relief device described above conforms to Section XI and Section III of the ASME Code and the National Board rules as defined in publications NB-65 and NB-102, current editions.

Certificate of Authorization No. 316 to use the "VR" stamp expires Dec. 13, 2001

Certificate of Authorization No. 20 to use the "NR" stamp expires July 17, 2001

Date _____, 19____, Signed Toledo Edison Nuclear Safety & Inspection Manager
Repair Organization Authorized Representative Title

CERTIFICATE OF INSPECTION

I, the undersigned, holding a valid commission issued by the National Board of Boiler and Pressure Vessel Inspectors, and certificate of competency issued by the State or Province of Ohio and employed by Protection Mutual

of Park Ridge, Illinois have inspected the repair, modification, or replacement described in this report

on _____, 19____ and state that to the best of my knowledge and belief, this repair, modification, or replacement has been made or constructed in accordance with Section XI and Section III of the ASME Code and the National Board rules as defined in the publication NB-65 and NB-102, current editions.

By signing this certificate, neither the Inspector nor his employer makes any warranty, expressed or implied, concerning the repair, modification or replacement described in this report. Furthermore, neither the Inspector nor his employer shall be liable in any manner for any personal injury or property damage or a loss of any kind arising from or connected with this inspection.

Authorized Inspector's Signature _____ Date _____

Commissions _____
 National Board, State, Province and Endorsements

* Factory Mutual Engineering Association

* Factory Mutual Ins. Co.

First Energy Nuclear Operating, 76 South Main St., Akron, Ohio 44308

Davis-Besse Nuclear Power Station Unit #1, 5501 N. St. Rt. 2, Oak Harbor, Ohio 43449

Commercial Service Date: 11/27/77

FORM NIS-2 / NR-1 / NVR-1 OWNER'S REPORT FOR REPAIRS OR REPLACEMENTS
 As Required by the Provisions of the ASME Code Section XI

1. Owner Toledo Edison Company Date 7/7/89
300 Madison Ave., Toledo, OH 43652 Sheet 1 of 1

2. Plant Davis-Besse Nuclear Power Station Unit #1
5501 N. St. Rt. 2, Oak Harbor, OH 43449 WO # 98-000140-000
 Repair Organization P.O. No., Job No., etc.

3. Work Performed By Toledo Edison Company Type Code Stamp NR
5501 N. St. Rt. 2, Oak Harbor, OH 43449 Authorization NO. 20
 Expiration Date 7-17-01

4. Identification of Systems DH REMOVAL COOLER 1-1 TO RCS 33B-GCB-10-H13 A116

5. (a) Applicable Construction Code ANSI B31.1 Edition 19 67 Addendum N/A Code Case N/A
 (b) Applicable Edition of Section XI Utilized for Repairs or Replacements 19 86
 (c) Design Responsibilities ITT GRINNELL

6. Identification of Components Repaired or Replaced and Replacement Components

Name of Component	Name of Manufacturer	Manufacturer's Serial NO.	National Board NO.	Other Identification/Jurisdictional NO.	Year Built	Repaired Replaced or Replacement	ASME Code Stamped
SNUBBER	ITT GRINNELL	10119	N/A	A-116	N/A	REPLACED	No
SNUBBER	ITT GRINNELL	14204	N/A	A-116	N/A	REPLACEMENT	No

7. Description of Work

ITT GRINNELL SNUBBER S/N 10119 WAS REMOVED AND REPLACED WITH LIKE-FOR-LIKE SNUBBER S/N 14204. REPLACEMENT SNUBBER S/N 14204 WAS REBUILT UNDER WO# 98-000740-000 AND WAS FUNCTIONALLY TESTED SATISFACTORILY.

8. Tests Conducted: Hydrostatic (NO) Pneumatic (NO) Nominal Operating Pressure (NO) Other (NO)
 Pressure N/A psi Test Temperature N/A °F

Pressure Relief Valves:
 Service N/A Size N/A
 Opening Pressure N/A Blowdown (if applicable) N/A
 Set Pressure and Blowdown Adjustments Made Using N/A
 at N/A (Test Medium)

NOTE: Supplemental sheets in the form of lists, sketches, or drawings may be used, provided (1) size is 8 1/2 x 11 in. (2) Information in items 1 through 6 on this report is included on each sheet, and (3) each sheet is numbered and the number of sheets is recorded at the top of this form.

First Energy Nuclear Operating, 76 South Main St., Akron, Ohio 44308

Davis-Besse Nuclear Power Station Unit #1, 5501 N. St. Rt. 2, Oak Harbor, Ohio 43449

Commercial Service Date: 11/27/77

FORM NIS-2 / NR-1 / NVR-1

B. REMARKS

Applicable Manufacturers Data Reports to be Attached

N/A

CERTIFICATE OF COMPLIANCE

We certify that the statements made in the report are correct and this replacement conforms to the rules of the ASME Code Section XI.

Type Code Symbol NR Certificate of Authorization No. 20 Expiration Date 7-17-01

Signed H. W. Stevens Nuclear Safety & Inspection Manager Date 7/9 19 99

Owner or Owner's Designee, Title

CERTIFICATE OF INSERVICE INSPECTION

I, the undersigned, holding a valid commission issued by the National Board of Boiler and Pressure Vessel Inspectors in the State or Province of Ohio and employed by Protection Mutual of Park Ridge, Illinois have inspected the components described in this Owner's Report during the period 10/8/97 to 7/20/99, and state that to the best of my knowledge and belief, the Owner has performed examinations and taken corrective measures described in this Owner's Report in accordance with the requirements of the ASME Code, Section XI.

By signing this certificate, neither the Inspector nor his employer makes any warranty, expressed or implied, concerning the examinations and corrective measures described in the Owner's Report. Furthermore, neither the Inspector nor his employer shall be liable in any manner for any personal injury or property damage or a loss of any kind arising from or connected with this inspection.

Thomas H. Lape
Inspector's Signature

7/20/99
Date

Commissions NB9330 "N I A" OHIO
National Board, State, Province and Endorsements COMM.

CERTIFICATE OF COMPLIANCE

We certify that the statements made in this report are correct and the repair, modification, or replacement of the pressure relief device described above conforms to Section XI and Section III of the ASME Code and the National Board rules as defined in publications NB-65 and NB-102, current editions.

Certificate of Authorization No. 316 to use the "VR" stamp expires Dec. 13, 2001

Certificate of Authorization No. 20 to use the "NR" stamp expires July 17, 2001

Date , 19 , Signed Toledo Edison Nuclear Safety & Inspection Manager
Repair Organization Authorized Representative Title

CERTIFICATE OF INSPECTION

I, the undersigned, holding a valid commission issued by the National Board of Boiler and Pressure Vessel Inspectors, and

certificate of competency issued by the State or Province of Ohio and employed by Protection Mutual

of Park Ridge, Illinois have inspected the repair, modification, or replacement described in this report

on , 19 and state that to the best of my knowledge and belief, this repair, modification, or replacement has been made or constructed in accordance with Section XI and Section III of the ASME Code and the National Board rules as defined in the publication NB-65 and NB-102, current editions.

By signing this certificate, neither the Inspector nor his employer makes any warranty, expressed or implied, concerning the repair, modification or replacement described in this report. Furthermore, neither the Inspector nor his employer shall be liable in any manner for any personal injury or property damage or a loss of any kind arising from or connected with this inspection.

Authorized Inspector's Signature

Date

Commissions

National Board, State, Province and Endorsements

* Factory Mutual Engineering Association

* Factory Mutual Ins. Co.

First Energy Nuclear Operating, 76 South Main St., Akron, Ohio 44308

Davis-Besse Nuclear Power Station Unit #1, 5501 N. St. Rt. 2, Oak Harbor, Ohio 43449

Commercial Service Date: 11/27/77

FORM NIS-2 / NR-1 / NVR-1 OWNER'S REPORT FOR REPAIRS OR REPLACEMENTS As Required by the Provisions of the ASME Code Section XI

1. Owner Toledo Edison Company Date 4/13/99
Name
300 Madison Ave., Toledo, OH 43652
Address
 Sheet 1 of 1

2. Plant Davis-Besse Nuclear Power Station Unit #1
Name
5501 N. SR 2, Oak Harbor, OH 43449
Address
WO# 98-000141-000
Repair Organization P.O. No., Job No., etc.

3. Work Performed By Toledo Edison Company Type Code Stamp NR
Name
5501 N. SR 2, Oak Harbor, OH 43449
Address
 Authorization NO. 20
 Expiration Date 7-17-01

4. Identification of Systems Decay Heat Cooler 1-1 to ISO Valve 33B-GCB-10-H32 (SUBSYS 049-02)

5. (a) Applicable Construction Code ANSI B31.1 Edition 19 67 Addendum N/A Code Case N/A
 (b) Applicable Edition of Section XI Utilized for Repairs or Replacements 19 86
 (c) Design Responsibilities ITT Grinnell

6. Identification of Components Repaired or Replaced and Replacement Components

Name of Component	Name of Manufacturer	Manufacturer's Serial NO.	National Board NO.	Other Identification/Jurisdictional NO.	Year Built	Repaired Replaced or Replacement	ASME Code Stamped
Snubber	ITT Grinnell	7977	N/A	A-123	N/A	Replaced	No
Snubber	ITT Grinnell	8018	N/A	A-123	N/A	Replacement	No

7. Description of Work

ITT Grinnell snubber S/N 7977 was removed and replaced with a like for like snubber S/N 8018. Replacement snubber was rebuilt under MWO 1-87-0515-13 and functionally tested satisfactory prior to installation.

8. Tests Conducted: Hydrostatic (NO) Pneumatic (NO) Nominal Operating Pressure (NO) Other (NO)
 Pressure N/A psi Test Temperature N/A °F

Pressure Relief Valves:
 Service N/A Size N/A
 Opening Pressure N/A Blowdown (if applicable) N/A
 Set Pressure and Blowdown Adjustments Made Using N/A
 at N/A (Test Medium)

NOTE: Supplemental sheets in the form of lists, sketches, or drawings may be used, provided (1) size is 8 1/2 x 11 in. (2) Information in items 1 through 6 on this report is included on each sheet, and (3) each sheet is numbered and the number of sheets is recorded at the top of this form.

1. REMARKS

Applicable Manufacturers Data Reports to be Attached

N/A

We certify that the statements made in the report are correct and this replacement conforms to the rules of the ASME Code Section XI.

Type Code Symbol NR Certificate of Authorization No. 20 Expiration Date 7-17-01

Signed *J. W. Starnes* Nuclear Safety & Inspection Manager Date 4/14 1999

I, the undersigned, holding a valid commission issued by the National Board of Boiler and Pressure Vessel Inspectors in the State or Province of Ohio and employed by Protection Mutual of Park Ridge, Illinois have inspected the components described in this Owner's Report during the period 11-18-97 to 4-15-99, and state that to the best of my knowledge and belief, the Owner has performed examinations and taken corrective measures described in this Owner's Report in accordance with the requirements of the ASME Code, Section XI.

By signing this certificate, neither the Inspector nor his employer makes any warranty, expressed or implied, concerning the examinations and corrective measures described in the Owner's Report. Furthermore, neither the Inspector nor his employer shall be liable in any manner for any personal injury or property damage or a loss of any kind arising from or connected with this inspection.

this inspection.

Inspector's Signature 4-15-95 Date

Commissions NB 9801 N.E.A. Ohio
National Board, State, Province and Endorsements

We certify that the statements made in this report are correct and the repair, modification, or replacement of the pressure relief device described above conforms to Section XI and Section III of the ASME Code and the National Board rules as defined in publications NB-65 and NB-102, current editions.

Certificate of Authorization No. 316 to use the "VR" stamp expires Dec. 13, 2001

Certificate of Authorization No. 20 to use the "NR" stamp expires July 17, 2001

Certificate of Authorization No. 20 Was the NRC safety license renewed

Date 12/15/88, 1988, Signed Toledo Edison Authorized Representative

Nuclear Safety & Inspection Manager

I, the undersigned, holding a valid commission issued by the National Board of Boiler and Pressure Vessel Inspectors, and certificate of competency issued by the State or Province of Ohio and employed by Protection Mutual * of Park Ridge, Illinois have inspected the repair, modification, or replacement described in this report

on _____, 19____ and state that to the best of my knowledge and belief, this repair, modification, or replacement has been made or constructed in accordance with Section XI and Section III of the ASME Code and the National Board rules as defined in the publication NB-65 and NB-102, current editions.

By signing this certificate, neither the Inspector nor his employer makes any warranty, expressed or implied, concerning the repair, modification or replacement described in this report. Furthermore, neither the Inspector nor his employer shall be liable in any manner for any personal injury or property damage or a loss of any kind arising from or connected with this inspection.

Authorized Inspector's Signature _____ Date _____ Commissions _____
 National Board, State, Province and Endorsements _____

* Factory Mutual Engineering Association

First Energy Nuclear Operating, 76 South Main St., Akron, Ohio 44308

Davis-Besse Nuclear Power Station Unit #1, 5501 N. St. Rt. 2, Oak Harbor, Ohio 43449

Commercial Service Date: 11/27/77

FORM NIS-2 / NR-1 / NVR-4-OWNER'S REPORT FOR REPAIRS OR REPLACEMENTS As Required by the Provisions of the ASME Code Section XI

1. Owner Toledo Edison Company Date 4/13/99
300 Madison Ave., Toledo, OH 43652 Sheet 1 of 1
Name Address

2. Plant Davis-Besse Nuclear Power Station Unit # 1
5501 N. SR 2, Oak Harbor, OH 43449 WO# 98-000142-000
Name Address Repair Organization P.O. No., Job No., etc.

3. Work Performed By Toledo Edison Company Type Code Stamp NR
5501 N. SR 2, Oak Harbor, OH 43449 Authorization NO. 20
Name Address Expiration Date 7-17-01

4. Identification of Systems Decay Heat Cooler 1-1 to ISO Valve 33B-GCB-10-H32 (SUBSYS 049-02)

5. (a) Applicable Construction Code ANSI B31.1 Edition 19 87 Addendum N/A Code Case N/A
 (b) Applicable Edition of Section XI Utilized for Repairs or Replacements 19 86
 (c) Design Responsibilities ITT Grinnell

6. Identification of Components Repaired or Replaced and Replacement Components

Name of Component	Name of Manufacturer	Manufacturer's Serial NO.	National Board NO.	Other Identification/Jurisdictional NO.	Year Built	Repaired Replaced or Replacement	ASME Code Stamped
Snubber	ITT Grinnell	8004	N/A	A-124	N/A	Replaced	No
Snubber	ITT Grinnell	8906	N/A	A-124	N/A	Replacement	No

7. Description of Work

ITT Grinnell snubber S/N 8004 was removed and replaced with a like for like snubber S/N 8906. Replacement snubber was rebuilt under MWO 1-88-1135-00 and functionally tested satisfactory prior to installation.

8. Tests Conducted: Hydrostatic (NO) Pneumatic (NO) Nominal Operating Pressure (NO) Other (NO)
 Pressure N/A psi Test Temperature N/A °F

Pressure Relief Valves:
 Service N/A Size N/A
 Opening Pressure N/A Blowdown (if applicable) N/A
 Set Pressure and Blowdown Adjustments Made Using N/A
 at N/A (Test Medium)

NOTE: Supplemental sheets in the form of lists, sketches, or drawings may be used, provided (1) size is 8 1/2 x 11 in. (2) Information in items 1 through 6 on this report is included on each sheet, and (3) each sheet is numbered and the number of sheets is recorded at the top of this form.

First Energy Nuclear Operating, 76 South Main St., Akron, Ohio 44308

Davis-Besse Nuclear Power Station Unit #1, 5501 N. St. Rt. 2, Oak Harbor, Ohio 43449

Commercial Service Date: 11/27/77

FORM NIS-2 /NR-1 /NVR-1-

8. REMARKS

Applicable Manufacturer Data Reports to be Attached

N/A

CERTIFICATE OF COMPLIANCE

We certify that the statements made in the report are correct and this replacement conforms to the rules of the ASME Code Section XI.

Type Code Symbol NR Certificate of Authorization No. 20 Expiration Date 7-17-01

Signed [Signature] Nuclear Safety & Inspection Manager Date 4/14 1999

Owner or Owner's Designee, Title

CERTIFICATE OF INSERVICE INSPECTION

I, the undersigned, holding a valid commission issued by the National Board of Boiler and Pressure Vessel Inspectors in the State or Province of Ohio and employed by Protection Mutual of Park Ridge, Illinois have inspected the components described in this Owner's Report during the period 11-18-97 to 4-15-99, and state that to the best of my knowledge and belief, the Owner has performed examinations and taken corrective measures described in this Owner's Report in accordance with the requirements of the ASME Code, Section XI.

By signing this certificate, neither the Inspector nor his employer makes any warranty, expressed or implied, concerning the examinations and corrective measures described in the Owner's Report. Furthermore, neither the Inspector nor his employer shall be liable in any manner for any personal injury or property damage or a loss of any kind arising from or connected with this inspection.

[Signature]
Inspector's Signature

4-15-99
Date

Commissions NB 9501 N. I. A. OHIO
National Board, State, Province and Endorsements

CERTIFICATE OF COMPLIANCE

We certify that the statements made in this report are correct and the repair, modification, or replacement of the pressure relief device described above conforms to Section XI and Section III of the ASME Code and the National Board rules as defined in publications NB-65 and NB-102, current editions.

Certificate of Authorization No. 316 to use the "VR" stamp expires Dec. 13, 2001

Certificate of Authorization No. 20 to use the "NR" stamp expires July 17, 2001

Date _____, 19____, Signed Toledo Edison Nuclear Safety & Inspection Manager

Repair Organization

Authorized Representative

Title

CERTIFICATE OF INSPECTION

I, the undersigned, holding a valid commission issued by the National Board of Boiler and Pressure Vessel Inspectors, and

certificate of competency issued by the State or Province of Ohio and employed by Protection Mutual

of Park Ridge, Illinois have inspected the repair, modification, or replacement described in this report

on _____, 19____ and state that to the best of my knowledge and belief, this repair, modification, or replacement has been made or constructed in accordance with Section XI and Section III of the ASME Code and the National Board rules as defined in the publication NB-65 and NB-102, current editions.

By signing this certificate, neither the Inspector nor his employer makes any warranty, expressed or implied, concerning the repair, modification or replacement described in this report. Furthermore, neither the Inspector nor his employer shall be liable in any manner for any personal injury or property damage or a loss of any kind arising from or connected with this inspection.

Authorized Inspector's Signature

Date

Commissions _____
National Board, State, Province and Endorsements

* Factory Mutual Engineering Association

First Energy Nuclear Operating, 76 South Main St., Akron, Ohio 44308

Davis-Besse Nuclear Power Station Unit #1, 5501 N. St. Rt. 2, Oak Harbor, Ohio 43449

Commercial Service Date: 11/27/77

FORM NIS-2 / NR-1 / NVR-1 OWNER'S REPORT FOR REPAIRS OR REPLACEMENTS As Required by the Provisions of the ASME Code Section XI

1. Owner Toledo Edison Company Date 5/2/00
Name
300 Madison Ave., Toledo, OH 43652
Address
 2. Plant Davis-Besse Nuclear Power Station Unit # 1
Name
5501 N. SR 2, Oak Harbor, OH 43449
Address
 3. Work Performed By Toledo Edison Company Type Code Stamp NR
Name
5501 N. SR 2, Oak Harbor, OH 43449
Address
 Authorization NO. 20
 Expiration Date 7-17-01

4. Identification of Systems Pressurizer Spray Valve RC2 (Subsys 064-04)

5. (a) Applicable Construction Code ASME III CL1 Edition 19 68 Addendum N/A Code Case N/A
 (b) Applicable Edition of Section XI Utilized for Repairs or Replacements 19 86
 (c) Design Responsibilities Velan Engineering Co., Toledo Edison Co

6. Identification of Components Repaired or Replaced and Replacement Components

Name of Component	Name of Manufacturer	Manufacturer's Serial NO.	National Board NO.	Other Identification/Jurisdictional NO.	Year Built	Repaired Replaced or Replacement	ASME Code Stamped
Valve Bonnet	Velan Engineering Co	HT Code 62719	N/A	RC2	1974	Replaced	No
Valve Bonnet	Velan Engineering Co	13691	N/A	RC2	1999	Replacement	No
Discs	Velan Engineering	F9152	N/A	RC2	1999	Replacement	No
Stem	Velan Engineering	HT# J8746	N/A	RC2	1999	Replacement	No
2- 5/8" Studs	Nova Machine	HT# 712917	N/A	RC2	1999	Replacement	No

7. Description of Work

Removed existing valve bonnet heat code 62719 and replaced it with a like for like bonnet serial # 13691. Replaced existing stem and discs.
 Removed bonnet leak off pipe, installed a 1/8" threaded pipe plug and a seal welded performed. Replace two existing bonnet studs.
 Valve was reassembled and a VT-2 leakage test was performed satisfactory.

8. Tests Conducted: Hydrostatic (NO) Pneumatic (NO) Nominal Operating Pressure (YES) Other (NO)
 Pressure 2128 psi Test Temperature NOT °F

Pressure Relief Valves:
 Service N/A Size N/A
 Opening Pressure N/A Blowdown (if applicable) N/A
 Set Pressure and Blowdown Adjustments Made Using N/A
 at N/A (Test Medium)

NOTE: Supplemental sheets in the form of lists, sketches, or drawings may be used, provided (1) size is 8 1/2 x 11 in. (2) information in items 1 through 6 on this report is included on each sheet, and (3) each sheet is numbered and the number of sheets is recorded at the top of this form.

First Energy Nuclear Operating, 76 South Main St., Akron, Ohio 44308

Davis-Besse Nuclear Power Station Unit #1, 5501 N. St. Rt. 2, Oak Harbor, Ohio 43449

Commercial Service Date: 11/27/77

FORM NIS-2 /NR-1 /NVR-1

8. REMARKS

Applicable Manufacturers Data Reports to be Attached

N/A

CERTIFICATE OF COMPLIANCE

We certify that the statements made in the report are correct and this replacement conforms to the rules of the ASME Code Section XI.

Type Code Symbol NR Certificate of Authorization No. 20 Expiration Date 7-17-01

Signed J.W. Starn, Jr. Nuclear Safety & Inspection Manager Date 5/3/2001
Owner or Owner's Designee, Title

CERTIFICATE OF INSERVICE INSPECTION

I, the undersigned, holding a valid commission issued by the National Board of Boiler and Pressure Vessel Inspectors in the State or Province of Ohio and employed by Factory Mutual Ins. Co. of Johnston, R.I. have inspected the components described in this Owner's Report during the period 3/15/99 to 5/4/00, and state that to the best of my knowledge and belief, the Owner has performed examinations and taken corrective measures described in this Owner's Report in accordance with the requirements of the ASME Code, Section XI.

By signing this certificate, neither the Inspector nor his employer makes any warranty, expressed or implied, concerning the examinations and corrective measures described in the Owner's Report. Furthermore, neither the Inspector nor his employer shall be liable in any manner for any personal injury or property damage or a loss of any kind arising from or connected with this inspection.

Inspector's Signature Thomas H. Sage Date ANR 5/4/00 Commissions NB 9336 N I A OHIO
National Board, State, Province and Endorsements

CERTIFICATE OF COMPLIANCE

We certify that the statements made in this report are correct and the repair, modification, or replacement of the pressure relief device described above conforms to Section XI and Section III of the ASME Code and the National Board rules as defined in publications NB-65 and NB-102, current editions.

Certificate of Authorization No. 316 to use the "VR" stamp expires Dec. 13, 2001.

Certificate of Authorization No. 20 to use the "NR" stamp expires July 17, 2001.

Date _____, 19____, Signed Toledo Edison Nuclear Safety & Inspection Manager
Repair Organization Authorized Representative Title

CERTIFICATE OF INSPECTION

I, the undersigned, holding a valid commission issued by the National Board of Boiler and Pressure Vessel Inspectors, and certificate of competency issued by the State or Province of Ohio and employed by Factory Mutual Ins. Co. of Johnston, R.I. have inspected the repair, modification, or replacement described in this report

on _____, 19____ and state that to the best of my knowledge and belief, this repair, modification, or replacement has been made or constructed in accordance with Section XI and Section III of the ASME Code and the National Board rules as defined in the publication NB-65 and NB-102, current editions.

By signing this certificate, neither the Inspector nor his employer makes any warranty, expressed or implied, concerning the repair, modification or replacement described in this report. Furthermore, neither the Inspector nor his employer shall be liable in any manner for any personal injury or property damage or a loss of any kind arising from or connected with this inspection.

Authorized Inspector's Signature _____ Date _____ Commissions _____
National Board, State, Province and Endorsements

* Factory Mutual Ins. Co

First Energy Nuclear Operating, 76 South Main St., Akron, Ohio 44308

Davis-Besse Nuclear Power Station Unit #1, 5501 N. St. Rt. 2, Oak Harbor, Ohio 43449

Commercial Service Date: 11/27/77

FORM NIS-2 / NR-1 / NVR-4 OWNER'S REPORT FOR REPAIRS OR REPLACEMENTS As Required by the Provisions of the ASME Code Section XI

1. Owner Toledo Edison Company Date 5/17/00
Name
300 Madison Ave., Toledo, OH 43652
Address
 Sheet 1 of 1

2. Plant Davis-Besse Nuclear Power Station Unit #1
Name
5501 N. SR 2, Oak Harbor, OH 43449
Address
WO 98-000456-002
Repair Organization P.O. No., Job No., etc.

3. Work Performed By Toledo Edison Company Type Code Stamp NR
Name
5501 N. SR 2, Oak Harbor, OH 43449
Address
 Authorization NO. 20
 Expiration Date 7-17-01

4. Identification of Systems Decay Heat and Low Pressure Injection RO1515 (Subsys 049-02)

5. (a) Applicable Construction Code ASME III CL2 Edition 19 71 Addendum N/A Code Case N/A
 (b) Applicable Edition of Section XI Utilized for Repairs or Replacements 19 86
 (c) Design Responsibilities Toledo Edison Co.

6. Identification of Components Repaired or Replaced and Replacement Components

Name of Component	Name of Manufacturer	Manufacturer's Serial NO.	National Board NO.	Other Identification/Jurisdictional NO.	Year Built	Repaired Replaced or Replacement	ASME Code Stamped
Studs 5/8-11x3-3/4"	Nova	n/a	n/a	Ht 715200	Unk	Replacement	No
Studs 5/8-11x3-1/4"	Nova	n/a	n/a	Ht 707814	Unk	Replacement	No
Nuts 5/8-11	Nova, Cardinal	n/a	n/a	n/a	Unk	Replacement	No

7. Description of Work

Specification M-200 requires bolting to be replaced with SA564, Gr630, when joint is disassembled. Replaced 8 studs (SA564, Gr 630) and 16 nuts (SA 194, Gr 8M) on RO1515. Replaced 4 studs (SA564, Gr630), 4 studs (SA193, Gr B8) and 16 nuts (SA194, Gr8M) on flange south of RO1515. Remaining SA 193, Gr B8 studs will be replaced at a later date when they become available. All materials installed are referenced and listed in this work order.

After installation, a VT-2 inservice test was performed satisfactory.

8. Tests Conducted: Hydrostatic (NO) Pneumatic (NO) Nominal Operating Pressure (YES) Other (NO)
 Pressure N/A psi Test Temperature N/A °F

Pressure Relief Valves:
 Service N/A Size N/A
 Opening Pressure N/A Blowdown (if applicable) N/A
 Set Pressure and Blowdown Adjustments Made Using N/A
 at N/A

NOTE: Supplemental sheets in the form of lists, sketches, or drawings may be used, provided (1) size is 8 1/2 x 11 in. (2) Information in items 1 through 6 on this report is included on each sheet, and (3) each sheet is numbered and the number of sheets is recorded at the top of this form.

First Energy Nuclear Operating, 76 South Main St., Akron, Ohio 44308

Davis-Besse Nuclear Power Station Unit #1, 5501 N. St. Rt. 2, Oak Harbor, Ohio 43449

Commercial Service Date: 11/27/77

FORM NIS-2 / NR-1 / NVR-4

NA

8. REMARKS

Applicable Manufacturers Data Reports to be Attached

CERTIFICATE OF COMPLIANCE

We certify that the statements made in the report are correct and this replacement conforms to the rules of the ASME Code Section XI.

Type Code Symbol NR Certificate of Authorization No. 20 Expiration Date 7-17-01

Signed [Signature] Nuclear Safety & Inspection Manager Date 5/17/00
Owner or Owner's Designee, Title

CERTIFICATE OF INSERVICE INSPECTION

I, the undersigned, holding a valid commission issued by the National Board of Boiler and Pressure Vessel Inspectors in the State or Province of Ohio and employed by Factory Mutual Ins. Co. of Johnston, R.I. have inspected the components described in this Owner's Report during the period 4-18-00 to 5-17-00, and state that to the best of my knowledge and belief, the Owner has performed examinations and taken corrective measures described in this Owner's Report in accordance with the requirements of the ASME Code, Section XI.

By signing this certificate, neither the Inspector nor his employer makes any warranty, expressed or implied, concerning the examinations and corrective measures described in the Owner's Report. Furthermore, neither the Inspector nor his employer shall be liable in any manner for any personal injury or property damage or a loss of any kind arising from or connected with this inspection.

Inspector's Signature

Date 5-17-00

Commissions NR 9101 "N" "E" "A" OHIO
National Board, State, Province and Endorsements

CERTIFICATE OF COMPLIANCE

We certify that the statements made in this report are correct and the repair, modification, or replacement of the pressure relief device described above conforms to Section XI and Section III of the ASME Code and the National Board rules as defined in publications NB-65 and NB-102, current editions.

Certificate of Authorization No. 316 to use the "VR" stamp expires Dec. 13, 2001

Certificate of Authorization No. 20 to use the "NR" stamp expires July 17, 2001

Date , 19 , Signed Toledo Edison Nuclear Safety & Inspection Manager
Repair Organization Authorized Representative Title

CERTIFICATE OF INSPECTION

I, the undersigned, holding a valid commission issued by the National Board of Boiler and Pressure Vessel Inspectors, and certificate of competency issued by the State or Province of Ohio and employed by Factory Mutual Ins. Co.

of Johnston, R.I. have inspected the repair, modification, or replacement described in this report

on , 19 and state that to the best of my knowledge and belief, this repair, modification, or replacement has been made or constructed in accordance with Section XI and Section III of the ASME Code and the National Board rules as defined in the publication NB-65 and NB-102, current editions.

By signing this certificate, neither the Inspector nor his employer makes any warranty, expressed or implied, concerning the repair, modification or replacement described in this report. Furthermore, neither the Inspector nor his employer shall be liable in any manner for any personal injury or property damage or a loss of any kind arising from or connected with this inspection.

Authorized Inspector's Signature

Date

Commissions

National Board, State, Province and Endorsements

* Factory Mutual Ins. Co

First Energy Nuclear Operating, 76 South Main St., Akron, Ohio 44308

Davis-Besse Nuclear Power Station Unit #1, 5501 N. St. Rt. 2, Oak Harbor, Ohio 43449

Commercial Service Date: 11/27/77

FORM NIS-2 / NR-1 / ~~AWR-1~~ OWNER'S REPORT FOR REPAIRS OR REPLACEMENTS As Required by the Provisions of the ASME Code Section XI

1. Owner Toledo Edison Company Date 5/16/00
Name
300 Madison Ave., Toledo, OH 43652
Address Sheet 1 of 1

2. Plant Davis-Besse Nuclear Power Station Unit # 1
Name
5501 N. SR 2, Oak Harbor, OH 43449
Address WO 98-000718-000
Repair Organization P.O. No., Job No., etc.

3. Work Performed By Toledo Edison Company Type Code Stamp NR
Name
5501 N. SR 2, Oak Harbor, OH 43449
Address Authorization NO. 20
 Expiration Date 7-17-01

4. Identification of Systems DH Cooldown Line to Emer Sump Relief Valve DH 4849 (Subsys 049-02)

5. (a) Applicable Construction Code ASME III CL2 Edition 19 71 Addendum W72 Code Case 1555 + 1574
 (b) Applicable Edition of Section XI Utilized for Repairs or Replacements 19 86
 (c) Design Responsibilities J. E. Lonergan Co.

6. Identification of Components Repaired or Replaced and Replacement Components

Name of Component	Name of Manufacturer	Manufacturer's Serial NO.	National Board NO.	Other Identification/Jurisdictional NO.	Year Built	Repaired Replaced or Replacement	ASME Code Stamped
Relief valve	J. E. Lonergan Co.	304400-104-1	N/A	DH 4849	1976	Replaced	Yes
Relief Valve	J. E. Lonergan Co.	7704435-104	N/A	DH 4849	1982	Replacement	Yes

7. Description of Work

Removed existing relief valve S/N 304400-104-1 and replaced it with a like for like spare valve S/N 7704435-104

Replacement valve was tested by MWO 1-05-0220-03, placed in storage and seat leakage tested prior to installation.

After installation a VT-2 test was performed satisfactory.

8. Tests Conducted: Hydrostatic (NO) Pneumatic (NO) Nominal Operating Pressure (YES) Other (NO)
 Pressure 227 psi Test Temperature NOT °F

Pressure Relief Valves:
 Service N/A Size N/A
 Opening Pressure N/A Blowdown (if applicable): N/A
 Set Pressure and Blowdown Adjustments Made Using N/A (Test Station)
 at N/A (Location)

NOTE: Supplemental sheets in the form of lists, sketches, or drawings may be used, provided (1) size is 8 1/2 x 11 in. (2) Information in items 1 through 6 on this report is included on each sheet, and (3) each sheet is numbered and the number of sheets is recorded at the top of this form.

First Energy Nuclear Operating, 76 South Main St., Akron, Ohio 44308

Davis-Besse Nuclear Power Station Unit #1, 5501 N. St. Rt. 2, Oak Harbor, Ohio 43449

Commercial Service Date: 11/27/77

FORM NIS-2 /NR-1 /NVR-1

8. REMARKS

Applicable Manufacturers Data Reports to be Attached

NA

CERTIFICATE OF COMPLIANCE

We certify that the statements made in the report are correct and this replacement conforms to the rules of the ASME Code Section XI.

Type Code Symbol NR Certificate of Authorization No. 20 Expiration Date 7-17-01

Signed [Signature] Nuclear Safety & Inspection Manager Date 5/17/00
Owner or Owner's Designee, Title

CERTIFICATE OF INSERVICE INSPECTION

I, the undersigned, holding a valid commission issued by the National Board of Boiler and Pressure Vessel Inspectors in the State or Province of Ohio and employed by Factory Mutual Ins. Co. of Johnston, R.I. have inspected the components described in this Owner's Report during the period 5-20-99 to 5-17-00, and state that to the best of my knowledge and belief, the Owner has performed examinations and taken corrective measures described in this Owner's Report in accordance with the requirements of the ASME Code, Section XI.

By signing this certificate, neither the Inspector nor his employer makes any warranty, expressed or implied, concerning the examinations and corrective measures described in the Owner's Report. Furthermore, neither the Inspector nor his employer shall be liable in any manner for any personal injury or property damage or a loss of any kind arising from or connected with this inspection.

Inspector's Signature [Signature] Date 5-17-00 Commissions NB 9101 N.I.A. [Signature]
National Board, State, Province and Endorsements

CERTIFICATE OF COMPLIANCE

We certify that the statements made in this report are correct and the repair, modification, or replacement of the pressure relief device described above conforms to Section XI and Section III of the ASME Code and the National Board rules as defined in publications NB-65 and NB-102, current editions.

Certificate of Authorization No. 316 to use the "VR" stamp expires Dec. 13, 2001

Certificate of Authorization No. 20 to use the "NR" stamp expires July 17, 2001

Date , 19 , Signed Toledo Edison Nuclear Safety & Inspection Manager
Repair Organization Authorized Representative Title

CERTIFICATE OF INSPECTION

I, the undersigned, holding a valid commission issued by the National Board of Boiler and Pressure Vessel Inspectors, and certificate of competency issued by the State or Province of Ohio and employed by Factory Mutual Ins. Co.

of Johnston, R.I. have inspected the repair, modification, or replacement described in this report

on , 19 and state that to the best of my knowledge and belief, this repair, modification, or replacement has been made or constructed in accordance with Section XI and Section III of the ASME Code and the National Board rules as defined in the publication NB-65 and NB-102, current editions.

By signing this certificate, neither the Inspector nor his employer makes any warranty, expressed or implied, concerning the repair, modification or replacement described in this report. Furthermore, neither the Inspector nor his employer shall be liable in any manner for any personal injury or property damage or a loss of any kind arising from or connected with this inspection.

Authorized Inspector's Signature Date Commissions
National Board, State, Province and Endorsements

* Factory Mutual Ins. Co

First Energy Nuclear Operating, 76 South Main St., Akron, Ohio 44308

Davis-Besse Nuclear Power Station Unit #1, 5501 N. St. Rt. 2, Oak Harbor, Ohio 43449

Commercial Service Date: 11/27/77

FORM NIS-2 / NR-1 / NVR-1 OWNER'S REPORT FOR REPAIRS OR REPLACEMENTS As Required by the Provisions of the ASME Code Section XI

1. Owner Toledo Edison Company Date 5/23/00
Name
300 Madison Ave., Toledo, OH 43652
Address Sheet 1 of 1
2. Plant Davis-Besse Nuclear Power Station Unit # 1
Name
5501 N. SR 2, Oak Harbor, OH 43449
Address WO 98-000913-001
Repair Organization P.O. No., Job No., etc.
3. Work Performed By Toledo Edison Company Type Code Stamp NR
Name
5501 N. SR 2, Oak Harbor, OH 43449
Address Authorization NO. 20
 Expiration Date 7-17-01
4. Identification of Systems MS Line #2 Atmospheric Vent Valve (AVV) ICS11A (Subsys 083-01)
5. (a) Applicable Construction Code ASME III CL2 Edition 19 71 Addendum N/A Code Case N/A
 (b) Applicable Edition of Section XI Utilized for Repairs or Replacements 19 86
 (c) Design Responsibilities Control Components Inc

6. Identification of Components Repaired or Replaced and Replacement Components

Name of Component	Name of Manufacturer	Manufacturer's Serial NO.	National Board NO.	Other Identification/Jurisdictional NO.	Year Built	Repaired Replaced or Replacement	ASME Code Stamped
Atmos Vent Valve	Western Piping & Eng.	I	N/A	ICS11A	1974	Replaced	Yes
Atmos Vent Valve	Control Components	645141-1-1	N/A	ICS11A	1994	Replacement	Yes
Inlet Flange Studs	Nova Machine Prod	HT# 28010	N/A	N/A	2000	Replacement	No
Inlet Flange Nuts	Nova Machine Prod	HT# 8099455	N/A	N/A	2000	Replacement	No

7. Description of Work

Removed existing atmospheric vent valve S/N I and replaced it with a new replacement valve S/N 645141-1-1.
 Also replaced existing inlet flange bolting. After installation, a VT-2 Inservice test was performed satisfactory.

8. Tests Conducted: Hydrostatic (NO) Pneumatic (NO) Nominal Operating Pressure (YES) Other (NO)
 Pressure 870 psi Test Temperature NOT °F
- Pressure Relief Valves:
 Service N/A Size N/A
 Opening Pressure N/A Blowdown (if applicable) N/A
 Set Pressure and Blowdown Adjustments Made Using N/A
 at N/A

NOTE: Supplemental sheets in the form of lists, sketches, or drawings may be used, provided (1) size is 8 1/2 x 11 in.
 (2) Information in items 1 through 6 on this report is included on each sheet, and (3) each sheet is numbered and the number of sheets is recorded at the top of this form.

12 RFO Inservice Inspection Summary

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First Energy Nuclear Operating, 76 South Main St., Akron, Ohio 44308

Davis-Besse Nuclear Power Station Unit #1, 5501 N. St. Rt. 2, Oak Harbor, Ohio 43449

Commercial Service Date: 11/27/77

FORM NIS-2 /NR-1 /NVR-1

8. REMARKS

Applicable Manufacturers Data Reports to be Attached

Control Components Inc.

CERTIFICATE OF COMPLIANCE

We certify that the statements made in the report are correct and this replacement conforms to the rules of the ASME Code Section XI.

Type Code Symbol NR Certificate of Authorization No. 20 Expiration Date 7-17-01

Signed St. W. Starnes Nuclear Safety & Inspection Manager Date 5/23/2000
Owner or Owner's Designee, Title

CERTIFICATE OF INSERVICE INSPECTION

I, the undersigned, holding a valid commission issued by the National Board of Boiler and Pressure Vessel Inspectors in the State or Province of Ohio and employed by Factory Mutual Ins. Co. of Johnston, R.I. have inspected the components described in this Owner's Report during the period 3/15/00 to 5/30/00, and state that to the best of my knowledge and belief, the Owner has performed examinations and taken corrective measures described in this Owner's Report in accordance with the requirements of the ASME Code, Section XI.

By signing this certificate, neither the Inspector nor his employer makes any warranty, expressed or implied, concerning the examinations and corrective measures described in the Owner's Report. Furthermore, neither the Inspector nor his employer shall be liable in any manner for any personal injury or property damage or a loss of any kind arising from or connected with this inspection.

Inspector's Signature Thomas A. Pope Date 5/30/00 Commissions NB 9330 "NIA" OHIO
National Board, State, Province and Endorsements

CERTIFICATE OF COMPLIANCE

We certify that the statements made in this report are correct and the repair, modification, or replacement of the pressure relief device described above conforms to Section XI and Section III of the ASME Code and the National Board rules as defined in publications NB-65 and NB-102, current editions.

Certificate of Authorization No. 316 to use the "VR" stamp expires Dec. 13, 2001.

Certificate of Authorization No. 20 to use the "NR" stamp expires July 17, 2001.

Date _____, 19____, Signed Toledo Edison Nuclear Safety & Inspection Manager
Repair Organization Authorized Representative Title

CERTIFICATE OF INSPECTION

I, the undersigned, holding a valid commission issued by the National Board of Boiler and Pressure Vessel Inspectors, and certificate of competency issued by the State or Province of Ohio and employed by Factory Mutual Ins. Co. of Johnston, R.I. have inspected the repair, modification, or replacement described in this report

on _____, 19____ and state that to the best of my knowledge and belief, this repair, modification, or replacement has been made or constructed in accordance with Section XI and Section III of the ASME Code and the National Board rules as defined in the publication NB-65 and NB-102, current editions.

By signing this certificate, neither the Inspector nor his employer makes any warranty, expressed or implied, concerning the repair, modification or replacement described in this report. Furthermore, neither the Inspector nor his employer shall be liable in any manner for any personal injury or property damage or a loss of any kind arising from or connected with this inspection.

Authorized Inspector's Signature _____ Date _____ Commissions _____
National Board, State, Province and Endorsements

* Factory Mutual Ins. Co

As Required by the Provisions of the ASME Code, Section III, Division 1

As Required by the Provisions of the ASME Code, Section III, Division 1

- | | | | | | | | | | | | | | | | | |
|---------------------------------------|--|--|--------------------|--|------------|--|----------------|--|-------------|--|---------|--|----------|--|-----------------|--|
| 1. Manufactured and certified by | Cont'l Components Inc. 22591 Avenida Express, Rancho Santa Margarita, CA 92688 | | | | | | | | | | | | | | | |
| 2. Manufactured for | TOLEDO EDISON COMPANY, 5501 NORTH STATE ROUTE 2, OAK HARBOR, OH. 43449 | | | | | | | | | | | | | | | |
| 3. Location of installation | DAVIS-BESSE POWER STATION, 5501 NORTH STATE ROUTE 2, OAK HARBOR, OH. 43449 | | | | | | | | | | | | | | | |
| 4. Model No., Serial No., or Type | ATMOSPHERIC | | VENT-VALVE | | Drawing | | 6-5141-1 | | Rev. | | E | | CRN | | N/A | |
| 5. ASME Code, Section III, Division 1 | 1971 | | (pressure) | | NO ADDENDA | | (addenda data) | | 2 | | (class) | | N/A | | (code case no.) | |
| 6. Pump or Valve | VALVE | | Nominal inlet size | | 6 INCH | | (in) | | Outlet size | | 55 INCH | | (in) | | | |
| 7. Material | SA-105 | | Bonnets | | SA-103 | | Disk | | SA-182-F11 | | Bolting | | SA-194-7 | | SA-183-B7 and | |

(a) Cart. Holder Serial No.	(b) Natl Board No.	(c) Body Serial No.	(d) Bonnet Serial No.	(e) Disk Serial No.
24541-1-1	N/A	240875	240875	80052
24541-1-2	N/A	240875	240875	80052

^aSupplementary information to this form is given, which may be used provided (1) size is 12 x 11; (2) information is limited to through-4; (3) information is limited to through-4; (4) information is limited to through-4.

Information may be obtained from ASME Order Dept., ASME, 22 Law Drive, Box 2300, Fairfield, NJ 07004-2300.

P03-00171-0003
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DBNP 3

DBNFs

First Energy Nuclear Operating, 76 South Main St., Akron, Ohio 44308

Davis-Besse Nuclear Power Station Unit #1, 5501 N. St. Rt. 2, Oak Harbor, Ohio 43449

Commercial Service Date: 11/27/77

FORM NPV-1 (Back - Pg. 2 of 2)

Certificate Holder's Serial No. 645141-1-1 & -1-2

8. Design conditions 1050 psi 600 Deg. F or Valve pressure class 800 ANSI (1)
 (pressure) (temperature)
 9. Cold working pressure 2220 psi at 100 Deg. F
 10. Hydrostatic test 3350 psi Disk differential pressure N/A psi

11. Remarks: LISTING OF ADDITIONAL CODE PARTS ON THESE ATMOSPHERIC VENT VALVES.

- BODY INLET FLANGE - MAT'L SA-105, MAT'L HEAT #EXP4.
 - BODY INLET/OUTLET EXTENSIONS - MAT'L SA-105 GRADE C, MAT'L HEAT 93228.
 - BODY OUTLET BASE FLANGE - MAT'L SA-105, MAT'L HEAT 240875.
 - BONNET DRAIN PIPE - MAT'L SA-105, MAT'L HEAT #222963.
 - BONNET DRAIN FLANGE - MAT'L SA-105, MAT'L HEAT #F3978.
 - BODY/BONNET BOLTING - STUD MAT'L SA-193-B7, HEAT CODE #F845 / NUT MAT'L SA-194-7, HEAT CODE #F845

CERTIFICATE OF DESIGN

Design Specification certified by PRASOON K. GOYAL P.E. State IL Reg. no. 30256
 Design Report certified by HERBERT L. MILLER P.E. State OH Reg. no. E-032776

CERTIFICATE OF COMPLIANCE

We certify that the statements made in this report are correct and that this pump or valve conforms to the rules for construction of the ASME Code, Section III, Division 1.

N Certificate of Authorization No. 2695 Expires JUNE 7, 1994 ***
 Date 6-29-94 Name CONTROL COMPONENTS INC. Signed Fred Zullo
 (N Certificate Holder) (authorized representative)

*** SEE ATTACHED "LETTER OF EXTENSION" FROM ASME REGARDING CCT'S EXPIRATION DATE.

CERTIFICATE OF INSPECTION

I, the undersigned, holding a valid commission issued by the National Board of Boiler and Pressure Vessel Inspectors and the State or Province of CALIFORNIA and employed by HARTFORD STEAM BOILER INSPECTION AND INSURANCE COMPANY of HARTFORD, CONNECTICUT have inspected the pump or valve, described in this Data Report on 6-29-94, and state that to the best of my knowledge and belief, the Certificate Holder has constructed the pump or valve in accordance with ASME Section III, Division 1.

By signing this certificate, neither the inspector nor his employer makes warranty, expressed or implied, concerning the component described in this Data Report. Furthermore, neither the inspector nor his employer shall be liable in any manner for any personal injury or property loss of any kind resulting from or connected with this inspection.

Date 6-29-94 Signed [Signature] Commissions [Signature]
 (Authorized Inspector) (NABIB: (including endorsement) and state or prov. and no.)

(1) For manually operated valves only

PO # 105141-1-1-2
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05/23/2000

DBNPS

First Energy Nuclear Operating, 76 South Main St., Akron, Ohio 44308

Davis-Besse Nuclear Power Station Unit #1, 5501 N. St. Rt. 2, Oak Harbor, Ohio 43449

Commercial Service Date: 11/27/77

FORM NIS-2 / NR-1 / HVR-1 OWNER'S REPORT FOR REPAIRS OR REPLACEMENTS As Required by the Provisions of the ASME Code Section XI

1. Owner Toledo Edison Company Date 5/23/00
Name
300 Madison Ave., Toledo, OH 43652
Address Sheet 1 of 1

2. Plant Davis-Besse Nuclear Power Station Unit # 1
Name
5501 N. St. Rt. 2, Oak Harbor, OH 43449
Address WO 88-000914-001
Repair Organization P.O. No., Job No., etc.

3. Work Performed By Toledo Edison Company Type Code Stamp NB
Name
5501 N. St. Rt. 2, Oak Harbor, OH 43449
Address Authorization NO. 20
Expiration Date 7-17-01

4. Identification of Systems MS Line #1 Atmospheric Vent Valve (AVV) ICS11B (Subsys 083-01)

5. (a) Applicable Construction Code ASME III CL2 Edition 19 71 Addendum N/A Code Case N/A
(b) Applicable Edition of Section XI Utilized for Repairs or Replacements 19 86
(c) Design Responsibilities Control Components Inc

6. Identification of Components Repaired or Replaced and Replacement Components

Name of Component	Name of Manufacturer	Manufacturer's Serial NO.	National Board NO.	Other Identification/Jurisdictional NO.	Year Built	Repaired Replaced or Replacement	ASME Code Stamped
Atmos Vent Valve	Western Piping & Eng.	II	N/A	ICS11B	1974	Replaced	Yes
Atmos Vent Valve	Control Components	645141-1-2	N/A	ICS11B	1994	Replacement	Yes
Inlet Flange Studs	Nova Machine Prod	HT# 28010	N/A	N/A	2000	Replacement	No
Inlet Flange Nuts	Nova Machine Prod	HT# 8099455	N/A	N/A	2000	Replacement	No

7. Description of Work

Removed existing atmospheric vent valve S/N II and replaced it with a new replacement valve S/N 645141-1-2.
Also replaced existing inlet flange bolting. After installation, A VT-2 Inservice test was performed satisfactory.

8. Tests Conducted: Hydrostatic (NO) Pneumatic (NO) Nominal Operating Pressure (YES) Other (NO)
Pressure 866 psi Test Temperature NOT °F

Pressure Relief Valves:
Service N/A Size N/A
Opening Pressure N/A Blowdown (if applicable) N/A
Set Pressure and Blowdown Adjustments Made Using N/A
at N/A (Test Medium)

NOTE: Supplemental sheets in the form of lists, sketches, or drawings may be used, provided (1) size is 8 1/2 x 11 in. (2) Information in items 1 through 6 on this report is included on each sheet, and (3) each sheet is numbered and the number of sheets is recorded at the top of this form.

First Energy Nuclear Operating, 76 South Main St., Akron, Ohio 44308

Davis-Besse Nuclear Power Station Unit #1, 5501 N. St. Rt. 2, Oak Harbor, Ohio 43449

Commercial Service Date: 11/27/77

FORM NIS-2 / NR-1 / NVR-4

8. REMARKS

Applicable Manufacturers Data Reports to be Attached

Control Components Inc.

CERTIFICATE OF COMPLIANCE

We certify that the statements made in the report are correct and this replacement conforms to the rules of the ASME Code Section XI.

Type Code Symbol NR Certificate of Authorization No. 20 Expiration Date 7-17-01

Signed J. W. Starnes, Jr. Nuclear Safety & Inspection Manager Date 5/23/2002
Owner or Owner's Designee, Title

CERTIFICATE OF INSERVICE INSPECTION

I, the undersigned, holding a valid commission issued by the National Board of Boiler and Pressure Vessel Inspectors in the State or Province of Ohio and employed by Factory Mutual Ins. Co. of Johnston, R.I. have inspected the components described in this Owner's Report during the period 5/15/00 to 5/30/00, and state that to the best of my knowledge and belief, the Owner has performed examinations and taken corrective measures described in this Owner's Report in accordance with the requirements of the ASME Code, Section XI.

By signing this certificate, neither the Inspector nor his employer makes any warranty, expressed or implied, concerning the examinations and corrective measures described in the Owner's Report. Furthermore, neither the Inspector nor his employer shall be liable in any manner for any personal injury or property damage or a loss of any kind arising from or connected with this inspection.

Thomas H. Lape 5/30/00 Commissions NB 9330 "NIA" OHIO
Inspector's Signature Date National Board, State, Province and Endorsements

CERTIFICATE OF COMPLIANCE

We certify that the statements made in this report are correct and the repair, modification, or replacement of the pressure relief device described above conforms to Section XI and Section III of the ASME Code and the National Board rules as defined in publications NB-65 and NB-102, current editions.

Certificate of Authorization No. 316 to use the "VR" stamp expires Dec. 13, 2001.

Certificate of Authorization No. 20 to use the "NR" stamp expires July 17, 2001.

Date _____, 19____, Signed Toledo Edison Nuclear Safety & Inspection Manager
Repair Organization Authorized Representative Title

CERTIFICATE OF INSPECTION

I, the undersigned, holding a valid commission issued by the National Board of Boiler and Pressure Vessel Inspectors, and certificate of competency issued by the State or Province of Ohio and employed by Factory Mutual Ins. Co.

of Johnston, R.I. have inspected the repair, modification, or replacement described in this report

on _____, 19____ and state that to the best of my knowledge and belief, this repair, modification, or replacement has been made or constructed in accordance with Section XI and Section III of the ASME Code and the National Board rules as defined in the publication NB-65 and NB-102, current editions.

By signing this certificate, neither the Inspector nor his employer makes any warranty, expressed or implied, concerning the repair, modification or replacement described in this report. Furthermore, neither the Inspector nor his employer shall be liable in any manner for any personal injury or property damage or a loss of any kind arising from or connected with this inspection.

Authorized Inspector's Signature _____ Date _____ Commissions NB 9330 "NIA" OHIO
National Board, State, Province and Endorsements

* Factory Mutual Ins. Co

Commercial Service Date: 11/27/77

As Required by the Provisions of the ASME Code, Section III, Division 1

Pg. 1 of 2

1. Manufactured and certified by Control Components Inc. 22591 Avenida Emperesa, Rancho Santa Margarita, CA 92688
(name and address of Certificate Holder)

2. Manufactured for TOLEDO EDISON COMPANY, 5501 NORTH STATE ROUTE 2, OAK HARBOR, OH, 43449
(name and address of purchaser)

3. Location of installation DAVIS-BESSE POWER STATION, 5501 NORTH STATE ROUTE 2, OAK HARBOR, OH, 43449
ATMOSPHERIC (name and address)

4. Model No., Serial No., or Type VENT-VALVE Drawing 645141-1 Rev. E CRN N/A

5. ASME Code, Section II, Division 1: 1971 NO ADDENDA 2 N/A
(edition) (addenda date) (class) (Code Case no.)

6. Pump or Valve VALVE Nominal inlet size 8 INCH Outlet size 36 INCH
(in) (in) SA-193-B7 and

7. Material: Body SA-105 Bonnet SA-105 Disk SA-182-F11 Bolting SA-194-7

[illegible]

Supplemental information in the form of lists, sketches, or drawings may be used provided: (1) size is 8 1/2 x 11; (2) information is itemized through the Data Reports included on each sheet; (3) each sheet is numbered and the number of sheets is recorded at the top of this form.

This form (E00037) may be obtained from the Order Dept., ASME, 22 Law Drive, Box 2300, Fairfield, NJ 07002-2300.

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First Energy Nuclear Operating, 76 South Main St., Akron, Ohio 44308

Davis-Besse Nuclear Power Station Unit #1, 5501 N. St. Rt. 2, Oak Harbor, Ohio 43449

Commercial Service Date: 11/27/77

FORM NPV-1 (Back - Pg. 2 of 2)

Certificate Holder's Serial No. 645141-1-1 & -1-2

8. Design conditions 1050 psi 600 Deg. F or Valve pressure class 600 ANSI (1)
 (pressure) (temperature)
 9. Cold working pressure 2220 psi at 100 Deg. F
 10. Hydrostatic test 8350 psi Disk differential pressure N/A psi

11. Remarks LISTING OF ADDITIONAL CODE PARTS ON THESE ATMOSPHERIC VENT VALVES.

- BODY INLET FLANGE - MAT'L SA-105, MAT'L HEAT #EXP4.
- BODY INLET/OUTLET EXTENSIONS - MAT'L SA-106 GRADE C, MAT'L HEAT #3228.
- BODY OUTLET BASE FLANGE - MAT'L SA-105, MAT'L HEAT #240875.
- BONNET DRAIN PIPE - MAT'L SA-105, MAT'L HEAT #222983.
- BONNET DRAIN FLANGE - MAT'L SA-105, MAT'L HEAT #F3978.
- BODY/BONNET BOLTING - STUD MAT'L SA-183-87, HEAT CODE #F845 / NUT MAT'L SA-184-7, HEAT CODE #F846

CERTIFICATE OF DESIGN

Design Specification certified by PRASOON K. GOYAL P.E. State IL Reg. no. 36256
 Design Report certified by HERBERT L. MILLER P.E. State OH Reg. no. E-632776

CERTIFICATE OF COMPLIANCE

We certify that the statements made in this report are correct and that this pump or valve conforms to the rules for construction of the ASME Code, Section III, Division 1.

N Certificate of Authorization No. 2695 Expires JUNE 7, 1994 ***

Date 6-29-94 Name CONTROL COMPONENTS INC. Signed Fred J. [Signature]
 (In Certificate Holder) (authorized representative)

***SEE ATTACHED "LETTER OF EXTENSION" FROM ASME REGARDING CCA'S EXPIRATION DATE.

CERTIFICATE OF INSPECTION

I, the undersigned, holding a valid commission issued by the National Board of Boiler and Pressure Vessel Inspectors and the State or Province of CALIFORNIA and employed by HARTFORD STEAM BOILER INSPECTION AND INSURANCE COMPANY of HARTFORD, CONNECTICUT have inspected the pump, or valve, described in this Data Report on 6-29-94, and state that to the best of my knowledge and belief, the Certificate Holder has constructed this pump or valve in accordance with ASME Section III, Division 1.

By signing this certificate, neither the inspector nor his employer makes warranty, expressed or implied, concerning the component described in this Data Report. Furthermore, neither the inspector nor his employer shall be liable in any manner for any personal injury or property loss of any kind arising from or connected with this inspection.

Date 6-29-94 Signed William [Signature] Commissions CA1494
 (authorized inspector) (State including endorsement and state approval no.)

NOT FOR use on valves only

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First Energy Nuclear Operating, 76 South Main St., Akron, Ohio 44308

Davis-Besse Nuclear Power Station Unit #1, 5501 N. St. Rt. 2, Oak Harbor, Ohio 43449

Commercial Service Date: 11/27/77

FORM NIS-2 / NR-1 / NVR-1 OWNER'S REPORT FOR REPAIRS OR REPLACEMENTS As Required by the Provisions of the ASME Code Section XI

1. Owner Toledo Edison Company Date 5/27/89
Name
300 Madison Ave., Toledo, OH 43652
Address
 Sheet 1 of 1

2. Plant Davis-Besse Nuclear Power Station Unit # 1
Name
5501 N. SR 2, Oak Harbor, OH 43449
Address
WO# 88-001201-000 (3-88-4805-01)
Repair Organization P.O. No., Job No., etc.

3. Work Performed By Toledo Edison Company Type Code Stamp NR / VR
Name
5501 N. SR 2, Oak Harbor, OH 43449
Address
 Authorization NO. 20 / 316
 Expiration Date 7-17-01 / 12-13-01

4. Identification of Systems Pressurizer Code Safety Valve SPARE (Subsys 064-04)

5. (a) Applicable Construction Code ASME III CL1 Edition 19 71 Addendum W72 Code Case N/A
 (b) Applicable Edition of Section XI Utilized for Repairs or Replacements 19 86
 (c) Design Responsibilities Crosby Valve & Gage Co., Toledo Edison Co.

6. Identification of Components Repaired or Replaced and Replacement Components

Name of Component	Name of Manufacturer	Manufacturer's Serial NO.	National Board NO.	Other Identification/Jurisdictional NO.	Year Built	Repaired Replaced or Replacement	ASME Code Stamped
Safety Valve	Crosby Valve	N56264000005	N/A	Spare Valve	1976	Repaired	Yes

7. Description of Work

This is a dedicated spare pressurizer code safety valve. This valve was sent to Wyle Laboratories for testing and rebuild. As found set pressure test failed. Valve was removed from the test header, jack & lapped by Toledo Edison under their N.B.-NR/VR stamp using MWO 3-88-4805-01. After valve reassembly it was placed by on the test header and tested satisfactory.

8. Tests Conducted: Hydrostatic (NO) Pneumatic (NO) Nominal Operating Pressure (NO) Other (NO)
 Pressure N/A psi Test Temperature N/A °F

Pressure Relief Valves:
 Service Steam Size 4"x6"
 Opening Pressure 2512 PSIG Blowdown (if applicable) N/A
 Set Pressure and Blowdown Adjustments Made Using Steam
 at Wyle Laboratories, 7800 Highway 20 West, Huntsville, AL
(Test Station)
(Location)

NOTE: Supplemental sheets in the form of lists, sketches, or drawings may be used, provided (1) size is 8 1/2 x 11 in. (2) Information in items 1 through 6 on this report is included on each sheet, and (3) each sheet is numbered and the number of sheets is recorded at the top of this form.

First Energy Nuclear Operating, 76 South Main St., Akron, Ohio 44308

Davis-Besse Nuclear Power Station Unit #1, 5501 N. St. Rt. 2, Oak Harbor, Ohio 43449

Commercial Service Date: 11/27/77

FORM NIS-2 / NR-1 / NVR-1

9. REMARKS

Applicable Manufacturers Data Reports to be Attached

N/A

CERTIFICATE OF COMPLIANCE

We certify that the statements made in the report are correct and this replacement conforms to the rules of the ASME Code Section XI.

Type Code Symbol NR Certificate of Authorization No. 20 Expiration Date 7-17-01

Signed [Signature] Nuclear Safety & Inspection Manager Date 5/27 1999

Owner or Owner's Designee, Title

CERTIFICATE OF INSERVICE INSPECTION

I, the undersigned, holding a valid commission issued by the National Board of Boiler and Pressure Vessel Inspectors in the State or Province of Ohio and employed by Protection Mutual of Park Ridge, Illinois TEL 6/8/99 have inspected the components described in this Owner's Report during the period 10/21/98 to 6/8/99, and state that to the best of my knowledge and belief, the Owner has performed examinations and taken corrective measures 6/8/99 described in this Owner's Report in accordance with the requirements of the ASME Code, Section XI.

By signing this certificate, neither the Inspector nor his employer makes any warranty, expressed or implied, concerning the examinations and corrective measures described in the Owner's Report. Furthermore, neither the Inspector nor his employer shall be liable in any manner for any personal injury or property damage or a loss of any kind arising from or connected with this inspection.

Thomas S. Laps
Inspector's Signature

6/8/99
Date

Commissions NB 9330 "N" "I" "A" OHIO
National Board, State, Province and Endorsements COMM.

CERTIFICATE OF COMPLIANCE

We certify that the statements made in this report are correct and the repair, modification, or replacement of the pressure relief device described above conforms to Section XI and Section III of the ASME Code and the National Board rules as defined in publications NB-65 and NB-102, current editions.

Certificate of Authorization No. 316 to use the "VR" stamp expires Dec. 13, 2001

Certificate of Authorization No. 20 to use the "NR" stamp expires July 17, 2001

Date 5/27, 19 99, Signed Toledo Edison [Signature] Nuclear Safety & Inspection Manager
Repair Organization Authorized Representative Title

CERTIFICATE OF INSPECTION

I, the undersigned, holding a valid commission issued by the National Board of Boiler and Pressure Vessel Inspectors, and certificate of competency issued by the State or Province of Ohio and employed by Protection Mutual

of Park Ridge, Illinois have inspected the repair, modification, or replacement described in this report

on 6/8/99, 1999 and state that to the best of my knowledge and belief, this repair, modification, or replacement has been made or constructed in accordance with Section XI and Section III of the ASME Code and the National Board rules as defined in the publication NB-65 and NB-102, current editions.

By signing this certificate, neither the Inspector nor his employer makes any warranty, expressed or implied, concerning the repair, modification or replacement described in this report. Furthermore, neither the Inspector nor his employer shall be liable in any manner for any personal injury or property damage or a loss of any kind arising from or connected with this inspection.

Thomas S. Laps
Authorized Inspector's Signature

6/8/99
Date

Commissions NB 9330 "N" "I" "A" OHIO
National Board, State, Province and Endorsements COMM.

* Factory Mutual Engineering Association

* FACTORY MUTUAL INS. CO.

First Energy Nuclear Operating, 76 South Main St., Akron, Ohio 44308

Davis-Besse Nuclear Power Station Unit #1, 5501 N. St. Rt. 2, Oak Harbor, Ohio 43449

Commercial Service Date: 11/27/77

FORM NIS-2 / NR-1 / NVR-4 OWNER'S REPORT FOR REPAIRS OR REPLACEMENTS As Required by the Provisions of the ASME Code Section XI

1. Owner Toledo Edison Company Date 5/25/99
Name
300 Madison Ave., Toledo, OH 43652
Address
 2. Plant Davis-Besse Nuclear Power Station Unit # 1
Name
5501 N. SR 2, Oak Harbor, OH 43449 WO# 98-001213-000
Address Repair Organization P.O. No., Job No., etc.
 3. Work Performed By Toledo Edison Company Type Code Stamp NR
Name
5501 N. SR 2, Oak Harbor, OH 43449 Authorization NO. 20
Address Expiration Date 7-17-01

4. Identification of Systems Pressurizer Code Safety Valve RC13A (Subsys 064-04)

5. (a) Applicable Construction Code ASME III CL1 Edition 19 68 Addendum S70 Code Case 1444
 (b) Applicable Edition of Section XI Utilized for Repairs or Replacements 19 86
 (c) Design Responsibilities Crosby Valve & Gage Co., Toledo Edison Co.

6. Identification of Components Repaired or Replaced and Replacement Components

Name of Component	Name of Manufacturer	Manufacturer's Serial NO.	National Board NO.	Other Identification/Jurisdictional NO.	Year Built	Repaired Replaced or Replacement	ASME Code Stamped
Safety Valve	Crosby Valve	N54891000002	N/A	RC13A	1976	Replaced	Yes
Safety Valve	Crosby Valve	N54891000001	N/A	RC13A	1976	Replacement	Yes

7. Description of Work

Replaced existing safety valve with a like for like replacement valve. Replacement safety valve set pressure was tested and verified Toledo Edison Co. at Wyle Laboratory under MWO 3-98-4805-01, no rebuild was required. A VT-2 test was performed satisfactory after installation.

8. Tests Conducted: Hydrostatic (NO) Pneumatic (NO) Nominal Operating Pressure (YES) Other (NO)
 Pressure 2128 psi Test Temperature 535 °F

Pressure Relief Valves:
 Service N/A Size N/A
 Opening Pressure N/A Blowdown (if applicable) N/A
 Set Pressure and Blowdown Adjustments Made Using N/A
 at N/A (Test Medium)

NOTE: Supplemental sheets in the form of lists, sketches, or drawings may be used, provided (1) size is 8 1/2 x 11 in. (2) Information in items 1 through 6 on this report is included on each sheet, and (3) each sheet is numbered and the number of sheets is recorded at the top of this form.

First Energy Nuclear Operating, 76 South Main St., Akron, Ohio 44308

Davis-Besse Nuclear Power Station Unit #1, 5501 N. St. Rt. 2, Oak Harbor, Ohio 43449

Commercial Service Date: 11/27/77

FORM NIS-2 /NR-1 /NVR-4

8. REMARKS

Applicable Manufacturers Data Reports to be Attached

N/A

CERTIFICATE OF COMPLIANCE

We certify that the statements made in the report are correct and this replacement conforms to the rules of the ASME Code Section XI.

Type Code Symbol NR Certificate of Authorization No. 20 Expiration Date 7-17-01

Signed H.W. S. Lamm Nuclear Safety & Inspection Manager Date 5/27 1999
Owner or Owner's Designee, Title

CERTIFICATE OF INSERVICE INSPECTION

I, the undersigned, holding a valid commission issued by the National Board of Boiler and Pressure Vessel Inspectors in the State or Province of Ohio and employed by Protection Mutual of Park Ridge, Illinois have inspected the components described in this Owner's Report during the period 4/8/99 to 6/4/99, and state that to the best of my knowledge and belief, the Owner has performed examinations and taken corrective measures described in this Owner's Report in accordance with the requirements of the ASME Code, Section XI.

By signing this certificate, neither the Inspector nor his employer makes any warranty, expressed or implied, concerning the examinations and corrective measures described in the Owner's Report. Furthermore, neither the Inspector nor his employer shall be liable in any manner for any personal injury or property damage or a loss of any kind arising from or connected with this inspection.

Thomas S. Lamm 6/4/99 Commissions NB 9336 N I A OHIO
Inspector's Signature Date National Board, State, Province and Endorsements COMM.

CERTIFICATE OF COMPLIANCE

We certify that the statements made in this report are correct and the repair, modification, or replacement of the pressure relief device described above conforms to Section XI and Section III of the ASME Code and the National Board rules as defined in publications NB-65 and NB-102, current editions.

Certificate of Authorization No. 316 to use the "VR" stamp expires Dec. 13, 2001

Certificate of Authorization No. 20 to use the "NR" stamp expires July 17, 2001

Date _____, 19____, Signed Toledo Edison Nuclear Safety & Inspection Manager
Repair Organization Authorized Representative Title

CERTIFICATE OF INSPECTION

I, the undersigned, holding a valid commission issued by the National Board of Boiler and Pressure Vessel Inspectors, and certificate of competency issued by the State or Province of Ohio and employed by Protection Mutual of Park Ridge, Illinois have inspected the repair, modification, or replacement described in this report

on _____, 19____ and state that to the best of my knowledge and belief, this repair, modification, or replacement has been made or constructed in accordance with Section XI and Section III of the ASME Code and the National Board rules as defined in the publication NB-65 and NB-102, current editions.

By signing this certificate, neither the Inspector nor his employer makes any warranty, expressed or implied, concerning the repair, modification or replacement described in this report. Furthermore, neither the Inspector nor his employer shall be liable in any manner for any personal injury or property damage or a loss of any kind arising from or connected with this inspection.

Authorized Inspector's Signature _____ Date _____ Commissions _____
National Board, State, Province and Endorsements

* Factory Mutual Engineering Association

* Factory Mutual Ins. Co.

First Energy Nuclear Operating, 76 South Main St., Akron, Ohio 44308

Davis-Besse Nuclear Power Station Unit #1, 5501 N. St. Rt. 2, Oak Harbor, Ohio 43449

Commercial Service Date: 11/27/77

FORM NIS-2 / NR-1 / NVR-4 OWNER'S REPORT FOR REPAIRS OR REPLACEMENTS As Required by the Provisions of the ASME Code Section XI

1. Owner Toledo Edison Company Date 5/25/99
Name
300 Madison Ave., Toledo, OH 43652
Address
 Sheet 1 of 2

2. Plant Davis-Besse Nuclear Power Station Unit # 1
Name
5501 N. SR 2, Oak Harbor, OH 43449
Address
WO# 98-001214-000
Repair Organization P.O. No., Job No., etc.

3. Work Performed By Toledo Edison Company Type Code Stamp NR
Name
5501 N. SR 2, Oak Harbor, OH 43449
Address
 Authorization NO. 20
 Expiration Date 7-17-01

4. Identification of Systems Pressurizer Code Safety Valve RC13B (Subsys 064-04)

5. (a) Applicable Construction Code ASME III CL1 Edition 19 74 Addendum S74 Code Case N/A
 (b) Applicable Edition of Section XI Utilized for Repairs or Replacements 19 86
 (c) Design Responsibilities Crosby Valve & Gage Co., Toledo Edison Co.

6. Identification of Components Repaired or Replaced and Replacement Components

Name of Component	Name of Manufacturer	Manufacturer's Serial NO.	National Board NO.	Other Identification/Jurisdictional NO.	Year Built	Repaired Replaced or Replacement	ASME Code Stamped
Safety Valve	Crosby Valve	N59303000001	N/A	RC13B	1978	Replaced	Yes
Safety Valve	Crosby Valve	N56264000005	N/A	RC13B	1976	Replacement	Yes

7. Description of Work

Replaced existing safety valve with a like for like replacement valve. Replacement safety valve was rebuilt and tested by Toledo Edison Co. at Wyle Laboratory under MWO 3-98-4805-01. A VT-2 test was performed satisfactory after installation.

8. Tests Conducted: Hydrostatic (NO) Pneumatic (NO) Nominal Operating Pressure (YES) Other (NO)
 Pressure 2128 psi Test Temperature 536 °F

Pressure Relief Valves:
 Service N/A Size N/A
 Opening Pressure N/A Blowdown (if applicable) N/A
 Set Pressure and Blowdown Adjustments Made Using N/A
(Test Medium)
 at N/A

NOTE: Supplemental sheets in the form of lists, sketches, or drawings may be used, provided (1) size is 8 1/2 x 11 in. (2) Information in items 1 through 6 on this report is included on each sheet, and (3) each sheet is numbered and the number of sheets is recorded at the top of this form.

FORM NIS-2 /NR-1 /NVR-4		
8. REMARKS	Applicable Manufacturer Data Reports to be Attached	
NVR-Toledo Edison Co		
CERTIFICATE OF COMPLIANCE		
We certify that the statements made in the report are correct and this <u>replacement</u> conforms to the rules of the ASME Code Section XI.		
Type Code Symbol <u>NR</u>	Certificate of Authorization No. <u>20</u>	Expiration Date <u>7-17-01</u>
Signed <u>D. W. S. [Signature]</u> Nuclear Safety & Inspection Manager		Date <u>5/27</u> 19 <u>99</u>
<small>Owner or Owner's Designee, Title</small>		
CERTIFICATE OF INSERVICE INSPECTION		
I, the undersigned, holding a valid commission issued by the National Board of Boiler and Pressure Vessel Inspectors in the State or Province of <u>Ohio</u> and employed by <u>Protection Mutual</u> of <u>Park Ridge, Illinois</u> have inspected the components described in this Owner's Report during the period <u>4/8/99</u> to <u>6/3/99</u> , and state that to the best of my knowledge and belief, the Owner has performed examinations and taken corrective measures described in this Owner's Report in accordance with the requirements of the ASME Code, Section XI.		
By signing this certificate, neither the Inspector nor his employer makes any warranty, expressed or implied, concerning the examinations and corrective measures described in the Owner's Report. Furthermore, neither the Inspector nor his employer shall be liable in any manner for any personal injury or property damage or a loss of any kind arising from or connected with this inspection.		
Inspector's Signature <u>Thomas H. [Signature]</u>	Date <u>6/3/99</u>	Commissions <u>NB9330</u> "NIA" <u>OHIO COMM.</u>
<small>National Board, State, Province and Endorsements</small>		
CERTIFICATE OF COMPLIANCE		
We certify that the statements made in this report are correct and the repair, modification, or replacement of the pressure relief device described above conforms to Section XI and Section III of the ASME Code and the National Board rules as defined in publications NB-65 and NB-102, current editions.		
Certificate of Authorization No. <u>316</u> to use the "VR" stamp expires <u>Dec. 13</u> , <u>2001</u>		
Certificate of Authorization No. <u>20</u> to use the "NR" stamp expires <u>July 17</u> , <u>2001</u>		
Date _____, 19____, Signed <u>Toledo Edison</u>	Nuclear Safety & Inspection Manager	Title _____
<small>Repair Organization Authorized Representative</small>		
CERTIFICATE OF INSPECTION		
I, the undersigned, holding a valid commission issued by the National Board of Boiler and Pressure Vessel Inspectors, and certificate of competency issued by the State or Province of <u>Ohio</u> and employed by <u>Protection Mutual</u> of <u>Park Ridge, Illinois</u> have inspected the repair, modification, or replacement described in this report on _____, 19____ and state that to the best of my knowledge and belief, this repair, modification, or replacement has been made or constructed in accordance with Section XI and Section III of the ASME Code and the National Board rules as defined in the publication NB-65 and NB-102, current editions.		
By signing this certificate, neither the Inspector nor his employer makes any warranty, expressed or implied, concerning the repair, modification or replacement described in this report. Furthermore, neither the Inspector nor his employer shall be liable in any manner for any personal injury or property damage or a loss of any kind arising from or connected with this inspection.		
Authorized Inspector's Signature _____	Date _____	Commissions _____
<small>National Board, State, Province and Endorsements</small>		
* Factory Mutual Engineering Association		

First Energy Nuclear Operating, 76 South Main St., Akron, Ohio 44308

Davis-Besse Nuclear Power Station Unit #1, 5501 N. St. Rt. 2, Oak Harbor, Ohio 43449

Commercial Service Date: 11/27/77

FORM NIS-2 / NR-1 / NVR-1 OWNER'S REPORT FOR REPAIRS OR REPLACEMENTS As Required by the Provisions of the ASME Code Section XI

1. Owner Toledo Edison Company Date 5/27/99
300 Madison Ave., Toledo, OH 43652 Sheet 1 of 1
 2. Plant Davis-Besse Nuclear Power Station Unit # 1
5501 N. SR 2, Oak Harbor, OH 43449 WO# 98-001201-000 (3-98-4805-01)
 3. Work Performed By Toledo Edison Company Type Code Stamp NR / VR
5501 N. SR 2, Oak Harbor, OH 43449 Authorization NO. 20 / 316
 Expiration Date 7-17-01 / 12-13-01

4. Identification of Systems Pressurizer Code Safety Valve SPARE (Subsys 064-04)

5. (a) Applicable Construction Code ASME III CL1 Edition 19 71 Addendum W72 Code Case N/A
 (b) Applicable Edition of Section XI Utilized for Repairs or Replacements 19 86
 (c) Design Responsibilities Crosby Valve & Gage Co., Toledo Edison Co.

6. Identification of Components Repaired or Replaced and Replacement Components

Name of Component	Name of Manufacturer	Manufacturer's Serial NO.	National Board NO.	Other Identification/Jurisdictional NO.	Year Built	Repaired Replaced or Replacement	ASME Code Stamped
Safety Valve	Crosby Valve	N56264000005	N/A	Spare Valve	1976	Repaired	Yes

7. Description of Work

This is a dedicated spare pressurizer code safety valve. This valve was sent to Wyle Laboratories for testing and rebuild. As found set pressure test failed. Valve was removed from the test header, jack & lapped by Toledo Edison under there N.B.-NR/VR stamp using MWO 3-98-4805-01. After valve reassembly it was placed by on the test header and tested satisfactory.

8. Tests Conducted: Hydrostatic (NO) Pneumatic (NO) Nominal Operating Pressure (NO) Other (NO)
 Pressure N/A psi Test Temperature N/A °F

Pressure Relief Valves:
 Service Steam Size 4"x6"
 Opening Pressure 2512 PSIG Blowdown (if applicable) N/A
 Set Pressure and Blowdown Adjustments Made Using Steam
 at Wyle Laboratories, 7800 Highway 20 West, Huntsville, AL (Test Station)

NOTE: Supplemental sheets in the form of lists, sketches, or drawings may be used, provided (1) size is 8 1/2 x 11 in. (2) Information in items 1 through 6 on this report is included on each sheet, and (3) each sheet is numbered and the number of sheets is recorded at the top of this form.

First Energy Nuclear Operating, 76 South Main St., Akron, Ohio 44308

Davis-Besse Nuclear Power Station Unit #1, 5501 N. St. Rt. 2, Oak Harbor, Ohio 43449

Commercial Service Date: 11/27/77

FORM NIS-2 /NR-1 / NVR-1

9. REMARKS

Applicable Manufacturer Data Reports to be Attached

N/A

CERTIFICATE OF COMPLIANCE

We certify that the statements made in the report are correct and this replacement conforms to the rules of the ASME Code Section XI.

Type Code Symbol NR Certificate of Authorization No. 20 Expiration Date 7-17-01

Signed TH S. Laps Nuclear Safety & Inspection Manager Date 5/27 1999
Owner or Owner's Designee, Title

CERTIFICATE OF INSERVICE INSPECTION

I, the undersigned, holding a valid commission issued by the National Board of Boiler and Pressure Vessel Inspectors in the State or Province of Ohio and employed by Protection Mutual of Park Ridge, Illinois YEL 6/8/99 have inspected the components described in this Owner's Report during the period 10/21/98 to 6/8/99, and state that to the best of my knowledge and belief, the Owner has performed examinations and taken corrective measures 6/8/99 described in this Owner's Report in accordance with the requirements of the ASME Code, Section XI.

By signing this certificate, neither the Inspector nor his employer makes any warranty, expressed or implied, concerning the examinations and corrective measures described in the Owner's Report. Furthermore, neither the Inspector nor his employer shall be liable in any manner for any personal injury or property damage or a loss of any kind arising from or connected with this inspection.

Inspector's Signature Thomas S. Laps Date 6/8/99 Commissions NB 9330 "N" "I" "A" "OHIO COMM."
National Board, State, Province and Endorsements

CERTIFICATE OF COMPLIANCE

We certify that the statements made in this report are correct and the repair, modification, or replacement of the pressure relief device described above conforms to Section XI and Section III of the ASME Code and the National Board rules as defined in publications NB-65 and NB-102, current editions.

Certificate of Authorization No. 316 to use the "VR" stamp expires Dec. 13, 2001

Certificate of Authorization No. 20 to use the "NR" stamp expires July 17, 2001

Date 5/27, 1999 Signed Toledo Edison TH S. Laps Nuclear Safety & Inspection Manager
Repair Organization Authorized Representative Title

CERTIFICATE OF INSPECTION

I, the undersigned, holding a valid commission issued by the National Board of Boiler and Pressure Vessel Inspectors, and certificate of competency issued by the State or Province of Ohio and employed by Protection Mutual

of Park Ridge, Illinois have inspected the repair, modification, or replacement described in this report

on 6/8/99, 1999 and state that to the best of my knowledge and belief, this repair, modification, or replacement has been made or constructed in accordance with Section XI and Section III of the ASME Code and the National Board rules as defined in the publication NB-65 and NB-102, current editions.

By signing this certificate, neither the Inspector nor his employer makes any warranty, expressed or implied, concerning the repair, modification or replacement described in this report. Furthermore, neither the Inspector nor his employer shall be liable in any manner for any personal injury or property damage or a loss of any kind arising from or connected with this inspection.

Authorized Inspector's Signature Thomas S. Laps Date 6/8/99 Commissions NB 9330 "N" "I" "A" "OHIO COMM."
National Board, State, Province and Endorsements

* Factory Mutual Engineering Association

* FACTORY MUTUAL INS. CO.

First Energy Nuclear Operating, 76 South Main St., Akron, Ohio 44308

Davis-Besse Nuclear Power Station Unit #1, 5501 N. St. Rt. 2, Oak Harbor, Ohio 43449

Commercial Service Date: 11/27/77

FORM NIS-2 / NR-1 / NVR-4 OWNER'S REPORT FOR REPAIRS OR REPLACEMENTS As Required by the Provisions of the ASME Code Section XI

1. Owner Toledo Edison Company Date 5/7/00
300 Madison Ave., Toledo, OH 43652 Sheet 1 of 1
Name Address
2. Plant Davis-Besse Nuclear Power Station Unit #1
5501 N. SR 2, Oak Harbor, OH 43449 WO 88-001710-000
Name Address Repair Organization P.O. No., Job No., etc.
3. Work Performed By Toledo Edison Company Type Code Stamp NR
5501 N. SR 2, Oak Harbor, OH 43449 Authorization NO. 20
Name Address Expiration Date 7-17-01
4. Identification of Systems CTMT Vacuum Relief ISO Butterfly Valve CV5076 (Subsys 060-01)
5. (a) Applicable Construction Code ASME III CL2 Edition 19 71 Addendum N/A Code Case N/A
 (b) Applicable Edition of Section XI Utilized for Repairs or Replacements 19 86
 (c) Design Responsibilities Valtek Inc. & Toledo Edison Co.

6. Identification of Components Repaired or Replaced and Replacement Components

Name of Component	Name of Manufacturer	Manufacturer's Serial NO.	National Board NO.	Other Identification/Jurisdictional NO.	Year Built	Repaired Replaced or Replacement	ASME Code Stamped
8" Control Valve	Fisher Controls Co.	5110734	N/A	CV 5076	1974	Replaced	Yes
8" Control Valve	Valtek Inc.	77385.001.6	N/A	CV 5076	1991	Replacement	Yes

7. Description of Work

Existing 8" butterfly control valve (Fisher Controls Co.) S/N 5110734 was removed and replaced with a 8" butterfly control valve (Valtek Inc.) S/N 77385.001.6. After installation, a functional test was performed satisfactory.

8. Tests Conducted: Hydrostatic (NO) Pneumatic (NO) Nominal Operating Pressure (YES) Other (NO)
 Pressure 38.5 psi Test Temperature NOT °F
- Pressure Relief Valves:
 Service N/A Size N/A
 Opening Pressure N/A Blowdown (if applicable) N/A
 Set Pressure and Blowdown Adjustments Made Using N/A
 at N/A (Test Station)

NOTE: Supplemental sheets in the form of lists, sketches, or drawings may be used, provided (1) size is 8 1/2 x 11 in. (2) information in items 1 through 6 on this report is included on each sheet, and (3) each sheet is numbered and the number of sheets is recorded at the top of this form.

First Energy Nuclear Operating, 76 South Main St., Akron, Ohio 44308

Davis-Besse Nuclear Power Station Unit #1, 5501 N. St. Rt. 2, Oak Harbor, Ohio 43449

Commercial Service Date: 11/27/77

FORM NIS-2 / NR-1 / NVR-1	
9. REMARKS	Applicable Manufacturers Data Reports to be Attached
Valtek Inc.	
CERTIFICATE OF COMPLIANCE	
We certify that the statements made in the report are correct and this <u>replacement</u> conforms to the rules of the ASME Code Section XI.	
Type Code Symbol <u>NR</u> Certificate of Authorization No. <u>20</u> Expiration Date <u>7-17-01</u>	
Signed <u>[Signature]</u> Nuclear Safety & Inspection Manager Date <u>5/7/2000</u>	
Owner or Owner's Designee, Title	
CERTIFICATE OF INSERVICE INSPECTION	
I, the undersigned, holding a valid commission issued by the National Board of Boiler and Pressure Vessel Inspectors in the State or Province of <u>Ohio</u> and employed by <u>Factory Mutual Ins. Co.</u> of <u>Johnston, R.I.</u> have inspected the components described in this Owner's Report during the period <u>3-20-00</u> to <u>5-7-00</u> , and state that to the best of my knowledge and belief, the Owner has performed examinations and taken corrective measures described in this Owner's Report in accordance with the requirements of the ASME Code, Section XI.	
By signing this certificate, neither the Inspector nor his employer makes any warranty, expressed or implied, concerning the examinations and corrective measures described in the Owner's Report. Furthermore, neither the Inspector nor his employer shall be liable in any manner for any personal injury or property damage or a loss of any kind arising from or connected with this inspection.	
Inspector's Signature <u>[Signature]</u> Date <u>5-7-00</u>	Commissions <u>NB9901</u> <u>N I A</u> <u>Ohio</u> National Board, State, Province and Endorsements
CERTIFICATE OF COMPLIANCE	
We certify that the statements made in this report are correct and the repair, modification, or replacement of the pressure relief device described above conforms to Section XI and Section III of the ASME Code and the National Board rules as defined in publications NB-65 and NB-102, current editions.	
Certificate of Authorization No. <u>316</u> to use the "VR" stamp expires <u>Dec. 13</u> , <u>2001</u> .	
Certificate of Authorization No. <u>20</u> to use the "NR" stamp expires <u>July 17</u> , <u>2001</u> .	
Date <u> </u> , 19 <u> </u> , Signed <u>Toledo Edison</u>	Nuclear Safety & Inspection Manager
Repair Organization	Authorized Representative
CERTIFICATE OF INSPECTION	
I, the undersigned, holding a valid commission issued by the National Board of Boiler and Pressure Vessel Inspectors, and certificate of competency issued by the State or Province of <u>Ohio</u> and employed by <u>Factory Mutual Ins. Co.</u> of <u>Johnston, R.I.</u> have inspected the repair, modification, or replacement described in this report	
on <u> </u> , 19 <u> </u> and state that to the best of my knowledge and belief, this repair, modification, or replacement has been made or constructed in accordance with Section XI and Section III of the ASME Code and the National Board rules as defined in the publication NB-65 and NB-102, current editions.	
By signing this certificate, neither the Inspector nor his employer makes any warranty, expressed or implied, concerning the repair, modification or replacement described in this report. Furthermore, neither the Inspector nor his employer shall be liable in any manner for any personal injury or property damage or a loss of any kind arising from or connected with this inspection.	
Authorized Inspector's Signature <u> </u> Date <u> </u>	Commissions <u> </u> National Board, State, Province and Endorsements
* Factory Mutual Ins. Co	

First Energy Nuclear Operating, 76 South Main St., Akron, Ohio 44308

Davis-Besse Nuclear Power Station Unit #1, 5501 N. St. Rt. 2, Oak Harbor, Ohio 43449

Commercial Service Date: 11/27/77

FORM NPV-1 MANUFACTURERS' DATA REPORT FOR NUCLEAR PUMPS OR VALVES*
(As Required by the Provisions of the ASME Code, Section III, Div. 1)

- 1 Manufactured by VALTEK INC., Mtn. Springs Pkwy., Springville, UT 84663
(Name and Address of Manufacturer)
2 Manufactured for Toledo Edison, Oak Harbor, Ohio
(Name and Address of Purchaser or Owner)
3 Location of Installation Davis-Besse Nuclear Power Station, 5501 N. State, Rt. 2, Oak Harbor, Ohio, 43449
(Name and Address)
4 Pump or Valve Control Valve Nominal Inlet Size 8 inch Outlet Size 8 inch
(Inch) (Inch)

(a) Model No. Series No. or Type	(b) Manufacturer Serial No.	(c) Canadian Registration No.	(d) Drawing No.	(e) Class	(f) Nat'l. Std. No.	(g) Year Built
(1) <u>Valdisk</u>	<u>77385-001-6</u>	<u>N/A</u>	<u>A77385-001</u>	<u>2</u>	<u>N/A</u>	<u>1991</u>
(2) <u>rev. 2</u>						
(3)						
(4)						
(5)						
(6)						
(7)						
(8)						
(9)						
(10)						

5. ISOLATION SERVICE

(Brief description of service for which equipment was designed)

6. Design Conditions _____ psi _____ °F or Valve Pressure Class 150 (1)
(Pressure) (Temperature)
7. Cold Working Pressure 450 psi at 100°F
8. Pressure Retaining Pieces

Mark No.	Material Spec. No.	Manufacturer	Remarks
(a) Castings			HEAT CODE
<u>V11830</u>	<u>SA 216-WCB</u>	<u>American Foundry</u>	<u>EXW</u>
<u>V11799</u>	<u>SA 351-CF8M</u>	<u>American Foundry</u>	<u>LOG</u>
(b) Forgings			

(1) For manually operated valves only.

* Supplemental sheets in form of lists, sketches or drawings may be used provided (1) size is 8-1/2" x 11", (2) information in Items 1, 2 and 5 on this data report is included on each sheet and (3) each sheet is numbered and tabular sheets are recorded at top of this form.

(3/77)

This form (EP0037) may be obtained from the Order Dept., ASME, 345 E. 47th St., New York, N.Y. 10017.

Davis-Besse Nuclear Power Station Unit #1, 5501 N. St. Rt. 2, Oak Harbor, Ohio 43449

Commercial Service Date: 11/27/77

3. Hydrostatic test 150 psi

We certify that the statements made in this report are correct and that this pump, or valve, conforms to the rules of construction of the ASME Code for Nuclear Power Plant Components, Section III, Div. 1, Edition 1986.
Addenda None, Code Case No. N/A, Date 15 Aug 91.
Signed Valtek by D. Marcar
(Manufacturer)
Our ASME Certificate or Authorization No. 1884 to use the N symbol expires 14 Oct 92.
(N (PSV)) (Date)

Design information on file at Valtek Inc., PO Box 2200, Mtn. Springs Pkwy., Springville
Stress analysis report (Class 1 only) on file at Valtek, Inc. Utah 84663

Design specifications certified by (1) Mark A. Harris
PE State Ohio Reg. No. E-053281

Stress analysis certified by (1) Fred M. Cain
PE State Utah Reg. No. 7080

(1) Signature not required. List name only.

1. the undersigned, holding a valid commission issued by the National Board of Boiler and Pressure Vessel Inspectors and the State or Province of Utah and employed by H.S.T. & I. Co. of Hartford, Connecticut have inspected the pump, or valve, described in this Data Report on 14 Aug 1991 and state that to the best of my knowledge and belief, the Manufacturer has constructed this pump, or valve, in accordance with the ASME Code, Section III.

By signing this certificate, neither the Inspector nor his employer makes any warranty, expressed or implied, concerning the equipment described in this Data Report. Furthermore, neither the Inspector nor his employer shall be liable in any manner for any personal injury or property damage or a loss of any kind arising from or connected with this inspection.

Date 8/15 1991 CHS Commission 11491 Ohio Comm.
(Inspector) Charles E. Smith

First Energy Nuclear Operating, 76 South Main St., Akron, Ohio 44308

Davis-Besse Nuclear Power Station Unit #1, 5501 N. St. Rt. 2, Oak Harbor, Ohio 43449

Commercial Service Date: 11/27/77

FORM NIS-2 / NR-1 / NVR-1 OWNER'S REPORT FOR REPAIRS OR REPLACEMENTS As Required by the Provisions of the ASME Code Section XI

1. Owner Toledo Edison Company Date 5/2/00
Name
300 Madison Ave., Toledo, OH 43652
Address Sheet 1 of 1
2. Plant Davis-Besse Nuclear Power Station Unit # 1
Name
5501 N. SR 2, Oak Harbor, OH 43449 WO 99-000269-000, 003, & 004
Address Repair Organization P.O. No., Job No., etc.
3. Work Performed By Toledo Edison Company Type Code Stamp NR
Name
5501 N. SR 2, Oak Harbor, OH 43449 Authorization NO. 20
Address Expiration Date 7-17-01
4. Identification of Systems RC Letdown Cooler 1-1 Inlet ISO Valve MU1A (Subsys 065-01)
5. (a) Applicable Construction Code ASME III CL1 Edition 19 88 Addendum N/A Code Case N/A
 (b) Applicable Edition of Section XI Utilized for Repairs or Replacements 19 86
 (c) Design Responsibilities Velan Engineering Co, Toledo Edison Co

6. Identification of Components Repaired or Replaced and Replacement Components

Name of Component	Name of Manufacturer	Manufacturer's Serial NO.	National Board NO.	Other Identification/Jurisdictional NO.	Year Built	Repaired Replaced or Replacement	ASME Code Stamped
Valve Bonnet	Velan Engineering Co	HT Code 62719	N/A	MU1A	1972	Replaced	No
Valve Bonnet	Velan Engineering Co	13783	N/A	MU1A	1999	Replacement	No
8 - 5/8" Studs	Nova Machine Prod.	HT# 71217	N/A	MU1A	1999	Replacement	No
		<i>8574 712917</i>					

7. Description of Work

Removed existing valve bonnet heat code 62719 and replaced it with a like for like bonnet serial # 13783. Replaced existing body to bonnet studs.
 Removed bonnet leak off pipe, installed a 1/8" threaded pipe plug and a seal welded performed.
 Valve was reassembled and a VT-2 leakage test was performed satisfactory.

8. Tests Conducted: Hydrostatic (NO) Pneumatic (NO) Nominal Operating Pressure (YES) Other (NO)
 Pressure 2128 psi Test Temperature NOT °F
- Pressure Relief Valves:
 Service N/A Size N/A
 Opening Pressure N/A Blowdown (if applicable) N/A
 Set Pressure and Blowdown Adjustments Made Using N/A
 at N/A (Test Medium)

NOTE: Supplemental sheets in the form of lists, sketches, or drawings may be used, provided (1) size is 8 1/2 x 11 in. (2) information in items 1 through 6 on this report is included on each sheet, and (3) each sheet is numbered and the number of sheets is recorded at the top of this form.

First Energy Nuclear Operating, 76 South Main St., Akron, Ohio 44308

Davis-Besse Nuclear Power Station Unit #1, 5501 N. St. Rt. 2, Oak Harbor, Ohio 43449

Commercial Service Date: 11/27/77

FORM NIS-2 /NR-1 /NVR-4

8. REMARKS

Applicable Manufacturers Data Reports to be Attached

N/A

CERTIFICATE OF COMPLIANCE

We certify that the statements made in the report are correct and this replacement

conforms to the rules of

Type Code Symbol NRCertificate of Authorization No. 20Expiration Date 7-17-01Signed A. W. Stanton, Jr.

Nuclear Safety & Inspection Manager

Date 5/3/00 18

Owner or Owner's Designee, Title

CERTIFICATE OF INSERVICE INSPECTION

I, the undersigned, holding a valid commission issued by the National Board of Boiler and Pressure Vessel Inspectors in the State or Province of Ohio and employed by Factory Mutual Ins. Co. of Johnston, R.I. have inspected the components described in this Owner's Report during the period 4/16/99 to 5/3/00, and state that to the best of my knowledge and belief, the Owner has performed examinations and taken corrective measures described in this Owner's Report in accordance with the requirements of the ASME Code, Section XI.

By signing this certificate, neither the Inspector nor his employer makes any warranty, expressed or implied, concerning the examinations and corrective measures described in the Owner's Report. Furthermore, neither the Inspector nor his employer shall be liable in any manner for any personal injury or property damage or a loss of any kind arising from or connected with this inspection.

Inspector's Signature

Thomas J. Lape

Date

5/4/00

Commissions

NB4330 NIA OHIO COMM.

National Board, State, Province and Endorsements

CERTIFICATE OF COMPLIANCE

We certify that the statements made in this report are correct and the repair, modification, or replacement of the pressure relief device described above conforms to Section XI and Section III of the ASME Code and the National Board rules as defined in publications NB-85 and NB-102, current editions.

Certificate of Authorization No. 316 to use the "VR" stamp expires Dec. 13, 2001.Certificate of Authorization No. 20 to use the "NR" stamp expires July 17, 2001.Date _____, 19____, Signed Toledo Edison

Repair Organization

Authorized Representative

Nuclear Safety & Inspection Manager
Title

CERTIFICATE OF INSPECTION

I, the undersigned, holding a valid commission issued by the National Board of Boiler and Pressure Vessel Inspectors, and certificate of competency issued by the State or Province of Ohio and employed by Factory Mutual Ins. Co.*

of Johnston, R.I. have inspected the repair, modification, or replacement described in this report

on _____, 19____, and state that to the best of my knowledge and belief, this repair, modification, or replacement has been made or constructed in accordance with Section XI and Section III of the ASME Code and the National Board rules as defined in the publication NB-65 and NB-102, current editions.

By signing this certificate, neither the Inspector nor his employer makes any warranty, expressed or implied, concerning the repair, modification or replacement described in this report. Furthermore, neither the Inspector nor his employer shall be liable in any manner for any personal injury or property damage or a loss of any kind arising from or connected with this inspection.

Authorized Inspector's Signature

Date

Commissions

National Board, State, Province and Endorsements

* Factory Mutual Ins. Co

First Energy Nuclear Operating, 76 South Main St., Akron, Ohio 44308

Davis-Besse Nuclear Power Station Unit #1, 5501 N. St. Rt. 2, Oak Harbor, Ohio 43449

Commercial Service Date: 11/27/77

FORM NIS-2 / NR-1 / NVR-1 OWNER'S REPORT FOR REPAIRS OR REPLACEMENTS As Required by the Provisions of the ASME Code Section XI

1. Owner Toledo Edison Company Date 5/18/00
300 Madison Ave., Toledo, OH 43652 Sheet 1 of 1
 2. Plant Davis-Besse Nuclear Power Station Unit # 1
5501 N. SR 2, Oak Harbor, OH 43449 WO 99-000403-000
 3. Work Performed By Toledo Edison Company Type Code Stamp NR
5501 N. SR 2, Oak Harbor, OH 43449 Authorization NO. 20
 Expiration Date 7-17-01

4. Identification of Systems CFT 1-2 Discharge Isolation Valve CF1A (Subsys 051-01)

5. (a) Applicable Construction Code ASME III C12 Edition 19 68 Addendum N/A Code Case N/A
 (b) Applicable Edition of Section XI Utilized for Repairs or Replacements 19 86
 (c) Design Responsibilities Velan Engineering Co.

6. Identification of Components Repaired or Replaced and Replacement Components

Name of Component	Name of Manufacturer	Manufacturer's Serial NO.	National Board NO.	Other Identification/Jurisdictional NO.	Year Built	Repaired Replaced or Replacement	ASME Code Stamped
Valve	Velan Engineering Co.	33809	N/A	CF1A	1972	Repaired	Yes
Valve Stem	Velan Engineering Co.	HT# 45808	N/A	N/A	1972	Replacement	No
Valve Stem	Velan Engineering Co.	HT#28608	N/A	N/A	1981	Replacement	No

7. Description of Work

Valve CF1A was disassembled and existing valve stem HT# 45808 was removed and replaced with a like for like replacement valve stem HT# 28608.
 After valve was reassembled and pressurized, verified no leakage from body to bonnet leakage.

8. Tests Conducted: Hydrostatic (NO) Pneumatic (NO) Nominal Operating Pressure (YES) Other (NO)
 Pressure NOP psi Test Temperature Not °F

Pressure Relief Valves:
 Service N/A Size N/A
 Opening Pressure N/A Blowdown (if applicable): N/A
 Set Pressure and Blowdown Adjustments Made Using N/A
 at N/A

NOTE: Supplemental sheets in the form of lists, sketches, or drawings may be used, provided (1) size is 8 1/2 x 11 in. (2) Information in items 1 through 6 on this report is included on each sheet, and (3) each sheet is numbered and the number of sheets is recorded at the top of this form.

First Energy Nuclear Operating, 76 South Main St., Akron, Ohio 44308

Davis-Besse Nuclear Power Station Unit #1, 5501 N. St. Rt. 2, Oak Harbor, Ohio 43449

Commercial Service Date: 11/27/77

FORM NIS-2 /NR-1 /NVR-1

8. REMARKS

Applicable Manufacturers Data Reports to be Attached

NA

CERTIFICATE OF COMPLIANCE

We certify that the statements made in the report are correct and this replacement conforms to the rules of the ASME Code Section XI.

Type Code Symbol NR Certificate of Authorization No. 20 Expiration Date 7-17-01

Signed R. W. Starnes Nuclear Safety & Inspection Manager Date 5/19/00

Owner or Owner's Designee, Title

CERTIFICATE OF INSERVICE INSPECTION

I, the undersigned, holding a valid commission issued by the National Board of Boiler and Pressure Vessel Inspectors in the State or Province of Ohio and employed by Factory Mutual Ins. Co. of Johnston, R.I. have inspected the components described in this Owner's Report during the period 3/13/00 to 5/23/00, and state that to the best of my knowledge and belief, the Owner has performed examinations and taken corrective measures described in this Owner's Report in accordance with the requirements of the ASME Code, Section XI.

By signing this certificate, neither the Inspector nor his employer makes any warranty, expressed or implied, concerning the examinations and corrective measures described in the Owner's Report. Furthermore, neither the Inspector nor his employer shall be liable in any manner for any personal injury or property damage or a loss of any kind arising from or connected with this inspection.

Thomas H. Laps
Inspector's Signature

ANTI 5/23/00
Date

Commissions NB9336 "NIA" OHIO
National Board, State, Province and Endorsements COMM.

CERTIFICATE OF COMPLIANCE

We certify that the statements made in this report are correct and the repair, modification, or replacement of the pressure relief device described above conforms to Section XI and Section III of the ASME Code and the National Board rules as defined in publications NB-65 and NB-102, current editions.

Certificate of Authorization No. 316 to use the "VR" stamp expires Dec. 13, 2001

Certificate of Authorization No. 20 to use the "NR" stamp expires July 17, 2001

Date _____, 19____, Signed Toledo Edison Nuclear Safety & Inspection Manager
Repair Organization Authorized Representative Title

CERTIFICATE OF INSPECTION

I, the undersigned, holding a valid commission issued by the National Board of Boiler and Pressure Vessel Inspectors, and certificate of competency issued by the State or Province of Ohio and employed by Factory Mutual Ins. Co. of Johnston, R.I. have inspected the repair, modification, or replacement described in this report

on _____, 19____ and state that to the best of my knowledge and belief, this repair, modification, or replacement has been made or constructed in accordance with Section XI and Section III of the ASME Code and the National Board rules as defined in the publication NB-65 and NB-102, current editions.

By signing this certificate, neither the Inspector nor his employer makes any warranty, expressed or implied, concerning the repair, modification or replacement described in this report. Furthermore, neither the Inspector nor his employer shall be liable in any manner for any personal injury or property damage or a loss of any kind arising from or connected with this inspection.

Authorized Inspector's Signature _____ Date _____

Commissions _____
National Board, State, Province and Endorsements

* Factory Mutual Ins. Co

First Energy Nuclear Operating, 76 South Main St., Akron, Ohio 44308

Davis-Besse Nuclear Power Station Unit #1, 5501 N. St. Rt. 2, Oak Harbor, Ohio 43449

Commercial Service Date: 11/27/77

FORM NIS-2 / NR-1 / NVR-4 OWNER'S REPORT FOR REPAIRS OR REPLACEMENTS As Required by the Provisions of the ASME Code Section XI

1. Owner Toledo Edison Company Date 5/18/99
Name
300 Madison Ave., Toledo, OH 43652
Address
 Sheet 1 of 4

2. Plant Davis-Besse Nuclear Power Station Unit #1
Name
5501 N. SR 2, Oak Harbor, OH 43449
Address
WO 99-001162-000
Repair Organization P.O. No., Job No., etc.

3. Work Performed By Toledo Edison Company Type Code Stamp NR
Name
5501 N. SR 2, Oak Harbor, OH 43449
Address
 Authorization NO. 20
 Expiration Date 7-17-01

4. Identification of Systems Main Steam Line #1 (Subsys 083-01)

5. (a) Applicable Construction Code ASME III CL2 Edition 19 68 Addendum W68 Code Case N/A
 (b) Applicable Edition of Section XI Utilized for Repairs or Replacements 19 86
 (c) Design Responsibilities Toledo Edison Co, Dresser Industries Inc., Nova Machine Prod. Inc.

6. Identification of Components Repaired or Replaced and Replacement Components

Name of Component	Name of Manufacturer	Manufacturer's Serial NO.	National Board NO.	Other Identification/Jurisdictional NO.	Year Built	Repaired or Replacement	ASME Code Stamped
Safety Valve	Dresser	BM 8635	N/A	SP17B4	1968	Replaced	Yes
Safety Valve	Dresser	BM 8633	N/A	SP17B4	1968	Replacement	Yes
Safety Valve	Dresser	BM 8640	N/A	SP17B6	1968	Replaced	Yes
Safety Valve	Dresser	BM 8639	N/A	SP17B6	1968	Replacement	Yes
Safety Valve	Dresser	BS 06255	N/A	SP17B7	1968	Repaired	Yes
* Stud/Nuts	UNK	UNK	N/A	SP17B7	Unk	Replaced	No

7. Description of Work

* During removal of safety valve SP17B4 one stud and two nuts on the inlet flange was replaced. Replacement stud was 1 3/8" - 8

SA 193 Gr B7 HT # 8094962 supplied by Nova Machine Product, and replacement nuts were 1 3/8" SA 194 Gr 2H, HT # AMH, supplied by Hamanaka Nut Mfg.

Safety valves SP17 B4 and B6 were repaired by NWS Technologies, prior to installation. These valves are like for like replacements.

Safety valve SP17B7 was removed, sent to Dresser Industries for rebuild, tested satisfactory at Wyle Laboratories, returned and reinstalled back in its original location.

A satisfactory VT-2 inservice test was performed on all three valves after installation.

8. Tests Conducted: Hydrostatic (NO) Pneumatic (NO) Nominal Operating Pressure (YES) Other (NO)
 Pressure 1205 psi Test Temperature NOT °F

Pressure Relief Valves:
 Service N/A Size N/A
 Opening Pressure N/A Blowdown (if applicable) N/A
 Set Pressure and Blowdown Adjustments Made Using N/A
 at N/A (Test Medium)

NOTE: Supplemental sheets in the form of lists, sketches, or drawings may be used, provided (1) size is 8 1/2 x 11 in. (2) information in items 1 through 6 on this report is included on each sheet, and (3) each sheet is numbered and the number of sheets is recorded at the top of this form.

12 RFO Inservice Inspection Summary

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First Energy Nuclear Operating, 76 South Main St., Akron, Ohio 44308

Davis-Besse Nuclear Power Station Unit #1, 5501 N. St. Rt. 2, Oak Harbor, Ohio 43449

Commercial Service Date: 11/27/77

FORM NIS-2 /NR-1 /NVR-4

B. REMARKS

Applicable Manufacturer Data Reports to be Attached

NVR's from NWS Technologies for Safety valves SP17 B4 serial # BM 8633 and SP17B6 serial # BM 8639.

NVR from Dresser Ind. for Safety valve SP17B7 serial # BS 86255.

CERTIFICATE OF COMPLIANCE

We certify that the statements made in the report are correct and this replacement conforms to the rules of the ASME Code Section XI.

Type Code Symbol NR Certificate of Authorization No. 20 Expiration Date 7-17-01Signed J.W. Storm Nuclear Safety & Inspection Manager Date 5/18 19 99
Owner or Owner's Designee, Title

CERTIFICATE OF INSERVICE INSPECTION

I, the undersigned, holding a valid commission issued by the National Board of Boiler and Pressure Vessel Inspectors in the State or Province of Ohio and employed by Protection Mutual of Park Ridge, Illinois have inspected the components described in this Owner's Report during the period 4/22/99 to 5/20/99, and state that to the best of my knowledge and belief, the Owner has performed examinations and taken corrective measures described in this Owner's Report in accordance with the requirements of the ASME Code, Section XI.

By signing this certificate, neither the Inspector nor his employer makes any warranty, expressed or implied, concerning the examinations and corrective measures described in the Owner's Report. Furthermore, neither the Inspector nor his employer shall be liable in any manner for any personal injury or property damage or a loss of any kind arising from or connected with this inspection.

Inspector's Signature Thomas J. LapeDate 5/20/99Commissions NB 9330 N I A OHIO
National Board, State, Province and Endorsements COMM

CERTIFICATE OF COMPLIANCE

We certify that the statements made in this report are correct and the repair, modification, or replacement of the pressure relief device described above conforms to Section XI and Section III of the ASME Code and the National Board rules as defined in publications NB-65 and NB-102, current editions.

Certificate of Authorization No. 316 to use the "VR" stamp expires Dec. 13 2001Certificate of Authorization No. 20 to use the "NR" stamp expires July 17 2001Date _____, 19____, Signed Toledo Edison Nuclear Safety & Inspection Manager
Repair Organization Authorized Representative Title

CERTIFICATE OF INSPECTION

I, the undersigned, holding a valid commission issued by the National Board of Boiler and Pressure Vessel Inspectors, and

certificate of competency issued by the State or Province of Ohio and employed by Protection Mutualof Park Ridge, Illinois have inspected the repair, modification, or replacement described in this report

on _____, 19____ and state that to the best of my knowledge and belief, this repair, modification, or replacement has been made or constructed in accordance with Section XI and Section III of the ASME Code and the National Board rules as defined in the publication NB-65 and NB-102, current editions.

By signing this certificate, neither the Inspector nor his employer makes any warranty, expressed or implied, concerning the repair, modification or replacement described in this report. Furthermore, neither the Inspector nor his employer shall be liable in any manner for any personal injury or property damage or a loss of any kind arising from or connected with this inspection.

Authorized Inspector's Signature

Date

Commissions

National Board, State, Province and Endorsements

* Factory Mutual Engineering Association

* Factory Mutual Ins. Co.

First Energy Nuclear Operating, 76 South Main St., Akron, Ohio 44308
Davis-Besse Nuclear Power Station Unit #1, 5501 N. St. Rt. 2, Oak Harbor, Ohio 43449
Commercial Service Date: 11/27/77

FORM NVR-1 REPORT OF REPAIR ☒ REPLACEMENT ☐
OF NUCLEAR PRESSURE RELIEF DEVICES

1. Work performed by: NWS Technologies, LLC Purchase Order # S 083319 D98
131 Venture Boulevard, Spartanburg, SC 29301

2. Work performed for: Centerior Energy, Davis-Besse Power Station

3/4. Owner - name, address and identification of nuclear power plant: Davis-Besse Power Station
5501 North State Route 2 Oak harbor, OH 43449-9760

5. a: Repaired pressure relief device: Main Steam Safety Valve
b: Name of manufacturer: Dresser - Consolidated
c: Identifying nos. 3707RA-RT21 BM8633 N/A steam 6 x 8 1972
(type) (mfr's S/N) (NB#) (service) (size) (yr.built)

d: Construction Code: ASME Sec. III Div. 1 1968 W68 N/A 2
(name/section/division) (edition) (addenda) (Code Cases(s)) (Code Class)

6. ASME Code Section XI applicable for inservice inspection: 1986 N/A N/A
(edition) (addenda) (Code Case(s))

7. ASME Code Section XI used for repairs, replacements: 1986 N/A N/A
(edition) (addenda) (Code Case(s))

8. Construction Code used for repairs, replacements: 1968 W68 N/A
(edition) (addenda) (Code Case(s))

9. Design responsibilities: N/A

10. Opening pressure: 1100 psig
Set-pressure adjustment made at: NWS Technologies, LLC using steam

11. Description of work (include name and identifying number of replacement parts): No parts replaced, disassembled, lapped seats, inspected, assembled. Certified set-pressure and seat tightness on steam.

12. Remarks N/A

CERTIFICATE OF COMPLIANCE

I, Cesar V. Sierra certify that the statements made in this report are correct and the repair or replacement of the pressure relief devices described above conforms to Section XI of the ASME Code and the National Board Inspection Code "VR" and "NR" rules.

National Board Certificate of Authorization No. 632 to use the "VR" stamp expires April 3, 2000.

National Board Certificate of Authorization No. 81 to use the "NR" stamp expires April 9, 2000.

4/27/98 NWS Technologies, LLC Manager, QA
Date (repair organization) (authorized representative) (title)

CERTIFICATE OF INSPECTION

I, Carl R. Enos holding a valid commission issued by The National Board of Boiler and Pressure Vessel Inspectors and certificate of competency issued by the jurisdiction of TD and employed by Hartford Steam Boiler Inspection & Insurance Co., Hartford, CT have inspected the repair or replacement described in this report on 4/28/98 and state that to the best of my

knowledge and belief, this repair or replacement has been completed in accordance with Section XI of the ASME Code and the National Board Inspection Code "VR" and "NR" rules.

By signing this certificate, neither the undersigned nor my employer makes any warranty, expressed or implied, concerning this repair or replacement described in this report. Furthermore, neither the undersigned nor my employer shall be liable in any manner for any personal injury, property damage or loss of any kind arising from or connected with this inspection.

4/28/98 Date Carl R. Enos Inspector's Signature NB 8460, A, N, I, TN 2236 Ohio Commissions (NB (incl endorsements), jurisdiction, & no.)

First Energy Nuclear Operating, 76 South Main St., Akron, Ohio 44308

Davis-Besse Nuclear Power Station Unit #1, 5501 N. St. Rt. 2, Oak Harbor, Ohio 43449

Commercial Service Date: 11/27/77

FORM NVR-1 REPORT OF REPAIR ☒ REPLACEMENT ☒ **OF NUCLEAR PRESSURE RELIEF DEVICES**

1. Work performed by: <u>NWS Technologies, LLC</u>		Purchase Order # <u>S 083319 D98</u>	
<u>131 Venture Boulevard, Spartanburg, SC 29301</u>			
2. Work performed for: <u>Centerior Energy, Davis-Besse Power Station</u>			
3/4. Owner - name, address and identification of nuclear power plant: <u>Davis-Besse Power Station</u>			
<u>5501 North State Route 2 Oak harbor, OH 43449-9760</u>			
5. a: Repaired pressure relief device: <u>Main Steam Safety Valve</u>			
b: Name of manufacturer: <u>Dresser - Consolidated</u>			
c: Identifying nos.	<u>3777QA-RT21</u>	<u>BM8639</u>	<u>N/A</u>
	(type)	(mfr's S/N)	(NB#)
		<u>1968</u>	<u>W68</u>
		(edition)	(addenda)
d: Construction Code:	<u>SME Sec. III Div. 1</u>	<u>1968</u>	<u>W68</u>
	(name/section/division)	(edition)	(addenda)
			<u>N/A</u>
			(Code Cases(s))
			<u>2</u>
			(Code Class)
6. ASME Code Section XI applicable for inservice inspection: <u>1986</u>			
(edition)			
<u>N/A</u>			
(addenda)			
(Code Case(s))			
7. ASME Code Section XI used for repairs, replacements: <u>1986</u>			
(edition)			
<u>N/A</u>			
(addenda)			
(Code Case(s))			
8. Construction Code used for repairs, replacements: <u>1968</u>			
(edition)			
<u>W68</u>			
(addenda)			
(Code Case(s))			
9. Design responsibilities: <u>N/A</u>			
10. Opening pressure: <u>1050 psig</u>			
Set-pressure adjustment made at: <u>NWS Technologies, LLC</u> using <u>steam</u>			
11. Description of work (include name and identifying number of replacement parts): <u>Valve disassembled, inspected,</u>			
<u>replaced disc/lift stop/pins, lapped seats, assembled. Certified set-pressure and seat tightness on steam.</u>			
12. Remarks <u>Disc S/N: AAS24, Lift Stop S/N: 8-9143, Cotter pins S/N: N/A</u>			

CERTIFICATE OF COMPLIANCE

I, Cesar V. Sierra certify that the statements made in this report are correct and the repair or replacement of the pressure relief devices described above conforms to Section XI of the ASME Code and the National Board Inspection Code "VR" and "NR" rules.

National Board Certificate of Authorization No. 632 to use the "VR" stamp expires April 3, 2000.

National Board Certificate of Authorization No. 81 to use the "NR" stamp expires April 9, 2000.

4/27/98 NWS Technologies, LLC [Signature] Manager, QA
 Date (repair organization) (authorized representative) (title)

CERTIFICATE OF INSPECTION

I, Carl R. Enos holding a valid commission issued by The National Board of Boiler and Pressure Vessel Inspectors and certificate of competency issued by the jurisdiction of TN and employed by Hartford Steam Boiler Inspection & Insurance Co., Hartford, CT have inspected the repair or replacement described in this report on 4/28/98 and state that to the best of my knowledge and belief, this repair or replacement has been completed in accordance with Section XI of the ASME Code and the National Board Inspection Code "VR" and "NR" rules.

By signing this certificate, neither the undersigned nor my employer makes any warranty, expressed or implied, concerning this repair or replacement described in this report. Furthermore, neither the undersigned nor my employer shall be liable in any manner for any personal injury, property damage or loss of any kind arising from or connected with this inspection.

4/28/98 [Signature] NB 8460, A, N, I, TN 2236 Ohio
 Date Inspector's Signature Commissions (NB (incl endorsements), jurisdiction, & no.)

Commercial Service Date: 11/27/77

CERTIFICATE OF INSPECTION
I, _____, certify that to the best of my knowledge and belief the statements made in this report are correct and the repair, modification or replacement of the pressure relief devices described above conforms to Section XI of the ASME Code and the National Board Inspection Code "VR" and "NR" rules.
70, 95, 97-118,
National Board Certificate of Authorization No. 120-124 to use the "VR" stamp expires 6-29-2001
National Board Certificate of Authorization No. 45, 47-61 to use the "NR" stamp expires 6-30-1999
Date 5-4-99 Same as Item 1 above. Signed Jerry W. Bane Quality Engineer
(name of certificate holder) (authorized representative) (title)

CERTIFICATE OF INSPECTION
I, BL. Dasher, holding a valid commission issued by The National Board of Boiler and Pressure Vessel Inspectors and certificate of competency issued by the jurisdiction of LOUISIANA and employed by Hartford Steam Boiler Inspection and Insurance Co. of Hartford, CT have inspected the repair, modification or replacement described in this report on 5/4/99 and state that to the best of my knowledge and belief, this repair, modification or replacement has been completed in accordance with Section XI of the ASME Code and the National Board Inspection Code "VR" and "NR" rules.
By signing this certificate, neither the undersigned nor my employer makes any warranty, expressed or implied, concerning the repair modification or replacement described in this report. Furthermore, neither the undersigned nor my employer shall be liable in any manner for any personal injury, property damage or loss of any kind arising from or connected with this inspection.
Date 5/4/99 Signed [Signature] Commissions NB7779 "A" & "L" 11664
(National Board (and endorsements), and jurisdiction, and no.)

First Energy Nuclear Operating, 76 South Main St., Akron, Ohio 44308

Davis-Besse Nuclear Power Station Unit #1, 5501 N. St. Rt. 2, Oak Harbor, Ohio 43449

Commercial Service Date: 11/27/77

FORM NIS-2 / NR-1 / NVR-4 OWNER'S REPORT FOR REPAIRS OR REPLACEMENTS As Required by the Provisions of the ASME Code Section XI

1. Owner Toledo Edison Company Date 4/30/00
Name
300 Madison Ave., Toledo, OH 43652
Address

2. Plant Davis-Besse Nuclear Power Station Unit #1
Name
5501 N. SR 2, Oak Harbor, OH 43449
Address WO 99-001365-000 & 99-003279-002
Repair Organization P.O. No., Job No., etc.

3. Work Performed By Toledo Edison Company Type Code Stamp NR
Name
5501 N. SR 2, Oak Harbor, OH 43449
Address Authorization NO. 20
 Expiration Date 7-17-01

4. Identification of Systems Main Steam System EBB-1-SR4 C-224 (Subsys 083-01)

5. (a) Applicable Construction Code ANSI B31.1 Edition 19 67 Addendum N/A Code Case N/A
 (b) Applicable Edition of Section XI Utilized for Repairs or Replacements 19 86
 (c) Design Responsibilities Lisega Co, Toledo Edison Co.

6. Identification of Components Repaired or Replaced and Replacement Components

Name of Component	Name of Manufacturer	Manufacturer's Serial NO.	National Board NO.	Other Identification/Jurisdictional NO.	Year Built	Repaired Replaced or Replacement	ASME Code Stamped
Load Pins	Toledo Edison Co	N/A	N/A	C-224	2000	Replacement	No

7. Description of Work

Toledo Edison fabricated two new load pins (1 support end and 1 pipe end) per work order 99-001365-000 and drawing C-610 for snubber C-224.
 Load pins were installed in snubber C-224 per work order 99-003279-002. Pins were replaced due to vibration wear.

8. Tests Conducted: Hydrostatic (NO) Pneumatic (NO) Nominal Operating Pressure (NO) Other (NO)
 Pressure N/A psi Test Temperature N/A °F

Pressure Relief Valves:
 Service N/A Size N/A
 Opening Pressure N/A Blowdown (if applicable) N/A
 Set Pressure and Blowdown Adjustments Made Using N/A
 at N/A (Foot Shading)

NOTE: Supplemental sheets in the form of lists, sketches, or drawings may be used, provided (1) size is 8 1/2 x 11 in. (2) Information in items 1 through 6 on this report is included on each sheet, and (3) each sheet is numbered and the number of sheets is recorded at the top of this form.

First Energy Nuclear Operating, 76 South Main St., Akron, Ohio 44308

Davis-Besse Nuclear Power Station Unit #1, 5501 N. St. Rt. 2, Oak Harbor, Ohio 43449

Commercial Service Date: 11/27/77

FORM NIS-2 /NR-1 /NVR-4

8. REMARKS

Applicable Manufacturers Data Reports to be Attached

NA

CERTIFICATE OF COMPLIANCE

We certify that the statements made in the report are correct and this replacement

conforms to the rules of

the ASME Code Section XI.

Type Code Symbol NRCertificate of Authorization No. 20Expiration Date 7-17-01Signed A.W. Starnes

Nuclear Safety & Inspection Manager

Date 5/3/2000

Owner or Owner's Designee, Title

CERTIFICATE OF INSERVICE INSPECTION

I, the undersigned, holding a valid commission issued by the National Board of Boiler and Pressure Vessel Inspectors in the State or Province of Ohio and employed by Factory Mutual Ins. Co. of Johnston, R.I. have inspected the components described in this Owner's Report during the period 4/25/00 to 5/3/00, and state that to the best of my knowledge and belief, the Owner has performed examinations and taken corrective measures described in this Owner's Report in accordance with the requirements of the ASME Code, Section XI.

By signing this certificate, neither the Inspector nor his employer makes any warranty, expressed or implied, concerning the examinations and corrective measures described in the Owner's Report. Furthermore, neither the Inspector nor his employer shall be liable in any manner for any personal injury or property damage or a loss of any kind arising from or connected with this inspection.

Inspector's Signature Thomas E. LapsDate 5/3/00Commissions NB4330 N I A "OHIO COMM."
National Board, State, Province and Endorsements

CERTIFICATE OF COMPLIANCE

We certify that the statements made in this report are correct and the repair, modification, or replacement of the pressure relief device described above conforms to Section XI and Section III of the ASME Code and the National Board rules as defined in publications NB-65 and NB-102, current editions.

Certificate of Authorization No. 316 to use the "VR" stamp expires Dec. 13, 2001.Certificate of Authorization No. 20 to use the "NR" stamp expires July 17, 2001.Date 19, Signed Toledo Edison

Repair Organization

Authorized Representative

Nuclear Safety &
Inspection Manager
Title

CERTIFICATE OF INSPECTION

I, the undersigned, holding a valid commission issued by the National Board of Boiler and Pressure Vessel Inspectors, and

certificate of competency issued by the State or Province of Ohio and employed by Factory Mutual Ins. Co.of Johnston, R.I. have inspected the repair, modification, or replacement described in this report

on 5/3/00, 1900 and state that to the best of my knowledge and belief, this repair, modification, or replacement has been made or constructed in accordance with Section XI and Section III of the ASME Code and the National Board rules as defined in the publication NB-65 and NB-102, current editions.

By signing this certificate, neither the Inspector nor his employer makes any warranty, expressed or implied, concerning the repair, modification or replacement described in this report. Furthermore, neither the Inspector nor his employer shall be liable in any manner for any personal injury or property damage or a loss of any kind arising from or connected with this inspection.

Authorized Inspector's Signature

Date

Commissions

National Board, State, Province and Endorsements

* Factory Mutual Ins. Co

First Energy Nuclear Operating, 76 South Main St., Akron, Ohio 44308

Davis-Besse Nuclear Power Station Unit #1, 5501 N. St. Rt. 2, Oak Harbor, Ohio 43449

Commercial Service Date: 11/27/77

FORM NIS-2 / NR-1 / NVR-1 OWNER'S REPORT FOR REPAIRS OR REPLACEMENTS As Required by the Provisions of the ASME Code Section XI

1. Owner Toledo Edison Company Date 4/30/00
Name
300 Madison Ave., Toledo, OH 43652
Address Sheet 1 of 1

2. Plant Davis-Besse Nuclear Power Station Unit # 1
Name
5501 N. SR 2, Oak Harbor, OH 43449 WO 89-001365-000 & 89-003339-002
Address Repair Organization P.O. No., Job No., etc.

3. Work Performed By Toledo Edison Company Type Code Stamp NB
Name
5501 N. SR 2, Oak Harbor, OH 43449 Authorization NO. 20
Address Expiration Date 7-17-01

4. Identification of Systems Main Steam System EBB-1-SR4 C-225 (Subsys 083-01)

5. (a) Applicable Construction Code ANSI B31.1 Edition 19 87 Addendum N/A Code Case N/A
 (b) Applicable Edition of Section XI Utilized for Repairs or Replacements 19 86
 (c) Design Responsibilities Liseaga Co, Toledo Edison Co.

6. Identification of Components Repaired or Replaced and Replacement Components

Name of Component	Name of Manufacturer	Manufacturer's Serial NO.	National Board NO.	Other Identification/Jurisdictional NO.	Year Built	Repaired Replaced or Replacement	ASME Code Stamped
Load Pins	Toledo Edison Co	N/A	N/A	C-225	2000	Replacement	No

7. Description of Work

Toledo Edison fabricated two new load pins (1 support end and 1 pipe end) per work order 89-001365-000 and drawing C-610 for snubber C-225.
 Load pins were installed in snubber C-225 per work order 89-003339-002. Pins were replaced due to vibration wear.

8. Tests Conducted: Hydrostatic (NO) Pneumatic (NO) Nominal Operating Pressure (NO) Other (NO)
 Pressure N/A psi Test Temperature N/A °F

Pressure Relief Valves:
 Service N/A Size N/A
 Opening Pressure N/A Blowdown (if applicable) N/A
 Set Pressure and Blowdown Adjustments Made Using N/A
 at N/A (Test Medium)

NOTE: Supplemental sheets in the form of lists, sketches, or drawings may be used, provided (1) size is 8 1/2 x 11 in.
 (2) Information in items 1 through 6 on this report is included on each sheet, and (3) each sheet is numbered and the number of sheets is recorded at the top of this form.

First Energy Nuclear Operating, 76 South Main St., Akron, Ohio 44308

Davis-Besse Nuclear Power Station Unit #1, 5501 N. St. Rt. 2, Oak Harbor, Ohio 43449

Commercial Service Date: 11/27/77

FORM NIS-2 /NR-1 /NVR-4

8. REMARKS

Applicable Manufacturers Data Reports to be Attached

NA

CERTIFICATE OF COMPLIANCE

We certify that the statements made in the report are correct and this replacement conforms to the rules of the ASME Code Section XI.

Type Code Symbol NR Certificate of Authorization No. 20 Expiration Date 7-17-01

Signed D.W. Starnes Jr. Nuclear Safety & Inspection Manager Date 5/3/00 1999
Owner or Owner's Designee, Title

CERTIFICATE OF INSERVICE INSPECTION

I, the undersigned, holding a valid commission issued by the National Board of Boiler and Pressure Vessel Inspectors in the State or Province of Ohio and employed by Factory Mutual Ins. Co. of Johnston, R.I. have inspected the components described in this Owner's Report during the period 4/25/00 to 5/3/00, and state that to the best of my knowledge and belief, the Owner has performed examinations and taken corrective measures described in this Owner's Report in accordance with the requirements of the ASME Code, Section XI.

By signing this certificate, neither the Inspector nor his employer makes any warranty, expressed or implied, concerning the examinations and corrective measures described in the Owner's Report. Furthermore, neither the Inspector nor his employer shall be liable in any manner for any personal injury or property damage or a loss of any kind arising from or connected with this inspection.

Inspector's Signature Thomas A. Page Date 5/3/00 Commissions NB 9336 N I A Ohio
National Board, State, Province and Endorsements

CERTIFICATE OF COMPLIANCE

We certify that the statements made in this report are correct and the repair, modification, or replacement of the pressure relief device described above conforms to Section XI and Section III of the ASME Code and the National Board rules as defined in publications NB-65 and NB-102, current editions.

Certificate of Authorization No. 316 to use the "VR" stamp expires Dec. 13, 2001.

Certificate of Authorization No. 20 to use the "NR" stamp expires July 17, 2001.

Date _____, 19____, Signed Toledo Edison Nuclear Safety & Inspection Manager
Repair Organization Authorized Representative Title

CERTIFICATE OF INSPECTION

I, the undersigned, holding a valid commission issued by the National Board of Boiler and Pressure Vessel Inspectors, and certificate of competency issued by the State or Province of Ohio and employed by Factory Mutual Ins. Co.* of Johnston, R.I. have inspected the repair, modification, or replacement described in this report

on _____, 19____ and state that to the best of my knowledge and belief, this repair, modification, or replacement has been made or constructed in accordance with Section XI and Section III of the ASME Code and the National Board rules as defined in the publication NB-65 and NB-102, current editions.

By signing this certificate, neither the Inspector nor his employer makes any warranty, expressed or implied, concerning the repair, modification or replacement described in this report. Furthermore, neither the Inspector nor his employer shall be liable in any manner for any personal injury or property damage or a loss of any kind arising from or connected with this inspection.

Authorized Inspector's Signature _____ Date _____ Commissions _____
National Board, State, Province and Endorsements

* Factory Mutual Ins. Co

First Energy Nuclear Operating, 76 South Main St., Akron, Ohio 44308

Davis-Besse Nuclear Power Station Unit #1, 5501 N. St. Rt. 2, Oak Harbor, Ohio 43449

Commercial Service Date: 11/27/77

FORM NIS-2 / NR-1 / NVR-4 OWNER'S REPORT FOR REPAIRS OR REPLACEMENTS

As Required by the Provisions of the ASME Code Section XI

1. Owner Toledo Edison Company Date 5/17/00
Name
300 Madison Ave., Toledo, OH 43652
Address

2. Plant Davis-Besse Nuclear Power Station Unit # 1
Name
5501 N. SR 2, Oak Harbor, OH 43449
Address WO 99-001791-001
Repair Organization P.O. No., Job No., etc.

3. Work Performed By Toledo Edison Company Type Code Stamp NR
Name
5501 N. SR 2, Oak Harbor, OH 43449
Address Authorization NO. 20
 Expiration Date 7-17-01

4. Identification of Systems HPI Line 2-1 Stop Check Valve HP57 (Subsys 052-01)

5. (a) Applicable Construction Code ASME III CL1 Edition 19 71 Addendum W 72 Code Case N/A
 (b) Applicable Edition of Section XI Utilized for Repairs or Replacements 19 86
 (c) Design Responsibilities Velan Engineering Co.

6. Identification of Components Repaired or Replaced and Replacement Components

Name of Component	Name of Manufacturer	Manufacturer's Serial NO.	National Board NO.	Other Identification/Jurisdictional NO.	Year Built	Repaired Replaced or Replacement	ASME Code Stamped
Stop Check Valve	Velan Engineering Co.	Unknown	N/A	HP 57	UNK	Repaired	Yes
5/8"- 11 Hex Nut	Nova Machine Prod.	HT# 711930	N/A	HP 57	1998	Replacement	No

7. Description of Work

Replaced one body to bonnet heavy hex nut due to wrong material installed. After installation, a VT-2 was performed satisfactory.

8. Tests Conducted: Hydrostatic (NO) Pneumatic (NO) Nominal Operating Pressure (YES) Other (NO)
 Pressure 2270 psi Test Temperature NOT °F

Pressure Relief Valves:
 Service N/A Size N/A
 Opening Pressure N/A Blowdown (if applicable)- N/A
 Set Pressure and Blowdown Adjustments Made Using N/A
 at N/A (Test Medium)

NOTE: Supplemental sheets in the form of lists, sketches, or drawings may be used, provided (1) size is 8 1/2 x 11 in. (2) Information in items 1 through 6 on this report is included on each sheet, and (3) each sheet is numbered and the number of sheets is recorded at the top of this form.

First Energy Nuclear Operating, 76 South Main St., Akron, Ohio 44308

Davis-Besse Nuclear Power Station Unit #1, 5501 N. St. Rt. 2, Oak Harbor, Ohio 43449

Commercial Service Date: 11/27/77

FORM NIS-2 /NR-1 /NVR-1

B. REMARKS

Applicable Manufacturers Data Reports to be Attached

NA

CERTIFICATE OF COMPLIANCE

We certify that the statements made in the report are correct and this replacement conforms to the rules of the ASME Code Section XI.

Type Code Symbol NR Certificate of Authorization No. 20 Expiration Date 7-17-01

Signed R. W. Starnes Jr. Nuclear Safety & Inspection Manager Date 5/18/00
Owner or Owner's Designee, Title

CERTIFICATE OF INSERVICE INSPECTION

I, the undersigned, holding a valid commission issued by the National Board of Boiler and Pressure Vessel Inspectors in the State or Province of Ohio and employed by Factory Mutual Ins. Co. of Johnston, R.I. have inspected the components described in this Owner's Report during the period 5/10/00 to 5/19/00, and state that to the best of my knowledge and belief, the Owner has performed examinations and taken corrective measures described in this Owner's Report in accordance with the requirements of the ASME Code, Section XI.

By signing this certificate, neither the Inspector nor his employer makes any warranty, expressed or implied, concerning the examinations and corrective measures described in the Owner's Report. Furthermore, neither the Inspector nor his employer shall be liable in any manner for any personal injury or property damage or a loss of any kind arising from or connected with this inspection.

Thomas H. Sege
Inspector's Signature

5/19/00
Date

Commissions NB 9330 NIA OHIO
National Board, State, Province and Endorsements

CERTIFICATE OF COMPLIANCE

We certify that the statements made in this report are correct and the repair, modification, or replacement of the pressure relief device described above conforms to Section XI and Section III of the ASME Code and the National Board rules as defined in publications NB-65 and NB-102, current editions.

Certificate of Authorization No. 316 to use the "VR" stamp expires Dec. 13, 2001

Certificate of Authorization No. 20 to use the "NR" stamp expires July 17, 2001

Date , 19 , Signed Toledo Edison
Repair Organization Authorized Representative Title

Nuclear Safety &
Inspection Manager

CERTIFICATE OF INSPECTION

I, the undersigned, holding a valid commission issued by the National Board of Boiler and Pressure Vessel Inspectors, and

certificate of competency issued by the State or Province of Ohio and employed by Factory Mutual Ins. Co.

of Johnston, R.I. have inspected the repair, modification, or replacement described in this report

on , 19 and state that to the best of my knowledge and belief, this repair, modification, or replacement has been made or constructed in accordance with Section XI and Section III of the ASME Code and the National Board rules as defined in the publication NB-65 and NB-102, current editions.

By signing this certificate, neither the Inspector nor his employer makes any warranty, expressed or implied, concerning the repair, modification or replacement described in this report. Furthermore, neither the Inspector nor his employer shall be liable in any manner for any personal injury or property damage or a loss of any kind arising from or connected with this inspection.

Authorized Inspector's Signature

Date

Commissions
National Board, State, Province and Endorsements

* Factory Mutual Ins. Co

First Energy Nuclear Operating, 76 South Main St., Akron, Ohio 44308

Davis-Besse Nuclear Power Station Unit #1, 5501 N. St. Rt. 2, Oak Harbor, Ohio 43449

Commercial Service Date: 11/27/77

FORM NIS-2 / NR-1 / NVR-1 OWNER'S REPORT FOR REPAIRS OR REPLACEMENTS As Required by the Provisions of the ASME Code Section XI

1. Owner Toledo Edison Company Date 5/12/99
Name
300 Madison Ave., Toledo, OH 43652
Address
2. Plant Davis-Besse Nuclear Power Station Unit #1
Name
5501 N. SR 2, Oak Harbor, OH 43449
Address W.O. 99-002640-000&99-002763-007
Repair Organization P.O. No., Job No., etc.
3. Work Performed By Toledo Edison Company Type Code Stamp NR
Name
5501 N. SR 2, Oak Harbor, OH 43449
Address Authorization NO. 20
 Expiration Date 7-17-01
4. Identification of Systems RC-33 LOOP 2-1 COLD LEG DRAIN STOP VALVE
5. (a) Applicable Construction Code ASME III CL1 Edition 19 72 Addendum W72 Code Case 1335-6
 (b) Applicable Edition of Section XI Utilized for Repairs or Replacements 19 86
 (c) Design Responsibilities VELAN

6. Identification of Components Repaired or Replaced and Replacement Components

Name of Component	Name of Manufacturer	Manufacturer's Serial NO.	National Board NO.	Other Identification/Jurisdictional NO.	Year Built	Repaired Replaced or Replacement	ASME Code Stamped
STUD 5/8" 11UNC	VELAN	NA	NA	NA	NA	REPLACED	No
STUD 5/8" 11UNC	NOVA	NA	NA	HT#715200	NA	REPLACEMENT	No
NUT 5/8" UNC 6BH	UNK	NA	NA	NA	NA	REPLACED	No
NUT 5/8" UNC 8M	NOVA	NA	NA	HT#42642	NA	REPLACEMENT	No

7. Description of Work

Replaced one broken body to bonnet stud with a like-for-like SA-564 ty 630 H1100, HT#715200. Replaced two wrong body to bonnet nuts, grade 6BH with the right type, SA194 Grade 8m, H#42642. Performed a VT-2 Leakage test after installation.

8. Tests Conducted: Hydrostatic (NO) Pneumatic (NO) Nominal Operating Pressure (YES) Other (NO)
 Pressure 2128 psi Test Temperature 636 °F
- Pressure Relief Valves:
 Service NA Size NA
 Opening Pressure NA Blowdown (if applicable) NA
 Set Pressure and Blowdown Adjustments Made Using NA
 at NA (Test Medium)

NOTE: Supplemental sheets in the form of lists, sketches, or drawings may be used, provided (1) size is 8 1/2 x 11 in. (2) Information in items 1 through 6 on this report is included on each sheet, and (3) each sheet is numbered and the number of sheets is recorded at the top of this form.

FORM NIS-2 /NR-1 /NVR-1

9. REMARKS

Applicable Manufacturer Data Reports to be Attached

N/A

CERTIFICATE OF COMPLIANCE

We certify that the statements made in the report are correct and this replacement conforms to the rules of the ASME Code Section XI.

Type Code Symbol NR Certificate of Authorization No. 20 Expiration Date 7-17-01

Signed A.W. Starnes Nuclear Safety & Inspection Manager Date 5/11 19 99
Owner or Owner's Designee, Title

CERTIFICATE OF INSERVICE INSPECTION

I, the undersigned, holding a valid commission issued by the National Board of Boiler and Pressure Vessel Inspectors in the State or Province of Ohio and employed by Protection Mutual of Park Ridge, Illinois have inspected the components described in this Owner's Report during the period 5/6/99 to 5/18/99, and state that to the best of my knowledge and belief, the Owner has performed examinations and taken corrective measures described in this Owner's Report in accordance with the requirements of the ASME Code, Section XI.

By signing this certificate, neither the Inspector nor his employer makes any warranty, expressed or implied, concerning the examinations and corrective measures described in the Owner's Report. Furthermore, neither the Inspector nor his employer shall be liable in any manner for any personal injury or property damage or a loss of any kind arising from or connected with this inspection.

Inspector's Signature Thomas H. Page Date 5/20/99 Commissions NB 9330 NIA OHIO COMM.
National Board, State, Province and Endorsements

CERTIFICATE OF COMPLIANCE

We certify that the statements made in this report are correct and the repair, modification, or replacement of the pressure relief device described above conforms to Section XI and Section III of the ASME Code and the National Board rules as defined in publications NB-65 and NB-102, current editions.

Certificate of Authorization No. 316 to use the "VR" stamp expires Dec. 13, 2001

Certificate of Authorization No. 20 to use the "NR" stamp expires July 17, 2001

Date _____, 19____, Signed Toledo Edison Nuclear Safety & Inspection Manager
Repair Organization Authorized Representative Title

CERTIFICATE OF INSPECTION

I, the undersigned, holding a valid commission issued by the National Board of Boiler and Pressure Vessel Inspectors, and certificate of competency issued by the State or Province of Ohio and employed by Protection Mutual

of Park Ridge, Illinois have inspected the repair, modification, or replacement described in this report

on _____, 19____ and state that to the best of my knowledge and belief, this repair, modification, or replacement has been made or constructed in accordance with Section XI and Section III of the ASME Code and the National Board rules as defined in the publication NB-65 and NB-102, current editions.

By signing this certificate, neither the Inspector nor his employer makes any warranty, expressed or implied, concerning the repair, modification or replacement described in this report. Furthermore, neither the Inspector nor his employer shall be liable in any manner for any personal injury or property damage or a loss of any kind arising from or connected with this inspection.

Authorized Inspector's Signature _____ Date _____ Commissions _____
National Board, State, Province and Endorsements

* Factory Mutual Engineering Association

* Factory Mutual Ins. Co.

First Energy Nuclear Operating, 76 South Main St., Akron, Ohio 44308

Davis-Besse Nuclear Power Station Unit #1, 5501 N. St. Rt. 2, Oak Harbor, Ohio 43449

Commercial Service Date: 11/27/77

FORM NIS-2 / NR-1 / NVR-1 OWNER'S REPORT FOR REPAIRS OR REPLACEMENTS As Required by the Provisions of the ASME Code Section XI

1. Owner Toledo Edison Company Date 5/13/99
Name
300 Madison Ave., Toledo, OH 43652
Address
 Sheet 1 of 1

2. Plant Davis-Besse Nuclear Power Station Unit #1
Name
5501 N. SR 2, Oak Harbor, OH 43449
Address
W.O. 99-002705-000
Repair Organization P.O. No., Job No., etc.

3. Work Performed By Toledo Edison Company Type Code Stamp NR
Name
5501 N. SR 2, Oak Harbor, OH 43449
Address
 Authorization NO. 20
 Expiration Date 7-17-01

4. Identification of Systems RC-38 LOOP 1-1 COLD LEG DRAIN ISOLATION VALVE

5. (a) Applicable Construction Code ASME III CL1 Edition 19 72 Addendum W72 Code Case 1335-6
 (b) Applicable Edition of Section XI Utilized for Repairs or Replacements 19 86
 (c) Design Responsibilities VELAN

6. Identification of Components Repaired or Replaced and Replacement Components

Name of Component	Name of Manufacturer	Manufacturer's Serial NO.	National Board NO.	Other Identification/Jurisdictional NO.	Year Built	Repaired or Replacement	ASME Code Stamped
5/8" 11UNC	HASKELL	NA	NA	NA	NA	REPLACED	NA
5/8" 11UNC	NOVA	NA	NA	NA	NA	REPLACEMENT	NA

7. Description of Work

Disassembled RC-38 to replaced the yoke. Replaced one body to bonnet nut SA-194 Grade 8M with a like-for-like nut. Reassembled valve and performed a VT-2 at completion.

8. Tests Conducted: Hydrostatic (NO) Pneumatic (NO) Nominal Operating Pressure (YES) Other (NO)
 Pressure 2128 psi Test Temperature 536 °F

Pressure Relief Valves:
 Service NA Size NA
 Opening Pressure NA Blowdown (if applicable) NA
 Set Pressure and Blowdown Adjustments Made Using NA
 at NA (Test Medium)

NOTE: Supplemental sheets in the form of lists, sketches, or drawings may be used, provided (1) size is 8 1/2 x 11 in. (2) Information in items 1 through 6 on this report is included on each sheet, and (3) each sheet is numbered and the number of sheets is recorded at the top of this form.

First Energy Nuclear Operating, 76 South Main St., Akron, Ohio 44308

Davis-Besse Nuclear Power Station Unit #1, 5501 N. St. Rt. 2, Oak Harbor, Ohio 43449

Commercial Service Date: 11/27/77

FORM NIS-2 /NR-1 /NVR-1

8. REMARKS

Applicable Manufacturer Data Reports to be Attached

N/A

CERTIFICATE OF COMPLIANCE

We certify that the statements made in the report are correct and this replacement conforms to the rules of the ASME Code Section XI.

Type Code Symbol NR Certificate of Authorization No. 20 Expiration Date 7-17-01

Signed St. W. Slattery, Jr. Nuclear Safety & Inspection Manager Date 5/17 1999

Owner or Owner's Designee, Title

CERTIFICATE OF INSERVICE INSPECTION

I, the undersigned, holding a valid commission issued by the National Board of Boiler and Pressure Vessel Inspectors in the State or Province of Ohio and employed by Protection Mutual of Park Ridge, Illinois have inspected the components described in this Owner's Report during the period 5/4/99 to 5/10/99, and state that to the best of my knowledge and belief, the Owner has performed examinations and taken corrective measures described in this Owner's Report in accordance with the requirements of the ASME Code, Section XI.

By signing this certificate, neither the Inspector nor his employer makes any warranty, expressed or implied, concerning the examinations and corrective measures described in the Owner's Report. Furthermore, neither the Inspector nor his employer shall be liable in any manner for any personal injury or property damage or a loss of any kind arising from or connected with this inspection.

Inspector's Signature Thomas G. LapeDate 5/10/99

Commissions NB 9330 N I A OHIO
National Board, State, Province and Endorsements COM.

CERTIFICATE OF COMPLIANCE

We certify that the statements made in this report are correct and the repair, modification, or replacement of the pressure relief device described above conforms to Section XI and Section III of the ASME Code and the National Board rules as defined in publications NB-65 and NB-102, current editions.

Certificate of Authorization No. 316 to use the "VR" stamp expires Dec. 13, 2001

Certificate of Authorization No. 20 to use the "NR" stamp expires July 17, 2001

Date _____, 19____, Signed Toledo Edison Nuclear Safety & Inspection Manager
Repair Organization Authorized Representative Title

CERTIFICATE OF INSPECTION

I, the undersigned, holding a valid commission issued by the National Board of Boiler and Pressure Vessel Inspectors, and certificate of competency issued by the State or Province of Ohio and employed by Protection Mutual

of Park Ridge, Illinois have inspected the repair, modification, or replacement described in this report

on _____, 19____ and state that to the best of my knowledge and belief, this repair, modification, or replacement has been made or constructed in accordance with Section XI and Section III of the ASME Code and the National Board rules as defined in the publication NB-65 and NB-102, current editions.

By signing this certificate, neither the Inspector nor his employer makes any warranty, expressed or implied, concerning the repair, modification or replacement described in this report. Furthermore, neither the Inspector nor his employer shall be liable in any manner for any personal injury or property damage or a loss of any kind arising from or connected with this inspection.

Authorized Inspector's Signature

Date

Commissions _____
National Board, State, Province and Endorsements

* Factory Mutual Engineering Association

* FACTORY MUTUAL INS. CO.

First Energy Nuclear Operating, 76 South Main St., Akron, Ohio 44308

Davis-Besse Nuclear Power Station Unit #1, 5501 N. St. Rt. 2, Oak Harbor, Ohio 43449

Commercial Service Date: 11/27/77

FORM NIS-2 / NR-1 / NVR-4 OWNER'S REPORT FOR REPAIRS OR REPLACEMENTS As Required by the Provisions of the ASME Code Section XI

1. Owner Toledo Edison Company Date 06/06/2000
Name
300 Madison Ave., Toledo, OH 43652
Address
 Sheet 1 of 1

2. Plant Davis-Besse Nuclear Power Station Unit # 1
Name
5501 N. SR 2, Oak Harbor, OH 43449
Address
WO # 99-002965-000
Repair Organization P.O. No., Job No., etc.

3. Work Performed By Toledo Edison Company Type Code Stamp NR
Name
5501 N. SR 2, Oak Harbor, OH 43449
Address
 Authorization NO. 20
 Expiration Date 7-17-01

4. Identification of Systems Reactor Coolant / RC 262 Pressurizer Spray Valve Bypass (SUB064-04)

5. (a) Applicable Construction Code ASME III CL1 Edition 19 72 Addendum W72 Code Case 1335-6
 (b) Applicable Edition of Section XI Utilized for Repairs or Replacements 19 85
 (c) Design Responsibilities Nova Machine Products Corp. / Toledo Edison Company

6. Identification of Components Repaired or Replaced and Replacement Components

Name of Component	Name of Manufacturer	Manufacturer's Serial NO.	National Board NO.	Other Identification/Jurisdictional NO.	Year Built	Repaired Replaced or Replacement	ASME Code Stamped
Stud	Nova	HT # 715200	NA	NA	NA	Replacement	NA

7. Description of Work

Removed one (1) existing stud and replaced with new like for like replacement.

Stud, 5/8" - 11 UNC 2A X 3" Continuous Thread, Thread to Thread, ASME SA-564, P.O. # 7008181, ASME III, NB Class 1, 86 Ed. No ADD.

After installation, a VT-2 Inservice Test was performed satisfactorily.

8. Tests Conducted: Hydrostatic (NO) Pneumatic (NO) Nominal Operating Pressure (YES) Other (NO)
 Pressure 2158 psi Test Temperature 534 °F

Pressure Relief Valves:
 Service NA Size NA
 Opening Pressure NA Blowdown (if applicable) NA
 Set Pressure and Blowdown Adjustments Made Using NA
 at NA (Test Medium)

NOTE: Supplemental sheets in the form of lists, sketches, or drawings may be used, provided (1) size is 8 1/2 x 11 in. (2) Information in items 1 through 6 on this report is included on each sheet, and (3) each sheet is numbered and the number of sheets is recorded at the top of this form.

FORM NIS-2 /NR-1 /NVR-1-

8. REMARKS

Applicable Manufacturer Data Reports to be Attached

N/A

CERTIFICATE OF COMPLIANCE

We certify that the statements made in the report are correct and this repair conforms to the rules of the ASME Code Section XI.

Type Code Symbol NR Certificate of Authorization No. 20 Expiration Date 7-17-01

Signed [Signature] Nuclear Safety & Inspection Manager Date 6/14/00

Owner or Owner's Designee, Title

CERTIFICATE OF INSERVICE INSPECTION

I, the undersigned, holding a valid commission issued by the National Board of Boiler and Pressure Vessel Inspectors in the State or Province of Ohio and employed by Factory Mutual Ins. Co. of Johnston, R.I. have inspected the components described in this Owner's Report during the period 4/29/00 to 6/15/00, and state that to the best of my knowledge and belief, the Owner has performed examinations and taken corrective measures described in this Owner's Report in accordance with the requirements of the ASME Code, Section XI.

By signing this certificate, neither the Inspector nor his employer makes any warranty, expressed or implied, concerning the examinations and corrective measures described in the Owner's Report. Furthermore, neither the Inspector nor his employer shall be liable in any manner for any personal injury or property damage or a loss of any kind arising from or connected with this inspection.

Thomas J. Pope
Inspector's Signature

6/15/00
Date

Commissions NB 9336 "N I A" Ohio
National Board, State, Province and Endorsements

CERTIFICATE OF COMPLIANCE

We certify that the statements made in this report are correct and the repair, modification, or replacement of the pressure relief device described above conforms to Section XI and Section III of the ASME Code and the National Board rules as defined in publications NB-65 and NB-102, current editions.

Certificate of Authorization No. 316 to use the "VR" stamp expires Dec. 13, 2001

Certificate of Authorization No. 20 to use the "NR" stamp expires July 17, 2001

Date 19, Signed Toledo Edison Nuclear Safety & Inspection Manager
Repair Organization Authorized Representative Title

CERTIFICATE OF INSPECTION

I, the undersigned, holding a valid commission issued by the National Board of Boiler and Pressure Vessel Inspectors, and certificate of competency issued by the State or Province of Ohio and employed by Factory Mutual Ins. Co.

of Johnston, R.I. have inspected the repair, modification, or replacement described in this report

on 19 and state that to the best of my knowledge and belief, this repair, modification, or replacement has been made or constructed in accordance with Section XI and Section III of the ASME Code and the National Board rules as defined in the publication NB-65 and NB-102, current editions.

By signing this certificate, neither the Inspector nor his employer makes any warranty, expressed or implied, concerning the repair, modification or replacement described in this report. Furthermore, neither the Inspector nor his employer shall be liable in any manner for any personal injury or property damage or a loss of any kind arising from or connected with this inspection.

Authorized Inspector's Signature Date

Commissions Factory Mutual Ins. Co
National Board, State, Province and Endorsements

* Factory Mutual Ins. Co

First Energy Nuclear Operating, 76 South Main St., Akron, Ohio 44308

Davis-Besse Nuclear Power Station Unit #1, 5501 N. St. Rt. 2, Oak Harbor, Ohio 43449

Commercial Service Date: 11/27/77

FORM NIS-2 / NR-1 / NVR-1 OWNER'S REPORT FOR REPAIRS OR REPLACEMENTS As Required by the Provisions of the ASME Code Section XI

1. Owner Toledo Edison Company Date 4/27/00
Name
300 Madison Ave., Toledo, OH 43652
Address Sheet 1 of 1
2. Plant Davis-Besse Nuclear Power Station Unit #1
Name
5501 N. SR 2, Oak Harbor, OH 43449
Address WO 99-003278-000
Repair Organization P.O. No., Job No., etc.
3. Work Performed By Toledo Edison Company Type Code Stamp NR
Name
5501 N. SR 2, Oak Harbor, OH 43449
Address Authorization NO. 20
 Expiration Date 7-17-01
4. Identification of Systems Main Steam System EBB-1-SR-4 C-169 (Subsys 083-01)
5. (a) Applicable Construction Code ANSI B31.1 Edition 19 67 Addendum N/A Code Case N/A
 (b) Applicable Edition of Section XI Utilized for Repairs or Replacements 19 86
 (c) Design Responsibilities Lisega

6. Identification of Components Repaired or Replaced and Replacement Components

Name of Component	Name of Manufacturer	Manufacturer's Serial NO.	National Board NO.	Other Identification/Jurisdictional NO.	Year Built	Repaired Replaced or Replacement	ASME Code Stamped
Snubber	Lisega	61252/40	N/A	C-169	N/A	Replaced	No
Snubber	Lisega	61233/40	N/A	C-169	N/A	Replacement	No

7. Description of Work

Lisega hydraulic snubber S/N 61252/40 was removed and replaced with a like for like Lisega snubber S/N 61233/40

Snubber was functionally tested satisfactory prior to installation.

8. Tests Conducted: Hydrostatic (NO) Pneumatic (NO) Nominal Operating Pressure (NO) Other (NO)
 Pressure N/A psi Test Temperature N/A °F
- Pressure Relief Valves:
 Service N/A Size N/A
 Opening Pressure N/A Blowdown (if applicable) N/A
 Set Pressure and Blowdown Adjustments Made Using N/A
 at N/A (Test Station)

NOTE: Supplemental sheets in the form of lists, sketches, or drawings may be used, provided (1) size is 8 1/2 x 11 in. (2) Information in items 1 through 6 on this report is included on each sheet, and (3) each sheet is numbered and the number of sheets is recorded at the top of this form.

First Energy Nuclear Operating, 76 South Main St., Akron, Ohio 44308

Davis-Besse Nuclear Power Station Unit #1, 5501 N. St. Rt. 2, Oak Harbor, Ohio 43449

Commercial Service Date: 11/27/77

FORM NIS-2 / NR-1 / NVR-1

9. REMARKS

Applicable Manufacturers Data Reports to be Attached

NA

CERTIFICATE OF COMPLIANCE

We certify that the statements made in the report are correct and this replacement conforms to the rules of the ASME Code Section XI.

Type Code Symbol NR Certificate of Authorization No. 20 Expiration Date 7-17-01

Signed [Signature] Nuclear Safety & Inspection Manager Date 4/30/2000 19

Owner or Owner's Designee, Title

CERTIFICATE OF INSERVICE INSPECTION

I, the undersigned, holding a valid commission issued by the National Board of Boiler and Pressure Vessel Inspectors in the State or Province of Ohio and employed by Factory Mutual Ins. Co. of Johnston, R.I. have inspected the components described in this Owner's Report during the period 4-23-99 to 4-30-00, and state that to the best of my knowledge and belief, the Owner has performed examinations and taken corrective measures described in this Owner's Report in accordance with the requirements of the ASME Code, Section XI.

By signing this certificate, neither the Inspector nor his employer makes any warranty, expressed or implied, concerning the examinations and corrective measures described in the Owner's Report. Furthermore, neither the Inspector nor his employer shall be liable in any manner for any personal injury or property damage or a loss of any kind arising from or connected with this inspection.

Inspector's Signature [Signature] Date 4-30-00 Commissions 169801 N I A AK
National Board, State, Province and Endorsements

CERTIFICATE OF COMPLIANCE

We certify that the statements made in this report are correct and the repair, modification, or replacement of the pressure relief device described above conforms to Section XI and Section III of the ASME Code and the National Board rules as defined in publications NB-65 and NB-102, current editions.

Certificate of Authorization No. 316 to use the "VR" stamp expires Dec. 13, 2001.

Certificate of Authorization No. 20 to use the "NR" stamp expires July 17, 2001.

Date , 19 , Signed Toledo Edison
Repair Organization Authorized Representative Title

Nuclear Safety & Inspection Manager

CERTIFICATE OF INSPECTION

I, the undersigned, holding a valid commission issued by the National Board of Boiler and Pressure Vessel Inspectors, and certificate of competency issued by the State or Province of Ohio and employed by Factory Mutual Ins. Co. of Johnston, R.I. have inspected the repair, modification, or replacement described in this report

on , 19 and state that to the best of my knowledge and belief, this repair, modification, or replacement has been made or constructed in accordance with Section XI and Section III of the ASME Code and the National Board rules as defined in the publication NB-65 and NB-102, current editions.

By signing this certificate, neither the Inspector nor his employer makes any warranty, expressed or implied, concerning the repair, modification or replacement described in this report. Furthermore, neither the Inspector nor his employer shall be liable in any manner for any personal injury or property damage or a loss of any kind arising from or connected with this inspection.

Authorized Inspector's Signature

Date

Commissions
National Board, State, Province and Endorsements

* Factory Mutual Ins. Co

First Energy Nuclear Operating, 76 South Main St., Akron, Ohio 44308

Davis-Besse Nuclear Power Station Unit #1, 5501 N. St. Rt. 2, Oak Harbor, Ohio 43449

Commercial Service Date: 11/27/77

FORM NIS-2 / NR-1 / NVR-1 OWNER'S REPORT FOR REPAIRS OR REPLACEMENTS As Required by the Provisions of the ASME Code Section XI

1. Owner Toledo Edison Company Date 5/2/00
300 Madison Ave., Toledo, OH 43652 Sheet 1 of 1
 2. Plant Davis-Besse Nuclear Power Station Unit #1
5501 N. SR 2, Oak Harbor, OH 43449 WO 99-003279-000
 3. Work Performed By Toledo Edison Company Type Code Stamp NR
5501 N. SR 2, Oak Harbor, OH 43449 Authorization NO. 20
 Expiration Date 7-17-01

4. Identification of Systems MAIN STEAM SYSTEM EBB-1-SR4 WDR BOTTOM (C-224) (SUB 083-01)

5. (a) Applicable Construction Code ANSI B31.1 Edition 19 87 Addendum N/A Code Case N/A
 (b) Applicable Edition of Section XI Utilized for Repairs or Replacements 19 86
 (c) Design Responsibilities LISEGA

6. Identification of Components Repaired or Replaced and Replacement Components

Name of Component	Name of Manufacturer	Manufacturer's Serial NO.	National Board NO.	Other Identification/Jurisdictional NO.	Year Built	Repaired or Replacement	ASME Code Stamped
Snubber	LISEGA	61233/38	N/A	C-224	Unk	Replaced	No
Snubber	LISEGA	99614610/22	N/A	C-224	Unk	Replacement	No

7. Description of Work

Liseqa hydraulic snubber 6/N61233/38 was removed and replaced with a like-for-like snubber 6/N99614610/22.
 Replacement snubber was functionally tested satisfactory prior to installation.

8. Tests Conducted: Hydrostatic (NO) Pneumatic (NO) Nominal Operating Pressure (NO) Other (NO)
 Pressure N/A psi Test Temperature N/A °F

Pressure Relief Valves:
 Service N/A Size N/A
 Opening Pressure N/A Blowdown (if applicable) N/A
 Set Pressure and Blowdown Adjustments Made Using N/A
 at N/A

NOTE: Supplemental sheets in the form of lists, sketches, or drawings may be used, provided (1) size is 8 1/2 x 11 in. (2) Information in items 1 through 6 on this report is included on each sheet, and (3) each sheet is numbered and the number of sheets is recorded at the top of this form.

First Energy Nuclear Operating, 76 South Main St., Akron, Ohio 44308

Davis-Besse Nuclear Power Station Unit #1, 5501 N. St. Rt. 2, Oak Harbor, Ohio 43449

Commercial Service Date: 11/27/77

FORM NIS-2 / NR-1 / NVR-1

8. REMARKS

Applicable Manufacturers Data Reports to be Attached

NA

CERTIFICATE OF COMPLIANCE

We certify that the statements made in the report are correct and this replacement conforms to the rules of the ASME Code Section XI.

Type Code Symbol NR Certificate of Authorization No. 20 Expiration Date 7-17-01

Signed A. W. S. [Signature] Nuclear Safety & Inspection Manager Date 5/2/2000
Owner or Owner's Designee, Title

CERTIFICATE OF INSERVICE INSPECTION

I, the undersigned, holding a valid commission issued by the National Board of Boiler and Pressure Vessel Inspectors in the State or Province of Ohio and employed by Factory Mutual Ins. Co. of Johnston, R.I. have inspected the components described in this Owner's Report during the period 9-23-99 to 5-2-00, and state that to the best of my knowledge and belief, the Owner has performed examinations and taken corrective measures described in this Owner's Report in accordance with the requirements of the ASME Code, Section XI.

By signing this certificate, neither the Inspector nor his employer makes any warranty, expressed or implied, concerning the examinations and corrective measures described in the Owner's Report. Furthermore, neither the Inspector nor his employer shall be liable in any manner for any personal injury or property damage or a loss of any kind arising from or connected with this inspection.

Inspector's Signature [Signature] Date 5/2/00

Commissions NB 9801 "N.I.A." OHIO
National Board, State, Province and Endorsements

CERTIFICATE OF COMPLIANCE

We certify that the statements made in this report are correct and the repair, modification, or replacement of the pressure relief device described above conforms to Section XI and Section III of the ASME Code and the National Board rules as defined in publications NB-65 and NB-102, current editions.

Certificate of Authorization No. 316 to use the "VR" stamp expires Dec. 13, 2001

Certificate of Authorization No. 20 to use the "NR" stamp expires July 17, 2001

Date , 19 , Signed Toledo Edison Nuclear Safety & Inspection Manager
Repair Organization Authorized Representative Title

CERTIFICATE OF INSPECTION

I, the undersigned, holding a valid commission issued by the National Board of Boiler and Pressure Vessel Inspectors, and certificate of competency issued by the State or Province of Ohio and employed by Factory Mutual Ins. Co.* of Johnston, R.I. have inspected the repair, modification, or replacement described in this report

on , 19 and state that to the best of my knowledge and belief, this repair, modification, or replacement has been made or constructed in accordance with Section XI and Section III of the ASME Code and the National Board rules as defined in the publication NB-65 and NB-102, current editions.

By signing this certificate, neither the Inspector nor his employer makes any warranty, expressed or implied, concerning the repair, modification or replacement described in this report. Furthermore, neither the Inspector nor his employer shall be liable in any manner for any personal injury or property damage or a loss of any kind arising from or connected with this inspection.

Authorized Inspector's Signature Date

Commissions
National Board, State, Province and Endorsements

* Factory Mutual Ins. Co

First Energy Nuclear Operating, 76 South Main St., Akron, Ohio 44308

Davis-Besse Nuclear Power Station Unit #1, 5501 N. St. Rt. 2, Oak Harbor, Ohio 43449

Commercial Service Date: 11/27/77

FORM NIS-2 / NR-1 / NVR-1 OWNER'S REPORT FOR REPAIRS OR REPLACEMENTS As Required by the Provisions of the ASME Code Section XI

1. Owner Toledo Edison Company Date 5/22/00
Name
300 Madison Ave., Toledo, OH 43652
Address
 Sheet 1 of 1
2. Plant Davis-Besse Nuclear Power Station Unit # 1
Name
5501 N. SR 2, Oak Harbor, OH 43449
Address
WO 99-003333-000
Repair Organization P.O. No., Job No., etc.
3. Work Performed By Toledo Edison Company Type Code Stamp NR
Name
5501 N. SR 2, Oak Harbor, OH 43449
Address
 Authorization NO. 20
 Expiration Date 7-17-01
4. Identification of Systems Main Steam Line #2 Code Safety Valves
5. (a) Applicable Construction Code ASME III CL2 Edition 19 68 Addendum W68 Code Case N/A
 (b) Applicable Edition of Section XI Utilized for Repairs or Replacements 19 86
 (c) Design Responsibilities Dresser Valve

6. Identification of Components Repaired or Replaced and Replacement Components

Name of Component	Name of Manufacturer	Manufacturer's Serial NO.	National Board NO.	Other Identification/Jurisdictional NO.	Year Built	Repaired Replaced or Replacement	ASME Code Stamped
SP17A5	Dresser Valve	BM-08636	n/a	n/a	1972	Replaced	Yes
SP17A5	Dresser Valve	BM-08048	n/a	n/a	1971	Replacement	Yes
SP17A7	Dresser Valve	BM-08641	n/a	n/a	1977	Replaced	Yes
SP17A7	Dresser Valve	BM-08052	n/a	n/a	1971	Replacement	Yes
SP17A9	Dresser Valve	BM-08643	n/a	n/a	1972	Replaced	Yes
SP17A9	Dresser Valve	BM-08049	n/a	n/a	1971	Replacement	Yes

7. Description of Work

Removed SP17A5, SP17A7, and SP17A9. Replaced with like for like valves rebuilt by Dresser under PO 7015788 and tested by Wyle Lab.
 One 1 3/8" x 11 3/8" SA193, Gr B7 (H#8094962) stud supplied by Nova was replaced on valve SP17A9 on the reinstallation. A VT-2 Inservice test was performed satisfactory at completion of work.

8. Tests Conducted: Hydrostatic (NO) Pneumatic (NO) Nominal Operating Pressure (YES) Other (NO)
 Pressure N/A psi Test Temperature N/A °F
- Pressure Relief Valves:
 Service N/A Size N/A
 Opening Pressure N/A Blowdown (if applicable) N/A
 Set Pressure and Blowdown Adjustments Made Using N/A
 at N/A

NOTE: Supplemental sheets in the form of lists, sketches, or drawings may be used, provided (1) size is 8 1/2 x 11 in. (2) Information in items 1 through 6 on this report is included on each sheet, and (3) each sheet is numbered and the number of sheets is recorded at the top of this form.

First Energy Nuclear Operating, 76 South Main St., Akron, Ohio 44308

Davis-Besse Nuclear Power Station Unit #1, 5501 N. St. Rt. 2, Oak Harbor, Ohio 43449

Commercial Service Date: 11/27/77

FORM NIS-2 /NR-1 /NVR-T

9. REMARKS

Applicable Manufacturers Data Reports to be Attached

Dresser Valve NVR-1 Reports (3)

CERTIFICATE OF COMPLIANCE

We certify that the statements made in the report are correct and this replacement conforms to the rules of the ASME Code Section XI.

Type Code Symbol NR Certificate of Authorization No. 20 Expiration Date 7-17-01

Signed A. W. S. [Signature] Nuclear Safety & Inspection Manager Date 5/22/00 19
Owner or Owner's Designee, Title

CERTIFICATE OF INSERVICE INSPECTION

I, the undersigned, holding a valid commission issued by the National Board of Boiler and Pressure Vessel Inspectors in the State or Province of Ohio and employed by Factory Mutual Ins. Co. of Johnston, R.I. have inspected the components described in this Owner's Report during the period 12/6/99 to 5/23/00, and state that to the best of my knowledge and belief, the Owner has performed examinations and taken corrective measures described in this Owner's Report in accordance with the requirements of the ASME Code, Section XI.

By signing this certificate, neither the Inspector nor his employer makes any warranty, expressed or implied, concerning the examinations and corrective measures described in the Owner's Report. Furthermore, neither the Inspector nor his employer shall be liable in any manner for any personal injury or property damage or a loss of any kind arising from or connected with this inspection.

Thomas J. Lepore AND 5/23/00 Commissions NB 9330 N I A OHIO
Inspector's Signature Date National Board, State, Province and Endorsements CMH

CERTIFICATE OF COMPLIANCE

We certify that the statements made in this report are correct and the repair, modification, or replacement of the pressure relief device described above conforms to Section XI and Section III of the ASME Code and the National Board rules as defined in publications NB-65 and NB-102, current editions.

Certificate of Authorization No. 316 to use the "VR" stamp expires Dec. 13, 2001

Certificate of Authorization No. 20 to use the "NR" stamp expires July 17, 2001

Date _____, 19____, Signed Toledo Edison Nuclear Safety & Inspection Manager
Repair Organization Authorized Representative Title

CERTIFICATE OF INSPECTION

I, the undersigned, holding a valid commission issued by the National Board of Boiler and Pressure Vessel Inspectors, and

certificate of competency issued by the State or Province of Ohio and employed by Factory Mutual Ins. Co.

of Johnston, R.I. have inspected the repair, modification, or replacement described in this report

on _____, 19____ and state that to the best of my knowledge and belief, this repair, modification, or replacement has been made or constructed in accordance with Section XI and Section III of the ASME Code and the National Board rules as defined in the publication NB-65 and NB-102, current editions.

By signing this certificate, neither the Inspector nor his employer makes any warranty, expressed or implied, concerning the repair, modification or replacement described in this report. Furthermore, neither the Inspector nor his employer shall be liable in any manner for any personal injury or property damage or a loss of any kind arising from or connected with this inspection.

Authorized Inspector's Signature _____ Date _____ Commissions _____
National Board, State, Province and Endorsements

* Factory Mutual Ins. Co

FORM NVR-1 REPORT OF REPAIR ☒ MODIFICATION ☒ OR REPLACEMENT ☒
OF NUCLEAR PRESSURE RELIEF DEVICES

1. Work performed by Dresser Valve Division; Dresser Equipment Group, Inc. 7015788 01-27014-0
 (name of organization) (P.O. no., job no., etc.)
Intersection Hwy. 157 @ 3225 North, Alexandria, Louisiana 71309
 (address)

2. Work performed for First Energy Corporation The Toledo Edison Company 5501 North State Route 2 Oak Harbor, Ohio 43449
 (name and address)

3. Owner The Toledo Edison Company Davis Besse Plant 5501 North State Route 2
 (name)
Oak Harbor, Ohio 43449
 (address)

4. Name, address and identification of nuclear power plant The Toledo Edison Company Davis Besse Plant
5501 North State Route 2

5. a: Repaired pressure relief device: Safety Valve
 b: Name of manufacturer Same as Item 1 above
 c: Identifying nos. 3707RA BM-08049 N/A STEAM 6" 1971
 (type) (mfr's serial no.) (Nat'l Bd. No.) (service) (size) (year built)
 d: Construction Code ASME Section III 1968 Winter 1968 N/A A Vessels, Art. 9
 (name/section/division) (edition) (addenda) (Code case(s)) (Code Class)

6. ASME Code Section XI applicable for in service inspection: 1977 Summer 1978 N/A
 (edition) (addenda) (Code Case(s))

7. ASME Code Section XI used for repairs, modifications, or replacements: 1977 Summer 1978 N/A
 (edition) (addenda) (Code Case(s))

8. Construction Code used for repairs, modifications, or replacements: 1968 Winter 1968 N/A
 (edition) (addenda) (Code Case(s))

9. Design responsibility Same as Item 1 above

10. Opening pressure: 1100 PSIG Blowdown (if applicable) N/A % Set pressure and blowdown adjustment
 made at: Wyle Lab. Huntsville, Alabama Using Steam
 (location) (test medium)

11. Description of work (include name and identifying number of replacement parts) Valve Was Disassembled, Inspected, Modified, Repaired, Assembled, & Tested. Replaced Comp. Screw P/N 4777601N, Spindle P/N 4337201N, Cotter Pin P/N 2220219, Disc Collar P/N 4350101N & Disc P/N WNN815GN S/N ABK45.

12. Remarks: Parts Added Are : Washer P/N 4351201, Bushing P/N 4351301, Cap Screw P/N 2311560 & Lockwasher P/N 2010206. Yoke & Yoke Rod Nuts Were Reworked By Machining.

CERTIFICATE OF INSPECTION

I, Terry W. Barnes, certify that to the best of my knowledge and belief the statements made in this report are correct and the repair, modification or replacement of the pressure relief devices described above conforms to Section XI of the ASME Code and the National Board Inspection Code "VR" and "NR" rules.

National Board Certificate of Authorization No. VR-70,85,87-118 & 120-124 to use the "VR" stamp expires 6-29-2001

National Board Certificate of Authorization No. NR-45 to use the "NR" stamp expires 6-30-2002

Date 4-12-2000 Same as Item 1 above. Signed Terry W. Barnes Quality Engineer
 (name of certificate holder) (authorized representative) (title)

CERTIFICATE OF INSPECTION

I, B.C. Dasher, holding a valid commission issued by The National Board of Boiler and Pressure Vessel Inspectors and certificate of competency issued by the jurisdiction of Louisiana and employed by Hartford Steam Boiler Inspection and Insurance Co. of Hartford, CT have inspected the repair, modification or replacement described in this report on 4/12/00 and state that to the best of my knowledge and belief, this repair, modification or replacement has been completed in accordance with Section XI of the ASME Code and the National Board Inspection Code "VR" and "NR" rules.

By signing this certificate, neither the undersigned nor my employer makes any warranty, expressed or implied, concerning the repair modification or replacement described in this report. Furthermore, neither the undersigned nor my employer shall be liable in any manner for any personal injury, property damage or loss of any kind arising from or connected with this inspection.

Date 4/12/00 Signed B.C. Dasher Commissions NB7779 A "N" LA664
 (National Board (incl. endorsements), and jurisdiction, and no.)

First Energy Nuclear Operating, 76 South Main St., Akron, Ohio 44308

Davis-Besse Nuclear Power Station Unit #1, 5501 N. St. Rt. 2, Oak Harbor, Ohio 43449

Commercial Service Date: 11/27/77

FORM NVR-1 REPORT OF REPAIR ☒ MODIFICATION ☒ OF NUCLEAR PRESSURE RELIEF DEVICES

OR REPLACEMENT ☒

1. Work performed by Dresser Valve Division; Dresser Equipment Group, Inc. 7015788 01-27014-0
 (name of organization) (P.O. no., job no., etc.)
Intersection Hwy. 167 @ 3225 North, Alexandria, Louisiana 71309
 (address)

2. Work performed for First Energy Corporation The Toledo Edison Company 5501 North State Route 2 Oak Harbor, Ohio 43449
 (name and address)

3. Owner The Toledo Edison Company Davis Besse Plant 5501 North State Route 2
 (name)
Oak Harbor, Ohio 43449
 (address)

4. Name, address and identification of nuclear power plant The Toledo Edison Company Davis Besse Plant
5501 North State Route 2

5. a: Repaired pressure relief device: Safety Valve
 b: Name of manufacturer Same as item 1 above
 c: Identifying nos. 3777QA BM-08052 N/A STEAM 6" 1971
 (type) (mfr's serial no.) (Natl. Bd. No.) (service) (size) (year built)
 d: Construction Code ASME Section III 1968 Winter 1968 N/A A Vessels, Art. 9
 (name/section/division) (edition) (addenda) (Code case(s)) (Code Class)

6. ASME Code Section XI applicable for in service inspection: 1977 Summer 1978 N/A
 (edition) (addenda) (Code Case(s))

7. ASME Code Section XI used for repairs, modifications, or replacements: 1977 Summer 1978 N/A
 (edition) (addenda) (Code Case(s))

8. Construction Code used for repairs, modifications, or replacements: 1968 Winter 1968 N/A
 (edition) (addenda) (Code Case(s))

9. Design responsibility Same as item 1 above

10. Opening pressure: 1050 PSIG Blowdown (if applicable) N/A % Set pressure and blowdown adjustment
 made at: Wyle Lab Huntsville, Alabama Using Steam
 (location) (test medium)

11. Description of work: (include name and identifying number of replacement parts) Valve Was Disassembled, Inspected, Modified, Repaired, Assembled, & Tested. Replaced Comp. Screw P/N 4777701N, Spindle P/N 4777801N, Cotter Pin P/N 2220219 & Disc P/N VGZB15KOS422 S/N AAP65.

12. Remarks: Parts Added Are : Washer P/N 4351201, Bushing P/N 4351301, Cap Screw P/N 2311560 & Lockwasher P/N 2010208. Yoke & Yoke Rod Nuts Were Reworked By Machining.

CERTIFICATE OF INSPECTION

I, Terry W. Barnes, certify that to the best of my knowledge and belief the statements made in this report are correct and the repair, modification or replacement of the pressure relief devices described above conforms to Section XI of the ASME Code and the National Board Inspection Code "VR" and "NR" rules.

National Board Certificate of Authorization No. VR-70,85,87-118 & 120-124 to use the "VR" stamp expires 6-29-2001

National Board Certificate of Authorization No. NR-45 to use the "NR" stamp expires 6-30-2002

Date 4-12-2000 Same as item 1 above. Signed Terry W. Barnes Quality Engineer
 (name of certificate holder) (authorized representative) (title)

CERTIFICATE OF INSPECTION

I, B.C. Dasher, holding a valid commission issued by The National Board of Boiler and Pressure Vessel Inspectors and certificate of competency issued by the jurisdiction of Louisiana and employed by Hartford Steam Boiler Inspection and Insurance Co. of Hartford, CT have inspected the repair, modification or replacement described in this report on 4/12/00 and state that to the best of my knowledge and belief, this repair, modification or replacement has been completed in accordance with Section XI of the ASME Code and the National Board Inspection Code "VR" and "NR" rules.

By signing this certificate, neither the undersigned nor my employer makes any warranty, expressed or implied, concerning the repair, modification or replacement described in this report. Furthermore, neither the undersigned nor my employer shall be liable in any manner for any personal injury, property damage or loss of any kind arising from or connected with this inspection.

Date 4/12/00 Signed B.C. Dasher Commissions VB7779722 LB 664
 (National Board (incl. endorsements), and jurisdiction, and no.)

FORM NVR-1 REPORT OF REPAIR ☒ MODIFICATION ☒ OR REPLACEMENT ☒
OF NUCLEAR PRESSURE RELIEF DEVICES

1. Work performed by Dresser Valve Division; Dresser Equipment Group, Inc. 7015788 01-27014-0
 (name of organization) (P.O. no., job no., etc.)
Intersection Hwy. 167 @ 3225 North, Alexandria, Louisiana 71309
 (address)

2. Work performed for First Energy Corporation The Toledo Edison Company 5501 North State Route 2 Oak Harbor, Ohio 43449
 (name and address)

3. Owner The Toledo Edison Company Davis Besse Plant 5501 North State Route 2
 (name)
Oak Harbor, Ohio 43449
 (address)

4. Name, address and identification of nuclear power plant The Toledo Edison Company Davis Besse Plant
5501 North State Route 2
 (address)

5. a: Repaired pressure relief device: Safety Valve
 b: Name of manufacturer Same as Item 1 above
 c: Identifying nos. 3707RA BM-08048 N/A STEAM 8" 1971
 (type) (mfr's serial no.) (Nat'l. Bd. No.) (service) (size) (year built)
 d: Construction Code ASME Section III 1968 Winter 1968 N/A A Vessels, Art. 9
 (name/section/division) (edition) (addenda) (Code Case(s)) (Code Class)

6. ASME Code Section XI applicable for in service inspection: 1977 Summer 1978 N/A
 (edition) (addenda) (Code Case(s))

7. ASME Code Section XI used for repairs, modifications, or replacements: 1977 Summer 1978 N/A
 (edition) (addenda) (Code Case(s))

8. Construction Code used for repairs, modifications, or replacements: 1968 Winter 1968 N/A
 (edition) (addenda) (Code Case(s))

9. Design responsibility Same as Item 1 above

10. Opening pressure: 1100 PSIG Blowdown (if applicable) N/A % Set pressure and blowdown adjustment
 made at: Wyle Lab. Huntsville, Alabama Using Steam
 (location) (test medium)

11. Description of work: (include name and identifying number of replacement parts) Valve Was Disassembled, Inspected, Modified, Repaired, Assembled, & Tested. Replaced Comp. Screw P/N 4777601N, Spindle P/N 4337201N, Cotter Pin P/N 2220219, Disc Collar P/N 4350101N & Disc P/N WNN815GOS421 S/N AAR53.

12. Remarks: Parts Added Are : Washer P/N 4351201, Bushing P/N 4351301, Cap Screw P/N 2311560 & Lockwasher P/N 2010206. Yoke & Yoke Rod Nuts Were Reworked By Machining.

CERTIFICATE OF INSPECTION

I, Terry W. Barnes, certify that to the best of my knowledge and belief the statements made in this report are correct and the repair, modification or replacement of the pressure relief devices described above conforms to Section XI of the ASME Code and the National Board Inspection Code "VR" and "NR" rules.

National Board Certificate of Authorization No. VR-70,95,97-118 & 120-124 to use the "VR" stamp expires 6-29-2001

National Board Certificate of Authorization No. NR-45 to use the "NR" stamp expires 6-30-2002

Date 4-12-2000 Same as Item 1 above. Signed Terry W. Barnes Quality Engineer
 (name of certificate holder) (authorized representative) (title)

CERTIFICATE OF INSPECTION

I, B.C. Dasher, holding a valid commission issued by The National Board of Boiler and Pressure Vessel Inspectors and certificate of competency issued by the jurisdiction of Louisiana and employed by Hartford Steam Boiler Inspection and Insurance Co. of Hartford, CT have inspected the repair, modification or replacement described in this report on 4/12/00 and state that to the best of my knowledge and belief, this repair, modification or replacement has been completed in accordance with Section XI of the ASME Code and the National Board Inspection Code "VR" and "NR" rules.

By signing this certificate, neither the undersigned nor my employer makes any warranty, expressed or implied, concerning the repair modification or replacement described in this report. Furthermore, neither the undersigned nor my employer shall be liable in any manner for any personal injury, property damage or loss of any kind arising from or connected with this inspection.

Date 4/12/00 Signed B.C. Dasher Commissions NB 7779 "A" & "L" 14664
 (National Board (incl. endorsements), and jurisdiction, and no.)

First Energy Nuclear Operating, 76 South Main St., Akron, Ohio 44308

Davis-Besse Nuclear Power Station Unit #1, 5501 N. St. Rt. 2, Oak Harbor, Ohio 43449

Commercial Service Date: 11/27/77

FORM NIS-2 / NR-1 / NVR-1 OWNER'S REPORT FOR REPAIRS OR REPLACEMENTS As Required by the Provisions of the ASME Code Section XI

1. Owner Toledo Edison Company Date 5/22/00
Name
300 Madison Ave., Toledo, OH 43652
Address
 Sheet 1 of 1

2. Plant Davis-Besse Nuclear Power Station Unit # 1
Name
5501 N. SR 2, Oak Harbor, OH 43449
Address
WO 99-003334-000
Repair Organization P.O. No., Job No., etc.

3. Work Performed By Toledo Edison Company Type Code Stamp NR
Name
5501 N. SR 2, Oak Harbor, OH 43449
Address
 Authorization NO. 20
 Expiration Date 7-17-01

4. Identification of Systems Main Steam Line #1 Code Safety Valves

5. (a) Applicable Construction Code ASME III CL2 Edition 19 68 Addendum W68 Code Case N/A
 (b) Applicable Edition of Section XI Utilized for Repairs or Replacements 19 86
 (c) Design Responsibilities Dresser Valve

6. Identification of Components Repaired or Replaced and Replacement Components

Name of Component	Name of Manufacturer	Manufacturer's Serial NO.	National Board NO.	Other Identification/Jurisdictional NO.	Year Built	Repaired Replaced or Replacement	ASME Code Stamped
6P17B3	Dresser Valve	BM-8634	n/a	n/a	1972	Replaced	Yes
SP17B3	Dresser Valve	BM-08635	n/a	n/a	1972	Replacement	Yes
SP17B6	Dresser Valve	BM-8639	n/a	n/a	1972	Replaced	Yes
SP17B6	Dresser Valve	BM-8640	n/a	n/a	1972	Replacement	Yes
SP17B7	Dresser Valve	BS-06255	n/a	n/a	1978	Replaced	Yes
SP17B7	Dresser Valve	BM-08051	n/a	n/a	1971	Replacement	Yes

7. Description of Work

Removed 6P17B3, 6P17B6, and 6P17B7. Replaced with like for like valve rebuilt by Dresser(SP17B7) under PO 7015788 and tested by Wyle Lab.
 SP17B3 and SP17B6 replaced with like for like valves rebuilt by NWS Technologies, LLC under PO 7014672 Rev. 4 and tested satisfactory at their lab.
 Seven studs (1 3/8" x 11 3/8" SA 193 Gr B7) and ten nuts (1 3/8"-8 UN 2B, SA-194 Gr 2H) were replaced on the inlet flanges. Bolting materials are referenced and listed in the body of the work order. After installation, a VT-2 inservice was performed satisfactory at completion of work.

8. Tests Conducted: Hydrostatic (NO) Pneumatic (NO) Nominal Operating Pressure (YES) Other (NO)
 Pressure N/A psi Test Temperature N/A °F

Pressure Relief Valves:
 Service N/A Size N/A
 Opening Pressure N/A Blowdown (if applicable) N/A
 Set Pressure and Blowdown Adjustments Made Using N/A
 at N/A

NOTE: Supplemental sheets in the form of lists, sketches, or drawings may be used, provided (1) size is 8 1/2 x 11 in. (2) Information in items 1 through 6 on this report is included on each sheet, and (3) each sheet is numbered and the number of sheets is recorded at the top of this form.

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First Energy Nuclear Operating, 76 South Main St., Akron, Ohio 44308

Davis-Besse Nuclear Power Station Unit #1, 5501 N. St. Rt. 2, Oak Harbor, Ohio 43449

Commercial Service Date: 11/27/77

FORM NIS-2 /NR-1 /NVR-1

9. REMARKS

Applicable Manufacturers Data Reports to be Attached

Dresser Valve NVR-1 Report (1), NWS Technologies, LLC NVR-1 Reports (2).

CERTIFICATE OF COMPLIANCE

We certify that the statements made in the report are correct and this replacement conforms to the rules of the ASME Code Section XI.

Type Code Symbol NR Certificate of Authorization No. 20 Expiration Date 7-17-01

Signed SW Starnes Nuclear Safety & Inspection Manager Date 5/22/2000

Owner or Owner's Designee, Title

CERTIFICATE OF INSERVICE INSPECTION

I, the undersigned, holding a valid commission issued by the National Board of Boiler and Pressure Vessel Inspectors in the State or Province of Ohio and employed by Factory Mutual Ins. Co. of Johnston, R.I. have inspected the components described in this Owner's Report during the period 10/25/99 to 5/23/00, and state that to the best of my knowledge and belief, the Owner has performed examinations and taken corrective measures described in this Owner's Report in accordance with the requirements of the ASME Code, Section XI.

By signing this certificate, neither the Inspector nor his employer makes any warranty, expressed or implied, concerning the examinations and corrective measures described in the Owner's Report. Furthermore, neither the Inspector nor his employer shall be liable in any manner for any personal injury or property damage or a loss of any kind arising from or connected with this inspection.

Inspector's Signature Thomas G. Laps ANII Date 5/23/00 Commissions NB9330 "N I A" "OHIO" "COMM." National Board, State, Province and Endorsements

CERTIFICATE OF COMPLIANCE

We certify that the statements made in this report are correct and the repair, modification, or replacement of the pressure relief device described above conforms to Section XI and Section III of the ASME Code and the National Board rules as defined in publications NB-65 and NB-102, current editions.

Certificate of Authorization No. 316 to use the "VR" stamp expires Dec. 13, 2001

Certificate of Authorization No. 20 to use the "NR" stamp expires July 17, 2001

Date , 19 , Signed Toledo Edison Nuclear Safety & Inspection Manager
Repair Organization Authorized Representative Title

CERTIFICATE OF INSPECTION

I, the undersigned, holding a valid commission issued by the National Board of Boiler and Pressure Vessel Inspectors, and

certificate of competency issued by the State or Province of Ohio and employed by Factory Mutual Ins. Co.

of Johnston, R.I. have inspected the repair, modification, or replacement described in this report

on , 19 and state that to the best of my knowledge and belief, this repair, modification, or replacement has been made or constructed in accordance with Section XI and Section III of the ASME Code and the National Board rules as defined in the publication NB-65 and NB-102, current editions.

By signing this certificate, neither the Inspector nor his employer makes any warranty, expressed or implied, concerning the repair, modification or replacement described in this report. Furthermore, neither the Inspector nor his employer shall be liable in any manner for any personal injury or property damage or a loss of any kind arising from or connected with this inspection.

Authorized Inspector's Signature

Date

Commissions

National Board, State, Province and Endorsements

* Factory Mutual Ins. Co

First Energy Nuclear Operating, 76 South Main St., Akron, Ohio 44308

Davis-Besse Nuclear Power Station Unit #1, 5501 N. St. Rt. 2, Oak Harbor, Ohio 43449

Commercial Service Date: 11/27/77

FORM NVR-1 REPORT OF REPAIR ☒ REPLACEMENT ☒ **OF NUCLEAR PRESSURE RELIEF DEVICES**

1. Work performed by: NWS Technologies, LLC Purchase Order # 7014672 Rev. 4
131 Venture Boulevard, Spartanburg, SC 29301
2. Work performed for: First Energy, Davis-Besse Power Station
- 3/4. Owner - name, address and identification of nuclear power plant: Davis-Besse Power Station
5501 North State Route 2 Oak Harbor, OH 43449-9760
5. a: Repaired pressure relief device: Main Steam Safety Valve
 b: Name of manufacturer: Dresser - Consolidated
 c: Identifying nos.
- | | 3707RA | BM08635 | N/A | steam | 6x8 | 1972 |
|--|-------------------------|-------------|----------------|----------------|--------------|------------|
| | (type) | (mfr's S/N) | (NB#) | (service) | (size) | (yr.built) |
| d: Construction Code: | ASME Sec. III Div. 1 | 1968 | W68 | N/A | 2 | |
| | (name/section/division) | (edition) | (addenda) | (Code Case(s)) | (Code Class) | |
| 6. ASME Code Section XI applicable for inservice inspection: | 1986 | N/A | N/A | | | |
| | (edition) | (addenda) | (Code Case(s)) | | | |
| 7. ASME Code Section XI used for repairs, replacements: | 1986 | N/A | N/A | | | |
| | (edition) | (addenda) | (Code Case(s)) | | | |
| 8. Construction Code used for repairs, replacements: | 1968 | W68 | N/A | | | |
| | (edition) | (addenda) | (Code Case(s)) | | | |
| 9. Design responsibilities: | N/A | | | | | |
| 10. Opening pressure: <u>1100 psig</u>
Set-pressure adjustment made at: <u>NWS Technologies, LLC</u> using <u>steam</u> | | | | | | |
| 11. Description of work (include name and identifying number of replacement parts): <u>Disassembled, inspected, disc replaced, lapped seats, cleaned, lubricated and assembled. Certified set-pressure and seat tightness using steam.</u> | | | | | | |
| 12. Remarks: <u>New disc installed in valve.</u> | | | | | | |

CERTIFICATE OF COMPLIANCE

I, Cesar V. Sierra certify that to the best of my knowledge and belief the statements made in this report are correct and the repair, modification or replacement of the pressure relief devices described above conforms to Section XI of the ASME Code and the National Board Inspection Code "VR" and "NR" rules.

National Board Certificate of Authorization No. 632 to use the "VR" stamp expires April 3, 2000.

National Board Certificate of Authorization No. 81 to use the "NR" stamp expires April 9, 2000.

2/29/00 NWS Technologies, LLC *Cesar V. Sierra* Manager, QA
 Date Repair Organization Authorized representative Title

CERTIFICATE OF INSPECTION

I, Joe C. Hair holding a valid commission issued by The National Board of Boiler and Pressure Vessel Inspectors and certificate of competency issued by the jurisdiction of Texas and employed by Hartford Steam Boiler Inspection & Insurance Co. of Hartford, CT have inspected the repair, modification or replacement described in this report on 2/29/00 and state that to the best of my knowledge and belief, this repair, modification or replacement has been completed in accordance with Section XI of the ASME Code and the National Board Inspection Code "VR" and "NR" rules.

By signing this certificate, neither the undersigned nor my employer makes any warranty, expressed or implied, concerning this repair, modification or replacement described in this report. Furthermore, neither the undersigned nor my employer shall be liable in any manner for any personal injury, property damage or loss of any kind arising from or connected with this inspection.

2/29/00 *Joe C. Hair* NB # 9496 N.A.B.I.S.I.S Texas 1080
 Date Inspector's Signature Commissions (NB (incl endorsements), jurisdiction & no.)

First Energy Nuclear Operating, 76 South Main St., Akron, Ohio 44308

Davis-Besse Nuclear Power Station Unit #1, 5501 N. St. Rt. 2, Oak Harbor, Ohio 43449

Commercial Service Date: 11/27/77

FORM NVR-1 REPORT OF REPAIR ☒ ...PLACEMENT ☒ OF NUCLEAR PRESSURE RELIEF DEVICES

1. Work performed by: NWS Technologies, LLC Purchase Order # 7014672 Rev. 4
131 Venture Boulevard, Spartanburg, SC 29301
2. Work performed for: First Energy, Davis-Besse Power Station
- 3/4. Owner - name, address and identification of nuclear power plant: Davis-Besse Power Station
5501 North State Route 2 Oak Harbor, OH 43449-9760
5. a: Repaired pressure relief device: Main Steam Safety Valve
 b: Name of manufacturer: Dresser - Consolidated
 c: Identifying nos.
- | | <u>3777QA-RT21</u>
(type) | <u>BM8640</u>
(mfr's S/N) | <u>N/A</u>
(NB#) | <u>steam</u>
(service) | <u>6x8</u>
(size) | <u>1972</u>
(yr. built) |
|-----------------------|--|------------------------------|-------------------------|-------------------------------|--------------------------|----------------------------|
| d: Construction Code: | <u>ASME Sec. III Div. 1</u>
(name/section/division) | <u>1968</u>
(edition) | <u>W68</u>
(addenda) | <u>N/A</u>
(Code Cases(s)) | <u>2</u>
(Code Class) | |
6. ASME Code Section XI applicable for inservice inspection: 1986 (edition) N/A (addenda) N/A (Code Case(s))
7. ASME Code Section XI used for repairs, replacements: 1986 (edition) N/A (addenda) N/A (Code Case(s))
8. Construction Code used for repairs, replacements: 1968 (edition) W68 (addenda) N/A (Code Case(s))
9. Design responsibilities: N/A
10. Opening pressure: 1050 psig
 Set-pressure adjustment made at: NWS Technologies, LLC using steam
11. Description of work (include name and identifying number of replacement parts): Disassembled, inspected, replaced guide & holder, lapped seats, assembled. Certified set-pressure and seat tightness on steam.
12. Remarks: New disc holder and guide installed in valve.

CERTIFICATE OF COMPLIANCE

I, Cesar V. Sierra certify that to the best of my knowledge and belief the statements made in this report are correct and the repair, modification or replacement of the pressure relief devices described above conforms to Section XI of the ASME Code and the National Board Inspection Code "VR" and "NR" rules.

National Board Certificate of Authorization No. 632 to use the "VR" stamp expires April 3, 2000.

National Board Certificate of Authorization No. 81 to use the "NR" stamp expires April 9, 2000.

2/29/00 NWS Technologies, LLC [Signature] Manager, QA
 Date Repair Organization Authorized representative Title

CERTIFICATE OF INSPECTION

I, Joe C. Hair holding a valid commission issued by The National Board of Boiler and Pressure Vessel Inspectors and certificate of competency issued by the jurisdiction of Texas and employed by Hartford Steam Boiler Inspection & Insurance Co. of Hartford, CT have inspected the repair, modification or replacement described in this report on 2/29/00 and state that to the best of my knowledge and belief, this repair, modification or replacement has been completed in accordance with Section XI of the of the ASME Code and the National Board Inspection Code "VR" and "NR" rules.

By signing this certificate, neither the undersigned nor my employer makes any warranty, expressed or implied, concerning this repair, modification or replacement described in this report. Furthermore, neither the undersigned nor my employer shall be liable in any manner for any personal injury, property damage or loss of any kind arising from or connected with this inspection.

2/29/00 Joe C. Hair NB # 9496 N.A.B.I.S.IS Texas 1080
 Date Inspector's Signature Commissions (NB (incl endorsements), jurisdiction, & no.)

First Energy Nuclear Operating, 76 South Main St., Akron, Ohio 44308

Davis-Besse Nuclear Power Station Unit #1, 5501 N. St. Rt. 2, Oak Harbor, Ohio 43449

Commercial Service Date: 11/27/77

FORM NIS-2 / NR-1 / NVR-1 OWNER'S REPORT FOR REPAIRS OR REPLACEMENTS As Required by the Provisions of the ASME Code Section XI

1. Owner Toledo Edison Company Date 4/30/00
Name
300 Madison Ave., Toledo, OH 43652
Address
 Sheet 1 of 1

2. Plant Davis-Besse Nuclear Power Station Unit #1
Name
5501 N. SR 2, Oak Harbor, OH 43449
Address
WO 99-003339-000
Repair Organization P.O. No., Job No., etc.

3. Work Performed By Toledo Edison Company Type Code Stamp NR
Name
5501 N. SR 2, Oak Harbor, OH 43449
Address
 Authorization NO. 20
 Expiration Date 7-17-01

4. Identification of Systems Main Steam System EBB-1-SR4 C-225 (Subsys 083-01)

5. (a) Applicable Construction Code ANSI B31.1 Edition 19 67 Addendum N/A Code Case N/A
 (b) Applicable Edition of Section XI Utilized for Repairs or Replacements 19 86
 (c) Design Responsibilities LISEGA

6. Identification of Components Repaired or Replaced and Replacement Components

Name of Component	Name of Manufacturer	Manufacturer's Serial NO.	National Board NO.	Other Identification/Jurisdictional NO.	Year Built	Repaired Replaced or Replacement	ASME Code Stamped
Snubber	Lisega	61252/53	N/A	C-225	N/A	Replaced	No
Snubber	Lisega	99614610/23	N/A	C-225	N/A	Replacement	No

7. Description of Work

Lisega hydraulic snubber S/N 61252/53 was removed and replaced with a like for like snubber S/N 99614610/23.

Snubber was functionally tested satisfactory prior to installation.

8. Tests Conducted: Hydrostatic (NO) Pneumatic (NO) Nominal Operating Pressure (NO) Other (NO)
 Pressure N/A psi Test Temperature N/A °F

Pressure Relief Valves:
 Service N/A Size N/A
 Opening Pressure N/A Blowdown (if applicable) N/A
 Set Pressure and Blowdown Adjustments Made Using N/A
 at N/A (Post Medium)

NOTE: Supplemental sheets in the form of lists, sketches, or drawings may be used, provided (1) size is 8 1/2 x 11 in. (2) Information in items 1 through 6 on this report is included on each sheet, and (3) each sheet is numbered and the number of sheets is recorded at the top of this form.

First Energy Nuclear Operating, 76 South Main St., Akron, Ohio 44308

Davis-Besse Nuclear Power Station Unit #1, 5501 N. St. Rt. 2, Oak Harbor, Ohio 43449

Commercial Service Date: 11/27/77

FORM NIS-2 /NR-1 /NVR-1	
8. REMARKS NA	<small>Applicable Manufacturers Data Reports to be Attached</small>
<p style="text-align: center;">CERTIFICATE OF COMPLIANCE</p> <p>We certify that the statements made in the report are correct and this <u>replacement</u> conforms to the rules of the ASME Code Section XI.</p> <p>Type Code Symbol <u>NR</u> Certificate of Authorization No. <u>20</u> Expiration Date <u>7-17-01</u></p> <p>Signed <u>[Signature]</u> Nuclear Safety & Inspection Manager Date <u>5/3/2000</u></p> <p style="text-align: center;"><small>Owner or Owner's Designee, Title</small></p>	
<p style="text-align: center;">CERTIFICATE OF INSERVICE INSPECTION</p> <p>I, the undersigned, holding a valid commission issued by the National Board of Boiler and Pressure Vessel Inspectors in the State or Province of <u>Ohio</u> and employed by <u>Factory Mutual Ins. Co.</u> of <u>Johnston, R.I.</u> have inspected the components described in this Owner's Report during the period <u>11/2/99</u> to <u>5/3/00</u>, and state that to the best of my knowledge and belief, the Owner has performed examinations and taken corrective measures described in this Owner's Report in accordance with the requirements of the ASME Code, Section XI.</p> <p>By signing this certificate, neither the Inspector nor his employer makes any warranty, expressed or implied, concerning the examinations and corrective measures described in the Owner's Report. Furthermore, neither the Inspector nor his employer shall be liable in any manner for any personal injury or property damage or a loss of any kind arising from or connected with this inspection.</p> <p>Inspector's Signature <u>Thomas S. Page</u> Date <u>5/3/00</u> Commissions <u>NB 9330</u> <u>N I A</u> <u>OHIO</u></p> <p style="text-align: center;"><small>National Board, State, Province and Endorsements</small></p>	
<p style="text-align: center;">CERTIFICATE OF COMPLIANCE</p> <p>We certify that the statements made in this report are correct and the repair, modification, or replacement of the pressure relief device described above conforms to Section XI and Section III of the ASME Code and the National Board rules as defined in publications NB-65 and NB-102, current editions.</p> <p>Certificate of Authorization No. <u>316</u> to use the "VR" stamp expires <u>Dec. 13</u>, <u>2001</u>.</p> <p>Certificate of Authorization No. <u>20</u> to use the "NR" stamp expires <u>July 17</u>, <u>2001</u>.</p> <p>Date <u> </u>, 19<u> </u>, Signed <u>Toledo Edison</u> Nuclear Safety & Inspection Manager</p> <p style="text-align: center;"><small>Repair Organization Authorized Representative Title</small></p>	
<p style="text-align: center;">CERTIFICATE OF INSPECTION</p> <p>I, the undersigned, holding a valid commission issued by the National Board of Boiler and Pressure Vessel Inspectors, and certificate of competency issued by the State or Province of <u>Ohio</u> and employed by <u>Factory Mutual Ins. Co.</u> of <u>Johnston, R.I.</u> have inspected the repair, modification, or replacement described in this report on <u> </u>, 19<u> </u> and state that to the best of my knowledge and belief, this repair, modification, or replacement has been made or constructed in accordance with Section XI and Section III of the ASME Code and the National Board rules as defined in the publication NB-65 and NB-102, current editions.</p> <p>By signing this certificate, neither the Inspector nor his employer makes any warranty, expressed or implied, concerning the repair, modification or replacement described in this report. Furthermore, neither the Inspector nor his employer shall be liable in any manner for any personal injury or property damage or a loss of any kind arising from or connected with this inspection.</p> <p>Authorized Inspector's Signature <u> </u> Date <u> </u> Commissions <u> </u></p> <p style="text-align: center;"><small>National Board, State, Province and Endorsements</small></p> <p style="text-align: center;">* Factory Mutual Ins. Co</p>	

First Energy Nuclear Operating, 76 South Main St., Akron, Ohio 44308

Davis-Besse Nuclear Power Station Unit #1, 5501 N. St. Rt. 2, Oak Harbor, Ohio 43449

Commercial Service Date: 11/27/77

FORM NIS-2 / NR-1 / NVR-4 OWNER'S REPORT FOR REPAIRS OR REPLACEMENTS
As Required by the Provisions of the ASME Code Section XI

1. Owner Toledo Edison Company Date 4/27/00
Name
300 Madison Ave., Toledo, OH 43652
Address
2. Plant Davis-Besse Nuclear Power Station Unit # 1
Name
5501 N. SR 2, Oak Harbor, OH 43449
Address WO 99-003340-000
Repair Organization P.O. No., Job No., etc.
3. Work Performed By Toledo Edison Company Type Code Stamp NR
Name
5501 N. SR 2, Oak Harbor, OH 43449
Address Authorization NO. 20
 Expiration Date 7-17-01
4. Identification of Systems Main Steam System EBB-1-SR-4 C-170 (Subsys 083-01)
5. (a) Applicable Construction Code ANSI B31.1 Edition 19 67 Addendum N/A Code Case N/A
 (b) Applicable Edition of Section XI Utilized for Repairs or Replacements 19 86
 (c) Design Responsibilities Lisega

6. Identification of Components Repaired or Replaced and Replacement Components

Name of Component	Name of Manufacturer	Manufacturer's Serial NO.	National Board NO.	Other Identification/Jurisdictional NO.	Year Built	Repaired Replaced or Replacement	ASME Code Stamped
Snubber	Lisega	61233/67	N/A	C-170	N/A	Replaced	No
Snubber	Lisega	61252/54	N/A	C-170	N/A	Replacement	No

7. Description of Work

Lisega hydraulic snubber S/N 61233/67 was removed and replaced with a like for like Lisega snubber S/N 61252/54.

Snubber was functionally tested satisfactory prior to installation.

8. Tests Conducted: Hydrostatic (NO) Pneumatic (NO) Nominal Operating Pressure (NO) Other (NO)
 Pressure N/A psi Test Temperature N/A °F
- Pressure Relief Valves:
 Service N/A Size N/A
 Opening Pressure N/A Blowdown (if applicable) N/A
 Set Pressure and Blowdown Adjustments Made Using N/A (Test Medium)
 at N/A (Location)

NOTE: Supplemental sheets in the form of lists, sketches, or drawings may be used, provided (1) size is 8 1/2 x 11 in. (2) information in items 1 through 6 on this report is included on each sheet, and (3) each sheet is numbered and the number of sheets is recorded at the top of this form.

12 RFO Inservice Inspection Summary

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First Energy Nuclear Operating, 76 South Main St., Akron, Ohio 44308

Davis-Besse Nuclear Power Station Unit #1, 5501 N. St. Rt. 2, Oak Harbor, Ohio 43449

Commercial Service Date: 11/27/77

FORM NIS-2 /NR-1 /NVR-4

9. REMARKS

Applicable Manufacturers Data Reports to be Attached

NA

CERTIFICATE OF COMPLIANCE

We certify that the statements made in the report are correct and this replacement conforms to the rules of the ASME Code Section XI.

Type Code Symbol NR Certificate of Authorization No. 20 Expiration Date 7-17-01

Signed [Signature] Nuclear Safety & Inspection Manager Date 4/30/2000
Owner or Owner's Designee, Title

CERTIFICATE OF INSERVICE INSPECTION

I, the undersigned, holding a valid commission issued by the National Board of Boiler and Pressure Vessel Inspectors in the State or Province of Ohio and employed by Factory Mutual Ins. Co. of Johnston, R.I. have inspected the components described in this Owner's Report during the period 11-02-99 to 4-30-00, and state that to the best of my knowledge and belief, the Owner has performed examinations and taken corrective measures described in this Owner's Report in accordance with the requirements of the ASME Code, Section XI.

By signing this certificate, neither the Inspector nor his employer makes any warranty, expressed or implied, concerning the examinations and corrective measures described in the Owner's Report. Furthermore, neither the Inspector nor his employer shall be liable in any manner for any personal injury or property damage or a loss of any kind arising from or connected with this inspection.

[Signature]
Inspector's Signature

4-30-00
Date

Commissions NB 9801 N.I.A. OHIO
National Board, State, Province and Endorsements

CERTIFICATE OF COMPLIANCE

We certify that the statements made in this report are correct and the repair, modification, or replacement of the pressure relief device described above conforms to Section XI and Section III of the ASME Code and the National Board rules as defined in publications NB-65 and NB-102, current editions.

Certificate of Authorization No. 316 to use the "VR" stamp expires Dec. 13, 2001.

Certificate of Authorization No. 20 to use the "NR" stamp expires July 17, 2001.

Date _____, 19____, Signed Toledo Edison [Signature] Nuclear Safety & Inspection Manager
Repair Organization Authorized Representative Title

CERTIFICATE OF INSPECTION

I, the undersigned, holding a valid commission issued by the National Board of Boiler and Pressure Vessel Inspectors, and certificate of competency issued by the State or Province of Ohio and employed by Factory Mutual Ins. Co.

of Johnston, R.I. have inspected the repair, modification, or replacement described in this report

on _____, 19____ and state that to the best of my knowledge and belief, this repair, modification, or replacement has been made or constructed in accordance with Section XI and Section III of the ASME Code and the National Board rules as defined in the publication NB-65 and NB-102, current editions.

By signing this certificate, neither the Inspector nor his employer makes any warranty, expressed or implied, concerning the repair, modification or replacement described in this report. Furthermore, neither the Inspector nor his employer shall be liable in any manner for any personal injury or property damage or a loss of any kind arising from or connected with this inspection.

Authorized Inspector's Signature

Date

Commissions

National Board, State, Province and Endorsements

* Factory Mutual Ins. Co

First Energy Nuclear Operating, 76 South Main St., Akron, Ohio 44308

Davis-Besse Nuclear Power Station Unit #1, 5501 N. St. Rt. 2, Oak Harbor, Ohio 43449

Commercial Service Date: 11/27/77

FORM NIS-2 / NR-1 / NVR-4 OWNER'S REPORT FOR REPAIRS OR REPLACEMENTS As Required by the Provisions of the ASME Code Section XI

1. Owner Toledo Edison Company Date 5/17/00
Name
300 Madison Ave., Toledo, OH 43652
Address

2. Plant Davis-Besse Nuclear Power Station Unit #1
Name
5501 N. SR 2, Oak Harbor, OH 43449
Address

3. Work Performed By Toledo Edison Company Type Code Stamp NR
Name
5501 N. SR 2, Oak Harbor, OH 43449
Address

WO 89-003398-000
Repair Organization P.O. No., Job No., etc.

Authorization NO. 20
 Expiration Date 7-17-01

4. Identification of Systems Pressurizer Code Safety Relief Valve RC13A (Subsys 064-04)

5. (a) Applicable Construction Code ASME III CL1 Edition 19 68 Addendum S70 Code Case 1440
 (b) Applicable Edition of Section XI Utilized for Repairs or Replacements 19 86
 (c) Design Responsibilities Crosby Valve & Gage Co.

6. Identification of Components Repaired or Replaced and Replacement Components

Name of Component	Name of Manufacturer	Manufacturer's Serial NO.	National Board NO.	Other Identification/Jurisdictional NO.	Year Built	Repaired Replaced or Replacement	ASME Code Stamped
Safety Relief Valve	Crosby Valve & Gage	N54891-00-001	N/A	RC13A	1976	Replaced	Yes
Safety Relief Valve	Crosby Valve & Gage	N54891-00-002	N/A	RC13A	1976	Replacement	Yes

7. Description of Work

Removed existing safety relief valve S/N N54891-00-001 and replaced it with a like for like dedicated spare safety relief valve S/N N54891-00-002.

Replacement valve was rebuilt by Toledo Edison Co (WO 89-005152-000) at NWS Industries Inc, Spartanburg, South Carolina.

After installation, a VT-2 leakage test was performed satisfactory.

8. Tests Conducted: Hydrostatic (NO) Pneumatic (NO) Nominal Operating Pressure (YES) Other (NO)
 Pressure 2158 psi Test Temperature NOT °F

Pressure Relief Valves:
 Service N/A Size N/A
 Opening Pressure N/A Blowdown (if applicable) N/A
 Set Pressure and Blowdown Adjustments Made Using N/A
 at N/A

NOTE: Supplemental sheets in the form of lists, sketches, or drawings may be used, provided (1) size is 8 1/2 x 11 in. (2) Information in items 1 through 6 on this report is included on each sheet, and (3) each sheet is numbered and the number of sheets is recorded at the top of this form.

First Energy Nuclear Operating, 76 South Main St., Akron, Ohio 44308

Davis-Besse Nuclear Power Station Unit #1, 5501 N. St. Rt. 2, Oak Harbor, Ohio 43449

Commercial Service Date: 11/27/77

FORM NIS-2 / NR-1 / NVR-4

8. REMARKS

Applicable Manufacturers Data Reports to be Attached

NA

CERTIFICATE OF COMPLIANCE

We certify that the statements made in the report are correct and this replacement conforms to the rules of the ASME Code Section XI.

Type Code Symbol NR Certificate of Authorization No. 20 Expiration Date 7-17-01

Signed G.W. Starnes Nuclear Safety & Inspection Manager Date 5/17/68

Owner or Owner's Designee, Title

CERTIFICATE OF INSERVICE INSPECTION

I, the undersigned, holding a valid commission issued by the National Board of Boiler and Pressure Vessel Inspectors in the State or Province of Ohio and employed by Factory Mutual Ins. Co. of Johnston, R.I. have inspected the components described in this Owner's Report during the period 11-1-99 to 5-17-00, and state that to the best of my knowledge and belief, the Owner has performed examinations and taken corrective measures described in this Owner's Report in accordance with the requirements of the ASME Code, Section XI.

By signing this certificate, neither the Inspector nor his employer makes any warranty, expressed or implied, concerning the examinations and corrective measures described in the Owner's Report. Furthermore, neither the Inspector nor his employer shall be liable in any manner for any personal injury or property damage or a loss of any kind arising from or connected with this inspection.

Inspector's Signature [Signature] Date 5-17-00 Commissions NS9801 N.E.A. Ohio Comm
National Board, State, Province and Endorsements

CERTIFICATE OF COMPLIANCE

We certify that the statements made in this report are correct and the repair, modification, or replacement of the pressure relief device described above conforms to Section XI and Section III of the ASME Code and the National Board rules as defined in publications NB-65 and NB-102, current editions.

Certificate of Authorization No. 316 to use the "VR" stamp expires Dec. 13, 2001

Certificate of Authorization No. 20 to use the "NR" stamp expires July 17, 2001

Date , 19 , Signed Toledo Edison Nuclear Safety & Inspection Manager
Repair Organization Authorized Representative Title

CERTIFICATE OF INSPECTION

I, the undersigned, holding a valid commission issued by the National Board of Boiler and Pressure Vessel Inspectors, and certificate of competency issued by the State or Province of Ohio and employed by Factory Mutual Ins. Co.

of Johnston, R.I. have inspected the repair, modification, or replacement described in this report

on , 19 and state that to the best of my knowledge and belief, this repair, modification, or replacement has been made or constructed in accordance with Section XI and Section III of the ASME Code and the National Board rules as defined in the publication NB-65 and NB-102, current editions.

By signing this certificate, neither the Inspector nor his employer makes any warranty, expressed or implied, concerning the repair, modification or replacement described in this report. Furthermore, neither the Inspector nor his employer shall be liable in any manner for any personal injury or property damage or a loss of any kind arising from or connected with this inspection.

Authorized Inspector's Signature Date Commissions
National Board, State, Province and Endorsements

* Factory Mutual Ins. Co

First Energy Nuclear Operating, 76 South Main St., Akron, Ohio 44308

Davis-Besse Nuclear Power Station Unit #1, 5501 N. St. Rt. 2, Oak Harbor, Ohio 43449

Commercial Service Date: 11/27/77

FORM NIS-2 / NR-1 / NVR-1 OWNER'S REPORT FOR REPAIRS OR REPLACEMENTS As Required by the Provisions of the ASME Code Section XI

1. Owner Toledo Edison Company Date 5/17/00
Name
300 Madison Ave., Toledo, OH 43652
Address Sheet 1 of 1

2. Plant Davis-Besse Nuclear Power Station Unit # 1
Name
5501 N. SR 2, Oak Harbor, OH 43449 WO 89-003399-000
Address Repair Organization P.O. No., Job No., etc.

3. Work Performed By Toledo Edison Company Type Code Stamp NB
Name
5501 N. SR 2, Oak Harbor, OH 43449 Authorization NO. 20
Address Expiration Date 7-17-01

4. Identification of Systems Pressurizer Code Safety Relief Valve RC13B (Subsys 064-04)

5. (a) Applicable Construction Code ASME III CL1 Edition 19 74 Addendum S73 Code Case N/A
 (b) Applicable Edition of Section XI Utilized for Repairs or Replacements 19 86
 (c) Design Responsibilities Crosby Valve & Gage Co.

6. Identification of Components Repaired or Replaced and Replacement Components

Name of Component	Name of Manufacturer	Manufacturer's Serial NO.	National Board NO.	Other Identification/Jurisdictional NO.	Year Built	Repaired Replaced or Replacement	ASME Code Stamped
Safety Relief Valve	Crosby Valve & Gage	N56264-00-0005	N/A	RC13B	1976	Replaced	Yes
Safety Relief Valve	Crosby Valve & Gage	N59303-00-0001	N/A	RC13B	1978	Replacement	Yes

7. Description of Work

Removed existing safety relief valve S/N N56264-00-0005 and replaced it with a like for like dedicated spare safety relief valve S/N N59303-00-0001.
 Replacement valve was rebuilt by Toledo Edison Co (WO 89-005123-000) at NWS Industries Inc, Spartanburg, South Carolina.
 After installation, a VT-2 leakage test was performed satisfactory.

8. Tests Conducted: Hydrostatic (NO) Pneumatic (NO) Nominal Operating Pressure (YES) Other (NO)
 Pressure 2158 psi Test Temperature NOT °F

Pressure Relief Valves:
 Service N/A Size N/A
 Opening Pressure N/A Blowdown (if applicable) N/A
 Set Pressure and Blowdown Adjustments Made Using N/A
 at N/A (Note Medium)

NOTE: Supplemental sheets in the form of lists, sketches, or drawings may be used, provided (1) size is 8 1/2 x 11 in. (2) Information in items 1 through 6 on this report is included on each sheet, and (3) each sheet is numbered and the number of sheets is recorded at the top of this form.

First Energy Nuclear Operating, 76 South Main St., Akron, Ohio 44308

Davis-Besse Nuclear Power Station Unit #1, 5501 N. St. Rt. 2, Oak Harbor, Ohio 43449

Commercial Service Date: 11/27/77

FORM NIS-2 /NR-1 /NVR-1

9. REMARKS

Applicable Manufacturers Data Reports to be Attached

NA

CERTIFICATE OF COMPLIANCE

We certify that the statements made in the report are correct and this replacement conforms to the rules of the ASME Code Section XI.

Type Code Symbol NR Certificate of Authorization No. 20 Expiration Date 7-17-01

Signed D.W. Johnson Nuclear Safety & Inspection Manager Date 5/17/80
Owner or Owner's Designee, Title

CERTIFICATE OF INSERVICE INSPECTION

I, the undersigned, holding a valid commission issued by the National Board of Boiler and Pressure Vessel Inspectors in the State or Province of Ohio and employed by Factory Mutual Ins. Co. of Johnston, R.I. have inspected the components described in this Owner's Report during the period 11-11-79 to 5-17-80, and state that to the best of my knowledge and belief, the Owner has performed examinations and taken corrective measures described in this Owner's Report in accordance with the requirements of the ASME Code, Section XI.

By signing this certificate, neither the Inspector nor his employer makes any warranty, expressed or implied, concerning the examinations and corrective measures described in the Owner's Report. Furthermore, neither the Inspector nor his employer shall be liable in any manner for any personal injury or property damage or a loss of any kind arising from or connected with this inspection.

Inspector's Signature [Signature] Date 5-17-80 Commissions NB 9801 N.E.A. Ohio
National Board, State, Province and Endorsements

CERTIFICATE OF COMPLIANCE

We certify that the statements made in this report are correct and the repair, modification, or replacement of the pressure relief device described above conforms to Section XI and Section III of the ASME Code and the National Board rules as defined in publications NB-65 and NB-102, current editions.

Certificate of Authorization No. 316 to use the "VR" stamp expires Dec. 13, 2001

Certificate of Authorization No. 20 to use the "NR" stamp expires July 17, 2001

Date , 19 , Signed Toledo Edison Nuclear Safety & Inspection Manager
Repair Organization Authorized Representative Title

CERTIFICATE OF INSPECTION

I, the undersigned, holding a valid commission issued by the National Board of Boiler and Pressure Vessel Inspectors, and certificate of competency issued by the State or Province of Ohio and employed by Factory Mutual Ins. Co.

of Johnston, R.I. have inspected the repair, modification, or replacement described in this report

on , 19 and state that to the best of my knowledge and belief, this repair, modification, or replacement has been made or constructed in accordance with Section XI and Section III of the ASME Code and the National Board rules as defined in the publication NB-65 and NB-102, current editions.

By signing this certificate, neither the Inspector nor his employer makes any warranty, expressed or implied, concerning the repair, modification or replacement described in this report. Furthermore, neither the Inspector nor his employer shall be liable in any manner for any personal injury or property damage or a loss of any kind arising from or connected with this inspection.

Authorized Inspector's Signature Date Commissions
National Board, State, Province and Endorsements

* Factory Mutual Ins. Co

First Energy Nuclear Operating, 76 South Main St., Akron, Ohio 44308

Davis-Besse Nuclear Power Station Unit #1, 5501 N. St. Rt. 2, Oak Harbor, Ohio 43449

Commercial Service Date: 11/27/77

FORM NIS-2 / NR-1 / NVR-1 OWNER'S REPORT FOR REPAIRS OR REPLACEMENTS As Required by the Provisions of the ASME Code Section XI

1. Owner Toledo Edison Company Date 5/30/00
Name
300 Madison Ave., Toledo, OH 43652
Address
 Sheet 1 of 1
2. Plant Davis-Besse Nuclear Power Station Unit # 1
Name
5501 N. SR 2, Oak Harbor, OH 43449
Address
WO 99-003666-000
Repair Organization P.O. No., Job No., etc.
3. Work Performed By Toledo Edison Company Type Code Stamp NR
Name
5501 N. SR 2, Oak Harbor, OH 43449
Address
 Authorization NO. 20
 Expiration Date 7-17-01
4. Identification of Systems Reactor Coolant Pump 1-2 P36-3 (Subsys 064-03)
5. (a) Applicable Construction Code ASME III CL1 Edition 19 77 Addendum S7B Code Case N/A
 (b) Applicable Edition of Section XI Utilized for Repairs or Replacements 19 86
 (c) Design Responsibilities BWIP International Inc.

6. Identification of Components Repaired or Replaced and Replacement Components

Name of Component	Name of Manufacturer	Manufacturer's Serial NO.	National Board NO.	Other Identification/ Jurisdictional NO.	Year Built	Repaired Replaced or Replacement	ASME Code Stamped
RCP Seal Cartridge	BWIP International	196985	N/A	P36-3	1998	Replaced	Yes
RCP Seal Cartridge	BWIP International	196486	N/A	P36-3	1998	Replacement	Yes

7. Description of Work

Removed existing seal cartridge S/N 196985 and replaced it with a like for like spare replacement seal cartridge S/N 196486.

Replacement seal cartridge S/N 196486 was rebuilt and tested by WO 98-000404-000 prior to installation.

After installation, a VT-2 leakage test was performed satisfactory.

8. Tests Conducted: Hydrostatic (NO) Pneumatic (NO) Nominal Operating Pressure (YES) Other (NO)
 Pressure 2157 psi Test Temperature NOT °F
- Pressure Relief Valves:
 Service N/A Size N/A
 Opening Pressure N/A Blowdown (if applicable) N/A
 Set Pressure and Blowdown Adjustments Made Using N/A
 at N/A (Test Medium)

NOTE: Supplemental sheets in the form of lists, sketches, or drawings may be used, provided (1) size is 8 1/2 x 11 in. (2) Information in items 1 through 6 on this report is included on each sheet, and (3) each sheet is numbered and the number of sheets is recorded at the top of this form.

12 RFO Inservice Inspection Summary

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First Energy Nuclear Operating, 76 South Main St., Akron, Ohio 44308

Davis-Besse Nuclear Power Station Unit #1, 5501 N. St. Rt. 2, Oak Harbor, Ohio 43449

Commercial Service Date: 11/27/77

FORM NIS-2 / NR-1 / NVR-1

8. REMARKS

Applicable Manufacturers Data Reports to be Attached

NA

CERTIFICATE OF COMPLIANCE

We certify that the statements made in the report are correct and this replacement conforms to the rules of the ASME Code Section XI.

Type Code Symbol NR Certificate of Authorization No. 20 Expiration Date 7-17-01

Signed [Signature] Nuclear Safety & Inspection Manager Date 5/30/00 19__

Owner or Owner's Designee, Title

CERTIFICATE OF INSERVICE INSPECTION

I, the undersigned, holding a valid commission issued by the National Board of Boiler and Pressure Vessel Inspectors in the State or Province of Ohio and employed by Factory Mutual Ins. Co. of Johnston, R.I. have inspected the components described in this Owner's Report during the period 10-21-99 to 6-1-00, and state that to the best of my knowledge and belief, the Owner has performed examinations and taken corrective measures described in this Owner's Report in accordance with the requirements of the ASME Code, Section XI.

By signing this certificate, neither the Inspector nor his employer makes any warranty, expressed or implied, concerning the examinations and corrective measures described in the Owner's Report. Furthermore, neither the Inspector nor his employer shall be liable in any manner for any personal injury or property damage or a loss of any kind arising from or connected with this inspection.

Inspector's Signature [Signature] Date 6-1-00 Commissions NB910 N.I.A. OHIO
National Board, State, Province and Endorsements COAH

CERTIFICATE OF COMPLIANCE

We certify that the statements made in this report are correct and the repair, modification, or replacement of the pressure relief device described above conforms to Section XI and Section III of the ASME Code and the National Board rules as defined in publications NB-65 and NB-102, current editions.

Certificate of Authorization No. 316 to use the "VR" stamp expires Dec. 13, 2001

Certificate of Authorization No. 20 to use the "NR" stamp expires July 17, 2001

Date , 19 , Signed Toledo Edison
Repair Organization Authorized Representative Title
Nuclear Safety & Inspection Manager

CERTIFICATE OF INSPECTION

I, the undersigned, holding a valid commission issued by the National Board of Boiler and Pressure Vessel Inspectors, and certificate of competency issued by the State or Province of Ohio and employed by Factory Mutual Ins. Co.

of Johnston, R.I. have inspected the repair, modification, or replacement described in this report

on , 19 and state that to the best of my knowledge and belief, this repair, modification, or replacement has been made or constructed in accordance with Section XI and Section III of the ASME Code and the National Board rules as defined in the publication NB-65 and NB-102, current editions.

By signing this certificate, neither the Inspector nor his employer makes any warranty, expressed or implied, concerning the repair, modification or replacement described in this report. Furthermore, neither the Inspector nor his employer shall be liable in any manner for any personal injury or property damage or a loss of any kind arising from or connected with this inspection.

Authorized Inspector's Signature Date Commissions
National Board, State, Province and Endorsements

* Factory Mutual Ins. Co

First Energy Nuclear Operating, 76 South Main St., Akron, Ohio 44308

Davis-Besse Nuclear Power Station Unit #1, 5501 N. St. Rt. 2, Oak Harbor, Ohio 43449

Commercial Service Date: 11/27/77

FORM NIS-2 / NR-1 / AWR-1 OWNER'S REPORT FOR REPAIRS OR REPLACEMENTS As Required by the Provisions of the ASME Code Section XI

1. Owner Toledo Edison Company Date 6/2/00
Name
300 Madison Ave., Toledo, OH 43652
Address Sheet 1 of 4

2. Plant Davis-Besse Nuclear Power Station Unit # 1
Name
5501 N. SR 2, Oak Harbor, OH 43449
Address WO 89-4337-008
Repair Organization P.O. No., Job No., etc.

3. Work Performed By Toledo Edison Company Type Code Stamp NR
Name
5501 N. SR 2, Oak Harbor, OH 43449
Address Authorization NO. 20
 Expiration Date 7-17-01

4. Identification of Systems Decay Heat and Low Pressure Injection (Subsys 049-02)

5. (a) Applicable Construction Code ASME III CL2 Edition 19 71 Addendum N/A Code Case N/A
 (b) Applicable Edition of Section XI Utilized for Repairs or Replacements 19 86
 (c) Design Responsibilities Toledo Edison Co. and Velan Inc.

6. Identification of Components Repaired or Replaced and Replacement Components

Name of Component	Name of Manufacturer	Manufacturer's Serial NO.	National Board NO.	Other Identification/Jurisdictional NO.	Year Built	Repaired Replaced or Replacement	ASME Code Stamped
Gate Valve	Velan Inc.	931109-4	N/A	DH-200	1993	Repair	Yes
Gate Valve	Velan Inc.	931109-3	N/A	DH-201	1993	Repair	Yes
Gate Valve	Velan Inc.	2046	N/A	HP-209	2000	Repair	Yes
Gate Valve	Velan Inc.	2045	N/A	HP-210	2000	Repair	Yes

7. Description of Work

Toledo Edison Co implemented Modification 87-0074-00 (Post-LOCA Boric Acid Precipitation control) which involved fabrication and installation of an additional new line to cross-tie High Pressure Injection to Decay Heat. This was to enhance Post-LOCA Boric Acid Precipitation capabilities. This Work Order directed the fabrication and installation of the new piping and valves. Materials installed are referenced and listed in the body of this Work Order. A Hydrostatic Test was performed after the installation. The Hydrostatic Test Pressure for the GCB-10 portion of the system was 650 psig, and the Test Pressure for the CCB-38 & CCB-2 portion was 3260 PSIG.

8. Tests Conducted: Hydrostatic (YES) Pneumatic (NO) Nominal Operating Pressure (NO) Other (NO)
 Pressure See Above psi Test Temperature 75.4 °F

Pressure Relief Valves:
 Service N/A Size N/A
 Opening Pressure N/A Blowdown (if applicable) N/A
 Set Pressure and Blowdown Adjustments Made Using N/A
 at N/A (Test Medium)

NOTE: Supplemental sheets in the form of lists, sketches, or drawings may be used, provided (1) size is 8 1/2 x 11 in. (2) information in items 1 through 6 on this report is included on each sheet, and (3) each sheet is numbered and the number of sheets is recorded at the top of this form.

12 RFO Inservice Inspection Summary

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First Energy Nuclear Operating, 76 South Main St., Akron, Ohio 44308

Davis-Besse Nuclear Power Station Unit #1, 5501 N. St. Rt. 2, Oak Harbor, Ohio 43449

Commercial Service Date: 11/27/77

FORM NIS-2 /NR-1 /NVR-1

8. REMARKS

Applicable Manufacturers Data Reports to be Attached

Velan Inc.

CERTIFICATE OF COMPLIANCE

We certify that the statements made in the report are correct and this repair conforms to the rules of the ASME Code Section XI.

Type Code Symbol NR Certificate of Authorization No. 20 Expiration Date 7-17-01

Signed [Signature] Nuclear Safety & Inspection Manager Date 6/6/2000

Owner or Owner's Designee, Title

CERTIFICATE OF INSERVICE INSPECTION

I, the undersigned, holding a valid commission issued by the National Board of Boiler and Pressure Vessel Inspectors in the State or Province of Ohio and employed by Factory Mutual Ins. Co. of Johnston, R.I. have inspected the components described in this Owner's Report during the period 2/18/00 to 6/8/00, and state that to the best of my knowledge and belief, the Owner has performed examinations and taken corrective measures described in this Owner's Report in accordance with the requirements of the ASME Code, Section XI.

By signing this certificate, neither the Inspector nor his employer makes any warranty, expressed or implied, concerning the examinations and corrective measures described in the Owner's Report. Furthermore, neither the Inspector nor his employer shall be liable in any manner for any personal injury or property damage or a loss of any kind arising from or connected with this inspection.

Inspector's Signature Thomas H. Pope Date 6/8/00 Commissions NB 9330 "N I A" OHIO
National Board, State, Province and Endorsements

CERTIFICATE OF COMPLIANCE

We certify that the statements made in this report are correct and the repair, modification, or replacement of the pressure relief device described above conforms to Section XI and Section III of the ASME Code and the National Board rules as defined in publications NB-65 and NB-102, current editions.

Certificate of Authorization No. 316 to use the "VR" stamp expires Dec. 13, 2001.

Certificate of Authorization No. 20 to use the "NR" stamp expires July 17, 2001.

Date _____, 19____, Signed Toledo Edison Nuclear Safety & Inspection Manager
Repair Organization Authorized Representative Title

CERTIFICATE OF INSPECTION

I, the undersigned, holding a valid commission issued by the National Board of Boiler and Pressure Vessel Inspectors, and certificate of competency issued by the State or Province of Ohio and employed by Factory Mutual Ins. Co.*

of Johnston, R.I. have inspected the repair, modification, or replacement described in this report

on _____, 19____ and state that to the best of my knowledge and belief, this repair, modification, or replacement has been made or constructed in accordance with Section XI and Section III of the ASME Code and the National Board rules as defined in the publication NB-65 and NB-102, current editions.

By signing this certificate, neither the Inspector nor his employer makes any warranty, expressed or implied, concerning the repair, modification or replacement described in this report. Furthermore, neither the Inspector nor his employer shall be liable in any manner for any personal injury or property damage or a loss of any kind arising from or connected with this inspection.

Authorized Inspector's Signature

Date

Commissions

National Board, State, Province and Endorsements

* Factory Mutual Ins. Co

First Energy Nuclear Operating, 76 South Main St., Akron, Ohio 44308

Davis-Besse Nuclear Power Station Unit #1, 5501 N. St. Rt. 2, Oak Harbor, Ohio 43449

Commercial Service Date: 11/27/77

FORM NDT-1 CERTIFICATE: JLDERS' DATA REPORT FOR NUCLEAR PUMPS OR VALVES
As Required by the Provisions of the ASME Code, Section III, Division 1
Pg. 1 of

Pg. 1 of 2

1. Manufactured and certified by VELAN INC. 450 McARTHUR STREET, MONTREAL, QUEBEC H4T 1X8 CANADA **002045**
(name and address of N Certificate Holder)
 2. Manufactured for FIRST ENERGY CORP. 5501 N.STATE ROUTE 2, MS3 OAK HARBOR, OHIO USA
(name and address of Purchaser)
 3. Location of installation THE CLEVELAND ELECT. ILLUMINATING CO. POWER STATION
(name and address)
 4. Model No., Series No., or Type B09-3054B-13MS Drawing P012-909690-N01 Rev. B CRN N/A
 5. ASME Code, Section III, Division 1: 1971 WINTER 1972 2 N/A
(edition) (addenda date) (class) (Code Case no.)
 6. Pump or valve BB GATE VALVE Nominal inlet size 2-1/2" Outlet size 2-1/2"
(in.)
 7. Material: Body SA182 F316 Bonnet SA182 F316 Disk SA182 F316 Bolt/nut SA-564 GR-630(H1100)
(in.)
 & SA-194 GR-8H

[illegible]

* Supplemental information in form of lists, sketches, or drawings may be used provided (1) size is 8 1/2" x 11", (2) information in items 1 through 4 on this Data Report is included on each sheet, (3) each sheet is numbered and the number of sheets is recorded at the top of this form.

(12/88)

This form (E00037) may be obtained from the Order Dept., ASME, 22 Law Drive, Box 2300, Fairfield, NJ 07007-2300.

FORM NPV-1 (Back — Pg. 2 of 2)

PAGE 4

Certificate Holder's Serial No. 002045

8. Design conditions 2500 (pressure) psi 600 (temperature) °F or valve pressure class 1500# (1)
9. Cold working pressure 3750 psi at 100°F
10. Hydrostatic test 5625 (Shell) psi. Disk differential test pressure N/A psi
11. Remarks: _____

CERTIFICATION OF DESIGN

Design Specification certified by PAUL N. CARR P.E. State OHIO USA Reg. No. E-39003
Design Report certified by NOT REQUIRED P.E. State _____ Reg. No. _____

CERTIFICATE OF COMPLIANCE

We certify that the statements made in this report are correct and that this pump or valve conforms to the rules for construction of the ASME Code, Section III, Division 1.

N Certificate of Authorization No. N2797-2 (N) Expires MAY 2, 2001
Date Oct 6/00 Name VELAN INC. Signed [Signature]
(N Certificate Holder) (authorized representative)

CERTIFICATE OF INSPECTION

I, the undersigned, holding a valid commission issued by the National Board of Boiler and Pressure Vessel Inspectors and the State or Province of QUEBEC and employed by REGIE DU BATIMENT DU QUE. of QUEBEC have inspected the pump, or valve, described in this Data Report on April 06/2000, and state that to the best of my knowledge and belief, the Certificate Holder has constructed this pump, or valve, in accordance with the ASME Code, Section III, Division 1.

By signing this certificate, neither the inspector nor his employer makes any warranty, expressed or implied, concerning the component described in this Data Report. Furthermore, neither the inspector nor his employer shall be liable in any manner for any personal injury or property damage or a loss of any kind arising from or connected with this inspection.

Date 02-06 Signed [Signature] Commissions REGIE DU BATIMENT DU QUEBEC QC # 8714
(authorized inspector) (Nat'l. Bd. (incl. endorsements) and state or prov. and no.) (K1)

(1) For manually operated valves only.

First Energy Nuclear Operating, 76 South Main St., Akron, Ohio 44308
Davis-Besse Nuclear Power Station Unit #1, 5501 N. St. Rt. 2, Oak Harbor, Ohio 43449
Commercial Service Date: 11/27/77

FORM NPV-1 CERTIFICA. HOLDERS' DATA REPORT FOR NUCLE. JMPs OR VALVES*

3

As Required by the Provisions of the ASME Code, Section III, Division 1

Pg. 1 of 2

- 350 McARTHUR STREET,
MONTREAL, QUEBEC H4T 1X8 CANADA 002046
1. Manufactured and certified by VELAN INC.
(name and address of N Certificate Holder)
2. Manufactured for FIRST ENERGY CORP. 5501 N.STATE ROUTE 2, MS3 OAK HARBOR, OHIO USA
(name and address of Purchaser) DAVIS-BESSE NUCLEAR
3. Location of installation THE CLEVELAND ELECT. ILLUMINATING CO. POWER STATION
(name and address)
4. Model No., Series No., or Type B09-3054B-13MS Drawing P012-909690-N01 Rev. B CRN N/A
5. ASME Code, Section III, Division 1: 1971 WINTER 1972 2 N/A
(edition) (addenda date) (class) (Code Case no.)
6. Pump or valve BB GATE VALVE Nominal inlet size 2-1/2" Outlet size 2-1/2"
(in.)
7. Material: Body SA182 F316 Bonnet SA182 F316 Disk SA182 F316 Bolting SA-564 GR-630(H1100)
& SA-194 GR-8H

[illegible]

*Supplemental information in form of lists, sketches, or drawings may be used provided (1) size is 8 1/2 x 11, (2) information in Items 1 through 4 on this Data Report is included on each sheet, (3) each sheet is numbered and the number of sheets is recorded at the top of this form.

(12/88)

This form (E00037) may be obtained from the Order Dept., ASME, 22 Law Drive, Box 2300, Fairfield, NJ 07007-2300.

PAGE 4

FORM NPV-1 (Back - Pg. 2 of 2)

Certificate Holder's Serial No. 002046

8. Design conditions 2500 psi 600 °F or valve pressure class 1500# (1)
(pressure) (temperature)

9. Cold working pressure 3750 psi at 100°F

10. Hydrostatic test 5625 psi. Disk differential test pressure N/A psi
(Shell)

11. Remarks: LINE#5, (page-1): EXCEPT MATERIAL SPEC. SUPPLIED IS TO ASME SEC. II
PART-A CODE EDITION 1989; ADDENDA: NONE

CERTIFICATION OF DESIGN

Design Specification certified by PAUL N. CARR P.E. State OHIO USA Reg. no. E-39003
Design Report certified by NOT REQUIRED P.E. State — Reg. no. —

CERTIFICATE OF COMPLIANCE

We certify that the statements made in this report are correct and that this pump or valve conforms to the rules for construction of the ASME Code, Section III, Division 1.

N Certificate of Authorization No. N2797-2 (N) Expires MAY 2, 2001

Date 11/26/00 Name VELAN INC. Signed [Signature]
(N Certificate Holder) (authorized representative)

CERTIFICATE OF INSPECTION

I, the undersigned, holding a valid commission issued by the National Board of Boiler and Pressure Vessel Inspectors and the State or Province of QUEBEC and employed by REGIE DU BATIMENT DU QUE. of QUEBEC have inspected the pump, or valve, described in this Data Report on April 06/2000, and state that to the best of my knowledge and belief, the Certificate Holder has constructed this pump, or valve, in accordance with the ASME Code, Section III, Division 1.

By signing this certificate, neither the inspector nor his employer makes any warranty, expressed or implied, concerning the component described in this Data Report. Furthermore, neither the inspector nor his employer shall be liable in any manner for any personal injury or property damage or a loss of any kind arising from or connected with this inspection.

Date 11-07-06 Signed [Signature] Commissions J. MARCHAND QC # 8714
(Authorized Inspector) (Nat'l. Bd. (incl. endorsements) and state or prov. and no.)

(1) For manually operated valves only.

9.5

Pg. 1 of ____

1. Manufactured and certified by VELAN INC. 2135 WARD AVE. MTL. CANADA
(name and address of N Certificate Holder)
2. Manufactured for B P W NUCLEAR SERVICE
(name and address of Purchaser) *
3. Location of installation N/A
(name and address)
4. Model No., Series No., or Type GATE Drawing PI-76800-1-23 Rev. 2 EX CRN N/A
* 1989 1991 NONE 1 N/A
(edition) (addenda date) (class) (Code Case no.)
5. ASME Code, Section III, Division 1: 1991 NONE 1 N/A
(edition) (addenda date) (class) (Code Case no.)
6. Pump or valve VALVE Nominal inlet size 1 1/2 Outlet size 1 1/2
(in.) (in.)
7. Material: Body SA-182, F316 Bonnet SA-182, F316 Disk SA-351, CFBH Bolting SA-564, 630H
SA-194, GR8M

[illegible]

* Supplemental information in form of lists, sketches, or drawings may be used provided (1) size is 8 1/2 x 11, (2) information in Items 1 through 4 on this Data Report is included on each sheet, (3) each sheet is numbered and the number of sheets is recorded at the top of this form.

rURM NPV-1 (Back - Pg. 2 of ____)

931109 R.6

Certificate Holder's Serial No. _____

8. Design conditions 4055 psi 100 °F or valve pressure class 1690 (1)

9. Cold working pressure 4055 psi at 100°F

10. Hydrostatic test 6075 psi. Disk differential test pressure 4600 psi

* 11. Remarks: VALUES ORIGINALLY CERTIFIED TO
1986 ED

REVISED COPY Blodigian
FEB-23-95

1059224

CERTIFICATION OF DESIGN

Design Specification certified by J. H. FARRELL P.E. State OH Reg. no. 30039
Design Report certified by S. ISBITSKY P.E. State OH Reg. no. 23115

CERTIFICATE OF COMPLIANCE

We certify that the statements made in this report are correct and that this pump or valve conforms to the rules for construction of the ASME Code, Section III, Division 1.

N Certificate of Authorization No. N-2797-1 Expires MAY 2/95

Date May 2/93 Name VEHAN INC. Signed Blodigian
(N Certificate Holder) (authorized representative)

CERTIFICATE OF INSPECTION

I, the undersigned, holding a valid commission issued by the National Board of Boiler and Pressure Vessel Inspectors and the State or Province of Ohio and employed by Seer of Seer have inspected the pump, or valve, described in this Data Report on May-21-93, and state that to the best of my knowledge and belief, the Certificate Holder has constructed this pump, or valve, in accordance with the ASME Code, Section III, Division 1.

By signing this certificate, neither the inspector nor his employer makes any warranty, expressed or implied, concerning the component described in this Data Report. Furthermore, neither the inspector nor his employer shall be liable in any manner for any personal injury or property damage or a loss of any kind arising from or connected with this inspection.

Date 9/3/93 Signed Seer Commissions 211
(Authorized Inspector) (Nat'l Bd. (incl. endorsements) and state or prov. and no.)

(1) For manually operated valves only.

First Energy Nuclear Operating, 76 South Main St., Akron, Ohio 44308

Davis-Besse Nuclear Power Station Unit #1, 5501 N. St. Rt. 2, Oak Harbor, Ohio 43449

Commercial Service Date: 11/27/77

FORM NIS-2 / NR-1 / NVR-4 OWNER'S REPORT FOR REPAIRS OR REPLACEMENTS As Required by the Provisions of the ASME Code Section XI

1. Owner Toledo Edison Company Date 5/30/00
Name
300 Madison Ave., Toledo, OH 43652
Address
2. Plant Davis-Besse Nuclear Power Station Sheet 1 of 1
Name
5501 N. SR 2, Oak Harbor, OH 43449
Address
3. Work Performed By Toledo Edison Company Unit # 1
Name
5501 N. SR 2, Oak Harbor, OH 43449
Address
- WO 99-004337-009
Repair Organization P.O. No., Job No., etc.
- Type Code Stamp NR
- Authorization NO. 20
- Expiration Date 7-17-01
4. Identification of Systems Decay Heat & Low Pressure Injection (Subsys 049-02)
5. (a) Applicable Construction Code ANSI B31.1 Edition 19 87 Addendum N/A Code Case N/A
- (b) Applicable Edition of Section XI Utilized for Repairs or Replacements 19 86
- (c) Design Responsibilities Toledo Edison Co.

6. Identification of Components Repaired or Replaced and Replacement Components

Name of Component	Name of Manufacturer	Manufacturer's Serial NO.	National Board NO.	Other Identification/Jurisdictional NO.	Year Built	Repaired Replaced or Replacement	ASME Code Stamped
Pipe Supports	Toledo Edison Co.	N/A	N/A	N/A	2000	Repaired	No

7. Description of Work

Toledo Edison Co implemented Modification 97-0074-00 which installed a new 2 1/2" pipe line CCB (Boric Acid Precipitation) tied into an existing piping system per WO 99-4337-008. The implementation of this modification also required the fabrication and installation of five new pipe supports per this work order 99-004337-009. All materials installed are referenced and listed in the body of this work order 99-004337-009.

After installation, a VT-3 was performed satisfactory on all five new supports.

8. Tests Conducted: Hydrostatic (NO) Pneumatic (NO) Nominal Operating Pressure (NO) Other (NO)
 Pressure N/A psi Test Temperature N/A °F
- Pressure Relief Valves:
 Service N/A Size N/A
 Opening Pressure N/A Blowdown (if applicable) N/A
 Set Pressure and Blowdown Adjustments Made Using N/A
 at N/A (Test Medium)

NOTE: Supplemental sheets in the form of lists, sketches, or drawings may be used, provided (1) size is 8 1/2 x 11 in. (2) Information in items 1 through 6 on this report is included on each sheet, and (3) each sheet is numbered and the number of sheets is recorded at the top of this form.

First Energy Nuclear Operating, 76 South Main St., Akron, Ohio 44308

Davis-Besse Nuclear Power Station Unit #1, 5501 N. St. Rt. 2, Oak Harbor, Ohio 43449

Commercial Service Date: 11/27/77

FORM NIS-2 /NR-1 /NVR-1

B. REMARKS

Applicable Manufacturers Data Reports to be Attached

N/A

CERTIFICATE OF COMPLIANCE

We certify that the statements made in the report are correct and this repair conforms to the rules of the ASME Code Section XI.

Type Code Symbol NR Certificate of Authorization No. 20 Expiration Date 7-17-01

Signed D. W. Starnes, Jr. Nuclear Safety & Inspection Manager Date 5/30/2000

Owner or Owner's Designee, Title

CERTIFICATE OF INSERVICE INSPECTION

I, the undersigned, holding a valid commission issued by the National Board of Boiler and Pressure Vessel Inspectors in the State or Province of Ohio and employed by Factory Mutual Ins. Co. of Johnston, R.I. have inspected the components described in this Owner's Report during the period 2-17-00 to 6-7-00, and state that to the best of my knowledge and belief, the Owner has performed examinations and taken corrective measures described in this Owner's Report in accordance with the requirements of the ASME Code, Section XI.

By signing this certificate, neither the Inspector nor his employer makes any warranty, expressed or implied, concerning the examinations and corrective measures described in the Owner's Report. Furthermore, neither the Inspector nor his employer shall be liable in any manner for any personal injury or property damage or a loss of any kind arising from or connected with this inspection.

Inspector's Signature

Date 6-7-00

Commissions

NB-6501 *N.I.A.* OH
National Board, State, Province and Endorsements

CERTIFICATE OF COMPLIANCE

We certify that the statements made in this report are correct and the repair, modification, or replacement of the pressure relief device described above conforms to Section XI and Section III of the ASME Code and the National Board rules as defined in publications NB-65 and NB-102, current editions.

Certificate of Authorization No. 316 to use the "VR" stamp expires Dec. 13, 2001.

Certificate of Authorization No. 20 to use the "NR" stamp expires July 17, 2001.

Date _____, 19____, Signed Toledo Edison Nuclear Safety & Inspection Manager
Repair Organization Authorized Representative Title

CERTIFICATE OF INSPECTION

I, the undersigned, holding a valid commission issued by the National Board of Boiler and Pressure Vessel Inspectors, and certificate of competency issued by the State or Province of Ohio and employed by Factory Mutual Ins. Co. of Johnston, R.I. have inspected the repair, modification, or replacement described in this report

on _____, 19____ and state that to the best of my knowledge and belief, this repair, modification, or replacement has been made or constructed in accordance with Section XI and Section III of the ASME Code and the National Board rules as defined in the publication NB-65 and NB-102, current editions.

By signing this certificate, neither the Inspector nor his employer makes any warranty, expressed or implied, concerning the repair, modification or replacement described in this report. Furthermore, neither the Inspector nor his employer shall be liable in any manner for any personal injury or property damage or a loss of any kind arising from or connected with this inspection.

Authorized Inspector's Signature

Date

Commissions

National Board, State, Province and Endorsements

* Factory Mutual Ins. Co

First Energy Nuclear Operating, 76 South Main St., Akron, Ohio 44308

Davis-Besse Nuclear Power Station Unit #1, 5501 N. St. Rt. 2, Oak Harbor, Ohio 43449

Commercial Service Date: 11/27/77

FORM NIS-2 / NR-1 / NVR-1 OWNER'S REPORT FOR REPAIRS OR REPLACEMENTS As Required by the Provisions of the ASME Code Section XI

1. Owner Toledo Edison Company Date 10/26/1999
Name
300 Madison Ave., Toledo, OH 43652
Address
 2. Plant Davis-Besse Nuclear Power Station Unit # 1
Name
5501 N. SR 2, Oak Harbor, OH 43449
Address
 3. Work Performed By Toledo Edison Company Type Code Stamp NR / VR
Name
5501 N. SR 2, Oak Harbor, OH 43449
Address
 Authorization NO. 20 / 316
 Expiration Date 7-17-01 / 12-13-01

4. Identification of Systems REACTOR COOLANT SYSTEM SPARE PRESSURIZER SAFETY VALVE (RC13B)
 5. (a) Applicable Construction Code ASME III CL1 Edition 19 74 Addendum 74 Code Case NA
 (b) Applicable Edition of Section XI Utilized for Repairs or Replacements 19 86
 (c) Design Responsibilities CROSBY VALVE & GAGE COMPANY

6. Identification of Components Repaired or Replaced and Replacement Components

Name of Component	Name of Manufacturer	Manufacturer's Serial NO.	National Board NO.	Other Identification/Jurisdictional NO.	Year Built	Repaired Replaced or Replacement	ASME Code Stamped
DISC INSERT	CROSBY	NA	NA	N94141-33-0004	NA	Replacement	No
SPRG WSR ASSY	CROSBY	NA	NA	K58655-34-0004	NA	Replacement	No

7. Description of Work

SPARE PRESSURIZER SAFETY VALVE RC13B, IDENTIFIED BY SERIAL NUMBER N59303-00-0001, WAS REBUILT REPLACING THE DISC INSERT AND SPRING WASHER ASSEMBLY. THE VALVE WAS THEN SET PRESSURE TESTED WITHIN NORMAL OPERATING PRESSURE AND TEMPERATURE RANGES. VALVE TESTED SATISFACTORY.

DISC INSERT - PN # N94141-33-0004, PO # 80047948 D93, ASME III 1974 ED, 1974 ADD, CLASS 1

SPRING WASHER ASSEMBLY - PO # 8087724 D98, SN K58655-34-0004, ASME III 1974 ED, 1974 ADD, CLASS 1

8. Tests Conducted: Hydrostatic (NO) Pneumatic (NO) Nominal Operating Pressure (YES) Other (NO)
 Pressure 2500 psi Test Temperature 483 °F
 Pressure Relief Valves:
 Service STEAM Size 4X M1 X6
 Opening Pressure 2515 Blowdown (if applicable) NA
 Set Pressure and Blowdown Adjustments Made Using SATURATED STEAM
 at NWS TECHNOLOGIES, INC., SPARTANBURG, SOUTH CAROLINA

NOTE: Supplemental sheets in the form of lists, sketches, or drawings may be used, provided (1) size is 8 1/2 x 11 in. (2) information in items 1 through 6 on this report is included on each sheet, and (3) each sheet is numbered and the number of sheets is recorded at the top of this form.

FORM NIS-2 / NR-1 / NVR-1

8. REMARKS

Applicable Manufacturers Data Reports to be Attached

NA

CERTIFICATE OF COMPLIANCE

We certify that the statements made in the report are correct and this repair conforms to the rules of the ASME Code Section XI.

Type Code Symbol NR Certificate of Authorization No. 20 Expiration Date 7-17-01

Signed W. N. Starn Nuclear Safety & Inspection Manager Date 11/8 19 99

Owner or Owner's Designee, Title

CERTIFICATE OF INSERVICE INSPECTION

I, the undersigned, holding a valid commission issued by the National Board of Boiler and Pressure Vessel Inspectors in the State or Province of Ohio and employed by Protection Mutual of Park Ridge, Illinois have inspected the components described in this Owner's Report during the period 9/21/89 to 11/9/99, and state that to the best of my knowledge and belief, the Owner has performed examinations and taken corrective measures described in this Owner's Report in accordance with the requirements of the ASME Code, Section XI.

By signing this certificate, neither the Inspector nor his employer makes any warranty, expressed or implied, concerning the examinations and corrective measures described in the Owner's Report. Furthermore, neither the Inspector nor his employer shall be liable in any manner for any personal injury or property damage or a loss of any kind arising from or connected with this inspection.

Thomas H. Lape 11/9/99
 Inspector's Signature Date

Commissions NB 9330 "N I A" OHIO COMM.
 National Board, State, Province and Endorsements

CERTIFICATE OF COMPLIANCE

We certify that the statements made in this report are correct and the repair, modification, or replacement of the pressure relief device described above conforms to Section XI and Section III of the ASME Code and the National Board rules as defined in publications NB-65 and NB-102, current editions.

Certificate of Authorization No. 316 to use the "VR" stamp expires Dec. 13, 2001

Certificate of Authorization No. 20 to use the "NR" stamp expires July 17, 2001

Date 11/8, 19 99, Signed Toledo Edison W. N. Starn Nuclear Safety & Inspection Manager
 Repair Organization Authorized Representative Title

CERTIFICATE OF INSPECTION

I, the undersigned, holding a valid commission issued by the National Board of Boiler and Pressure Vessel Inspectors, and certificate of competency issued by the State or Province of Ohio and employed by Protection Mutual

of Park Ridge, Illinois have inspected the repair, modification, or replacement described in this report

on Nov. 9, 19 99 and state that to the best of my knowledge and belief, this repair, modification, or replacement has been made or constructed in accordance with Section XI and Section III of the ASME Code and the National Board rules as defined in the publication NB-65 and NB-102, current editions.

By signing this certificate, neither the Inspector nor his employer makes any warranty, expressed or implied, concerning the repair, modification or replacement described in this report. Furthermore, neither the Inspector nor his employer shall be liable in any manner for any personal injury or property damage or a loss of any kind arising from or connected with this inspection.

Thomas H. Lape 11/9/99
 Authorized Inspector's Signature Date

Commissions NB 9330 "N I A" OHIO COMM.
 National Board, State, Province and Endorsements

* Factory Mutual Engineering Association

* Factory Mutual Ins. Co.

First Energy Nuclear Operating, 76 South Main St., Akron, Ohio 44308

Davis-Besse Nuclear Power Station Unit #1, 5501 N. St. Rt. 2, Oak Harbor, Ohio 43449

Commercial Service Date: 11/27/77

FORM NIS-2 / NR-1 / NVR-4 OWNER'S REPORT FOR REPAIRS OR REPLACEMENTS As Required by the Provisions of the ASME Code Section XI

1. Owner Toledo Edison Company Date 3/16/00
Name
300 Madison Ave., Toledo, OH 43652
Address
 Sheet 1 of 1

2. Plant Davis-Besse Nuclear Power Station Unit # 1
Name
5501 N. SR 2, Oak Harbor, OH 43449
Address
WO 99-005147-056
Repair Organization P.O. No., Job No., etc.

3. Work Performed By Toledo Edison Company Type Code Stamp NR
Name
5501 N. SR 2, Oak Harbor, OH 43449
Address
 Authorization NO. 20
 Expiration Date 7-17-01

4. Identification of Systems Containment Spray System 33A-HCB-3-H15 A-97 (Subsys 061-01)

5. (a) Applicable Construction Code ANSI B31.1 Edition 19 67 Addendum N/A Code Case N/A
 (b) Applicable Edition of Section XI Utilized for Repairs or Replacements 19 86
 (c) Design Responsibilities ITT Grinnell

6. Identification of Components Repaired or Replaced and Replacement Components

Name of Component	Name of Manufacturer	Manufacturer's Serial NO.	National Board NO.	Other Identification/Jurisdictional NO.	Year Built	Repaired Replaced or Replacement	ASME Code Stamped
Snubber	ITT Grinnell	30513	N/A	A-97	Unk	Replaced	No
Snubber	ITT Grinnell	17655	N/A	A-97	Unk	Replacement	No

7. Description of Work

ITT Grinnell hydraulic snubber S/N 30513 was removed and replaced with a like-for-like snubber S/N 17655.

Replacement snubber S/N 17655 was rebuilt by WO 99-005150-001. Replacement snubber was functionally tested satisfactory prior to installation.

8. Tests Conducted: Hydrostatic (NO) Pneumatic (NO) Nominal Operating Pressure (NO) Other (NO)
 Pressure N/A psi Test Temperature N/A °F

Pressure Relief Valves:
 Service N/A Size N/A
 Opening Pressure N/A Blowdown (if applicable) N/A
 Set Pressure and Blowdown Adjustments Made Using N/A
 at N/A (Test Medium)

NOTE: Supplemental sheets in the form of lists, sketches, or drawings may be used, provided (1) size is 8 1/2 x 11 in. (2) Information in items 1 through 6 on this report is included on each sheet, and (3) each sheet is numbered and the number of sheets is recorded at the top of this form.

12 RFO Inservice Inspection Summary

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First Energy Nuclear Operating, 76 South Main St., Akron, Ohio 44308

Davis-Besse Nuclear Power Station Unit #1, 5501 N. St. Rt. 2, Oak Harbor, Ohio 43449

Commercial Service Date: 11/27/77

FORM NIS-2 /NR-1 /NVR-4-

8. REMARKS

Applicable Manufacturers Data Reports to be Attached

NA

CERTIFICATE OF COMPLIANCE

We certify that the statements made in the report are correct and this replacement conforms to the rules of the ASME Code Section XI.

Type Code Symbol NR Certificate of Authorization No. 20 Expiration Date 7-17-01
 Signed J. W. Starn Quality Assurance Date 3/16 2000
 of Nuclear Safety & Inspection Manager
 Owner or Owner's Designee, Title

CERTIFICATE OF INSERVICE INSPECTION

I, the undersigned, holding a valid commission issued by the National Board of Boiler and Pressure Vessel Inspectors in the State or Province of Ohio and employed by Protection Mutual of Park Ridge, Illinois FACTORY MUTUAL INS. CO. JOHNSTON, R.I. have inspected the components described in this Owner's Report during the period 2/10/00 to 3/20/00, and state that to the best of my knowledge and belief, the Owner has performed examinations and taken corrective measures described in this Owner's Report in accordance with the requirements of the ASME Code, Section XI.

By signing this certificate, neither the Inspector nor his employer makes any warranty, expressed or implied, concerning the examinations and corrective measures described in the Owner's Report. Furthermore, neither the Inspector nor his employer shall be liable in any manner for any personal injury or property damage or a loss of any kind arising from or connected with this inspection.

Thomas J. Loya 3/20/00 Commissions NB 9338 "N" "I" "A" OHIO
 Inspector's Signature Date National Board, State, Province and Endorsements COMM.

CERTIFICATE OF COMPLIANCE

We certify that the statements made in this report are correct and the repair, modification, or replacement of the pressure relief device described above conforms to Section XI and Section III of the ASME Code and the National Board rules as defined in publications NB-65 and NB-102, current editions.

Certificate of Authorization No. 316 to use the "VR" stamp expires Dec. 19, 2001

Certificate of Authorization No. 20 to use the "NR" stamp expires July 17, 2001

Date 19, 1999, Signed Toledo Edison Nuclear Safety & Inspection Manager
 Repair Organization Authorized Representative Title

CERTIFICATE OF INSPECTION

I, the undersigned, holding a valid commission issued by the National Board of Boiler and Pressure Vessel Inspectors, and

certificate of competency issued by the State or Province of Ohio and employed by Protection Mutual

of Park Ridge, Illinois have inspected the repair, modification, or replacement described in this report

on 3/20/00, 1999 and state that to the best of my knowledge and belief, this repair, modification, or replacement has been made or constructed in accordance with Section XI and Section III of the ASME Code and the National Board rules as defined in the publication NB-65 and NB-102, current editions.

By signing this certificate, neither the Inspector nor his employer makes any warranty, expressed or implied, concerning the repair, modification or replacement described in this report. Furthermore, neither the Inspector nor his employer shall be liable in any manner for any personal injury or property damage or a loss of any kind arising from or connected with this inspection.

Authorized Inspector's Signature _____ Date _____ Commissions _____
 National Board, State, Province and Endorsements

* Factory Mutual Engineering Association -
Tel 312/000

* FACTORY MUTUAL INS. CO.

First Energy Nuclear Operating, 76 South Main St., Akron, Ohio 44308

Davis-Besse Nuclear Power Station Unit #1, 5501 N. St. Rt. 2, Oak Harbor, Ohio 43449

Commercial Service Date: 11/27/77

FORM NIS-2 / NR-1 / NVR-1 OWNER'S REPORT FOR REPAIRS OR REPLACEMENTS As Required by the Provisions of the ASME Code Section XI

1. Owner Toledo Edison Company Date 5/16/00
Name
300 Madison Ave., Toledo, OH 43652
Address
2. Plant Davis-Besse Nuclear Power Station Unit # 1
Name
5501 N. SR 2, Oak Harbor, OH 43449
Address WO 89-005148-002
Repair Organization P.O. No., Job No., etc.
3. Work Performed By Toledo Edison Company Type Code Stamp NR
Name
5501 N. SR 2, Oak Harbor, OH 43449
Address Authorization NO. 20
 Expiration Date 7-17-01
4. Identification of Systems Aux Feed Pump and Piping 6C-EBB-4-H15 C-186 (Subsys 050-03)
5. (a) Applicable Construction Code ANSI B31.1 Edition 19 87 Addendum N/A Code Case N/A
 (b) Applicable Edition of Section XI Utilized for Repairs or Replacements 19 86
 (c) Design Responsibilities ITT Grinnell

6. Identification of Components Repaired or Replaced and Replacement Components

Name of Component	Name of Manufacturer	Manufacturer's Serial NO.	National Board NO.	Other Identification/Jurisdictional NO.	Year Built	Repaired Replaced or Replacement	ASME Code Stamped
Snubber	ITT Grinnell	19191	N/A	C-186	N/A	Replaced	No
Snubber	ITT Grinnell	74440215	N/A	C-186	N/A	Replacement	N/A

7. Description of Work

Existing ITT Grinnell hydraulic snubber S/N 19191 was removed and replaced with a like for like snubber S/N 74440215.

Replacement snubber was functionally test satisfactory prior to installation.

8. Tests Conducted: Hydrostatic (NO) Pneumatic (NO) Nominal Operating Pressure (NO) Other (NO)
 Pressure N/A psi Test Temperature N/A °F
- Pressure Relief Valves:
 Service N/A Size N/A
 Opening Pressure N/A Blowdown (if applicable) N/A
 Set Pressure and Blowdown Adjustments Made Using N/A (Test Medium)
 at N/A (Location)

NOTE: Supplemental sheets in the form of lists, sketches, or drawings may be used, provided (1) size is 8 1/2 x 11 in. (2) Information in items 1 through 6 on this report is included on each sheet, and (3) each sheet is numbered and the number of sheets is recorded at the top of this form.

FORM NIS-2 / NR-1 / NVR-1	
<p style="text-align: center; margin: 0;">9. REMARKS</p> <p style="margin: 0;">NA</p>	<p style="text-align: right; font-size: small; margin: 0;">Applicable Manufacturers Data Reports to be Attached</p>
<p style="text-align: center; margin: 0;">CERTIFICATE OF COMPLIANCE</p> <p style="margin: 5px 0;">We certify that the statements made in the report are correct and this <u>replacement</u> conforms to the rules of the ASME Code Section XI.</p> <p style="margin: 5px 0;">Type Code Symbol <u>NR</u> Certificate of Authorization No. <u>20</u> Expiration Date <u>7-17-01</u></p> <p style="margin: 5px 0;">Signed <u>D.W. Starnes</u> Nuclear Safety & Inspection Manager Date <u>5/17/03</u></p> <p style="text-align: center; font-size: x-small; margin: 0;">Owner or Owner's Designee, Title</p>	
<p style="text-align: center; margin: 0;">CERTIFICATE OF INSERVICE INSPECTION</p> <p style="margin: 5px 0;">I, the undersigned, holding a valid commission issued by the National Board of Boiler and Pressure Vessel Inspectors in the State or Province of <u>Ohio</u> and employed by <u>Factory Mutual Ins. Co. of Johnston, R.I.</u> have inspected the components described in this Owner's Report during the period <u>9-21-99</u> to <u>5-17-00</u>, and state that to the best of my knowledge and belief, the Owner has performed examinations and taken corrective measures described in this Owner's Report in accordance with the requirements of the ASME Code, Section XI.</p> <p style="margin: 5px 0;">By signing this certificate, neither the Inspector nor his employer makes any warranty, expressed or implied, concerning the examinations and corrective measures described in the Owner's Report. Furthermore, neither the Inspector nor his employer shall be liable in any manner for any personal injury or property damage or a loss of any kind arising from or connected with this inspection.</p> <p style="margin: 5px 0;">Inspector's Signature <u>[Signature]</u> Date <u>5-17-00</u> Commissions <u>NBSPV "N.I.A." Ohio</u> <small>National Board, State, Province and Endorsements</small></p>	
<p style="text-align: center; margin: 0;">CERTIFICATE OF COMPLIANCE</p> <p style="margin: 5px 0;">We certify that the statements made in this report are correct and the repair, modification, or replacement of the pressure relief device described above conforms to Section XI and Section III of the ASME Code and the National Board rules as defined in publications NB-65 and NB-102, current editions.</p> <p style="margin: 5px 0;">Certificate of Authorization No. <u>316</u> to use the "VR" stamp expires <u>Dec. 13</u>, <u>2001</u></p> <p style="margin: 5px 0;">Certificate of Authorization No. <u>20</u> to use the "NR" stamp expires <u>July 17</u>, <u>2001</u></p> <p style="margin: 5px 0;">Date <u> </u>, 19 <u> </u>, Signed <u>Toledo Edison</u> Nuclear Safety & Inspection Manager <small>Repair Organization Authorized Representative Title</small></p>	
<p style="text-align: center; margin: 0;">CERTIFICATE OF INSPECTION</p> <p style="margin: 5px 0;">I, the undersigned, holding a valid commission issued by the National Board of Boiler and Pressure Vessel Inspectors, and certificate of competency issued by the State or Province of <u>Ohio</u> and employed by <u>Factory Mutual Ins. Co.</u> of <u>Johnston, R.I.</u> have inspected the repair, modification, or replacement described in this report on <u> </u>, 19 <u> </u> and state that to the best of my knowledge and belief, this repair, modification, or replacement has been made or constructed in accordance with Section XI and Section III of the ASME Code and the National Board rules as defined in the publication NB-65 and NB-102, current editions.</p> <p style="margin: 5px 0;">By signing this certificate, neither the Inspector nor his employer makes any warranty, expressed or implied, concerning the repair, modification or replacement described in this report. Furthermore, neither the Inspector nor his employer shall be liable in any manner for any personal injury or property damage or a loss of any kind arising from or connected with this inspection.</p> <p style="margin: 5px 0;">Authorized Inspector's Signature <u> </u> Date <u> </u> Commissions <u> </u> <small>National Board, State, Province and Endorsements</small></p> <p style="text-align: center; margin: 5px 0;">* Factory Mutual Ins. Co</p>	

FORM NIS-2 / NR-1 / NVR-1 OWNER'S REPORT FOR REPAIRS OR REPLACEMENTS
 As Required by the Provisions of the ASME Code Section XI

1. Owner Toledo Edison Company Date 4/29/00
Name
300 Madison Ave., Toledo, OH 43652
Address
 Sheet 1 of 1

2. Plant Davis-Besse Nuclear Power Station Unit # 1
Name
5501 N. SR 2, Oak Harbor, OH 43449
Address
WO 89-005148-006
Repair Organization P.O. No., Job No., etc.

3. Work Performed By Toledo Edison Company Type Code Stamp NR
Name
5501 N. SR 2, Oak Harbor, OH 43449
Address
 Authorization NO. 20
 Expiration Date 7-17-01

4. Identification of Systems Pressurizer Surge Line PSU-1-R2 C-204 (Subsys 064-04)

5. (a) Applicable Construction Code ANSI B31.1 Edition 19 87 Addendum N/A Code Case N/A
 (b) Applicable Edition of Section XI Utilized for Repairs or Replacements 19 86
 (c) Design Responsibilities ITT Grinnell

6. Identification of Components Repaired or Replaced and Replacement Components

Name of Component	Name of Manufacturer	Manufacturer's Serial NO.	National Board NO.	Other Identification/Jurisdictional NO.	Year Built	Repaired Replaced or Replacement	ASME Code Stamped
Snubber	ITT Grinnell	74402393-22	N/A	C-204	N/A	Replaced	No
Snubber	ITT Grinnell	12477	N/A	C-204	N/A	Replacement	No

7. Description of Work

ITT Grinnell hydraulic snubber S/N 7442393-22 was removed and replaced with a like for like snubber S/N 12477.
 Snubber was functionally tested satisfactory prior to installation.

8. Tests Conducted: Hydrostatic (NO) Pneumatic (NO) Nominal Operating Pressure (NO) Other (NO)
 Pressure N/A psi Test Temperature N/A °F

Pressure Relief Valves:
 Service N/A Size N/A
 Opening Pressure N/A Blowdown (if applicable) N/A
 Set Pressure and Blowdown Adjustments Made Using N/A
 at N/A (Test Medium)

NOTE: Supplemental sheets in the form of lists, sketches, or drawings may be used, provided (1) size is 8 1/2 x 11 in.
 (2) Information in items 1 through 6 on this report is included on each sheet, and (3) each sheet is numbered and the number of sheets is recorded at the top of this form.

12 RFO Inservice Inspection Summary

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First Energy Nuclear Operating, 76 South Main St., Akron, Ohio 44308

Davis-Besse Nuclear Power Station Unit #1, 5501 N. St. Rt. 2, Oak Harbor, Ohio 43449

Commercial Service Date: 11/27/77

FORM NIS-2 / NR-1 / NVR-4

9. REMARKS

Applicable Manufacturers Data Reports to be Attached

NA

CERTIFICATE OF COMPLIANCE

We certify that the statements made in the report are correct and this replacement conforms to the rules of the ASME Code Section XI.

Type Code Symbol NR Certificate of Authorization No. 20 Expiration Date 7-17-01

Signed J. N. Starnes, Jr. Nuclear Safety & Inspection Manager Date 4/30/2000
Owner or Owner's Designee, Title

CERTIFICATE OF INSERVICE INSPECTION

I, the undersigned, holding a valid commission issued by the National Board of Boiler and Pressure Vessel Inspectors in the State or Province of Ohio and employed by Factory Mutual Ins. Co. of Johnston, R.I. have inspected the components described in this Owner's Report during the period 4-23-99 to 4-30-00, and state that to the best of my knowledge and belief, the Owner has performed examinations and taken corrective measures described in this Owner's Report in accordance with the requirements of the ASME Code, Section XI.

By signing this certificate, neither the Inspector nor his employer makes any warranty, expressed or implied, concerning the examinations and corrective measures described in the Owner's Report. Furthermore, neither the Inspector nor his employer shall be liable in any manner for any personal injury or property damage or a loss of any kind arising from or connected with this inspection.

Inspector's Signature [Signature] Date 4-30-00 Commissions NBSPV1 N E A Ohio
National Board, State, Province and Endorsements

CERTIFICATE OF COMPLIANCE

We certify that the statements made in this report are correct and the repair, modification, or replacement of the pressure relief device described above conforms to Section XI and Section III of the ASME Code and the National Board rules as defined in publications NB-65 and NB-102, current editions.

Certificate of Authorization No. 316 to use the "VR" stamp expires Dec. 13, 2001.

Certificate of Authorization No. 20 to use the "NR" stamp expires July 17, 2001.

Date _____, 19____, Signed Toledo Edison Nuclear Safety & Inspection Manager
Repair Organization Authorized Representative Title

CERTIFICATE OF INSPECTION

I, the undersigned, holding a valid commission issued by the National Board of Boiler and Pressure Vessel Inspectors, and certificate of competency issued by the State or Province of Ohio and employed by Factory Mutual Ins. Co.*

of Johnston, R.I. have inspected the repair, modification, or replacement described in this report

on _____, 19____ and state that to the best of my knowledge and belief, this repair, modification, or replacement has been made or constructed in accordance with Section XI and Section III of the ASME Code and the National Board rules as defined in the publication NB-65 and NB-102, current editions.

By signing this certificate, neither the Inspector nor his employer makes any warranty, expressed or implied, concerning the repair, modification or replacement described in this report. Furthermore, neither the Inspector nor his employer shall be liable in any manner for any personal injury or property damage or a loss of any kind arising from or connected with this inspection.

Authorized Inspector's Signature _____ Date _____ Commissions _____
National Board, State, Province and Endorsements

* Factory Mutual Ins. Co

12 RFO Inservice Inspection Summary
 First Energy Nuclear Operating, 76 South Main St., Akron, Ohio 44308
 Davis-Besse Nuclear Power Station Unit #1, 5501 N. St. Rt. 2, Oak Harbor, Ohio 43449
 Commercial Service Date: 11/27/77

FORM NIS-2 / NR-1 / NVR-4 OWNER'S REPORT FOR REPAIRS OR REPLACEMENTS
 As Required by the Provisions of the ASME Code Section XI

1. Owner Toledo Edison Company Date 4/18/00
 Name 300 Madison Ave., Toledo, OH 43652 Sheet 1 of 1
 Address
 2. Plant Davis-Besse Nuclear Power Station Unit # 1
 Name 5501 N. SR 2, Oak Harbor, OH 43449 WO 99-005148-010
 Address Repair Organization P.O. No., Job No., etc.
 3. Work Performed By Toledo Edison Company Type Code Stamp NR
 Name 5501 N. SR 2, Oak Harbor, OH 43449 Authorization NO. 20
 Address Expiration Date 7-17-01
 4. Identification of Systems Decay Heat & Low Pressure Injection 33B-CCB-6-H2 C-277 (Subsys 049-02)
 5. (a) Applicable Construction Code ANSI B31.1 Edition 19 87 Addendum N/A Code Case N/A
 (b) Applicable Edition of Section XI Utilized for Repairs or Replacements 19 86
 (c) Design Responsibilities ITT Grinnell

6. Identification of Components Repaired or Replaced and Replacement Components

Name of Component	Name of Manufacturer	Manufacturer's Serial NO.	National Board NO.	Other Identification/Jurisdictional NO.	Year Built	Repaired Replaced or Replacement	ASME Code Stamped
Snubber	ITT Grinnell	9872	N/A	C-277	N/A	Replaced	No
Snubber	ITT Grinnell	12036	N/A	C-277	N/A	Replacement	No

7. Description of Work

ITT Grinnell hydraulic snubber S/N 9872 was removed and replaced with a like for like snubber S/N 12036.
 Replacement snubber was functionally tested satisfactory prior to installation.

8. Tests Conducted: Hydrostatic (NO) Pneumatic (NO) Nominal Operating Pressure (NO) Other (NO)
 Pressure N/A psi Test Temperature N/A °F
 Pressure Relief Valves:
 Service N/A Size N/A
 Opening Pressure N/A Blowdown (if applicable) N/A
 Set Pressure and Blowdown Adjustments Made Using N/A
 at N/A (Test Medium)

NOTE: Supplemental sheets in the form of lists, sketches, or drawings may be used, provided (1) size is 8 1/2 x 11 in. (2) Information in items 1 through 6 on this report is included on each sheet, and (3) each sheet is numbered and the number of sheets is recorded at the top of this form.

First Energy Nuclear Operating, 76 South Main St., Akron, Ohio 44308

Davis-Besse Nuclear Power Station Unit #1, 5501 N. St. Rt. 2, Oak Harbor, Ohio 43449

Commercial Service Date: 11/27/77

FORM NIS-2 /NR-1 /NVR-4

9. REMARKS

Applicable Manufacturers Data Reports to be Attached

NA

CERTIFICATE OF COMPLIANCE

We certify that the statements made in the report are correct and this replacement conforms to the rules of the ASME Code Section XI.

Type Code Symbol NR Certificate of Authorization No. 20 Expiration Date 7-17-01

Signed AKNS [Signature] Nuclear Safety & Inspection Manager Date 4/19 19 2000

Owner or Owner's Designee, Title

CERTIFICATE OF INSERVICE INSPECTION

I, the undersigned, holding a valid commission issued by the National Board of Boiler and Pressure Vessel Inspectors in the State or Province of Ohio and employed by Factory Mutual Ins. Co. of Johnston, R.I. have inspected the components described in this Owner's Report during the period 9/23/99 to 4/19/00, and state that to the best of my knowledge and belief, the Owner has performed examinations and taken corrective measures described in this Owner's Report in accordance with the requirements of the ASME Code, Section XI.

By signing this certificate, neither the Inspector nor his employer makes any warranty, expressed or implied, concerning the examinations and corrective measures described in the Owner's Report. Furthermore, neither the Inspector nor his employer shall be liable in any manner for any personal injury or property damage or a loss of any kind arising from or connected with this inspection.

Inspector's Signature

Thomas E. Laps

Date

4/19/00

Commissions

NB 9330"N I A"OHIO

National Board, State, Province and Endorsements

CERTIFICATE OF COMPLIANCE

We certify that the statements made in this report are correct and the repair, modification, or replacement of the pressure relief device described above conforms to Section XI and Section III of the ASME Code and the National Board rules as defined in publications NB-65 and NB-102, current editions.

Certificate of Authorization No. 316 to use the "VR" stamp expires Dec. 13, 2001

Certificate of Authorization No. 20 to use the "NR" stamp expires July 17, 2001

Date _____, 19____, Signed Toledo Edison

Repair Organization

Authorized Representative

Nuclear Safety & Inspection Manager

Title

CERTIFICATE OF INSPECTION

I, the undersigned, holding a valid commission issued by the National Board of Boiler and Pressure Vessel Inspectors, and certificate of competency issued by the State or Province of Ohio and employed by Factory Mutual Ins. Co.*

of Johnston, R.I. have inspected the repair, modification, or replacement described in this report

on _____, 19____ and state that to the best of my knowledge and belief, this repair, modification, or replacement has been made or constructed in accordance with Section XI and Section III of the ASME Code and the National Board rules as defined in the publication NB-65 and NB-102, current editions.

By signing this certificate, neither the Inspector nor his employer makes any warranty, expressed or implied, concerning the repair, modification or replacement described in this report. Furthermore, neither the Inspector nor his employer shall be liable in any manner for any personal injury or property damage or a loss of any kind arising from or connected with this inspection.

Authorized Inspector's Signature

Date

Commissions

National Board, State, Province and Endorsements

* Factory Mutual Ins. Co

First Energy Nuclear Operating, 76 South Main St., Akron, Ohio 44308

Davis-Besse Nuclear Power Station Unit #1, 5501 N. St. Rt. 2, Oak Harbor, Ohio 43449

Commercial Service Date: 11/27/77

FORM NIS-2 / NR-1 / NVR-1 OWNER'S REPORT FOR REPAIRS OR REPLACEMENTS As Required by the Provisions of the ASME Code Section XI

1. Owner Toledo Edison Company Date 5/2/00
Name
300 Madison Ave., Toledo, OH 43652
Address Sheet 1 of 1
2. Plant Davis-Besse Nuclear Power Station Unit # 1
Name
5501 N. SR 2, Oak Harbor, OH 43449 WO 99-005148-012
Address Repair Organization P.O. No., Job No., etc.
3. Work Performed By Toledo Edison Company Type Code Stamp NR
Name
5501 N. SR 2, Oak Harbor, OH 43449 Authorization NO. 20
Address Expiration Date 7-17-01
4. Identification of Systems Reactor Coolant Pumps RCP 3-5 (E/W) C-304 (Subsys 064-03)
5. (a) Applicable Construction Code ANSI B31.1 Edition 19 67 Addendum N/A Code Case N/A
 (b) Applicable Edition of Section XI Utilized for Repairs or Replacements 19 86
 (c) Design Responsibilities ITT Grinnell

6. Identification of Components Repaired or Replaced and Replacement Components

Name of Component	Name of Manufacturer	Manufacturer's Serial NO.	National Board NO.	Other Identification/Jurisdictional NO.	Year Built	Repaired Replaced or Replacement	ASME Code Stamped
Snubber	ITT Grinnell	15104	N/A	C-304	N/A	Replaced	No
Snubber	ITT Grinnell	15105	N/A	C-304	N/A	Replacement	No

7. Description of Work

Existing ITT Grinnell hydraulic snubber S/N 15104 was removed and replaced with a like for like snubber S/N 15105.
 Replacement snubber was functionally tested satisfactory prior to installation.

8. Tests Conducted: Hydrostatic (NO) Pneumatic (NO) Nominal Operating Pressure (NO) Other (NO)
 Pressure N/A psi Test Temperature N/A °F
- Pressure Relief Valves:
 Service N/A Size N/A
 Opening Pressure N/A Blowdown (if applicable) N/A
 Set Pressure and Blowdown Adjustments Made Using N/A
 at N/A (Check Medium)

NOTE: Supplemental sheets in the form of lists, sketches, or drawings may be used, provided (1) size is 8 1/2 x 11 in. (2) Information in items 1 through 6 on this report is included on each sheet, and (3) each sheet is numbered and the number of sheets is recorded at the top of this form.

FORM NIS-2 /NR-1 /NVR-4	
B. REMARKS	Applicable Manufacturer Data Reports to be Attached
CERTIFICATE OF COMPLIANCE	
We certify that the statements made in the report are correct and this <u>replacement</u> conforms to the rules of the ASME Code Section XI.	
Type Code Symbol <u>NR</u>	Certificate of Authorization No. <u>20</u> Expiration Date <u>7-17-01</u>
Signed <u>[Signature]</u> Nuclear Safety & Inspection Manager Date <u>5/3/2000</u> <small>Owner or Owner's Designee Title</small>	
CERTIFICATE OF INSERVICE INSPECTION	
I, the undersigned, holding a valid commission issued by the National Board of Boiler and Pressure Vessel Inspectors in the State or Province of <u>Ohio</u> and employed by <u>Factory Mutual Ins. Co. of Johnston, R.I.</u> have inspected the components described in this Owner's Report during the period <u>9/23/99 to 5/3/00</u> , and state that to the best of my knowledge and belief, the Owner has performed examinations and taken corrective measures described in this Owner's Report in accordance with the requirements of the ASME Code, Section XI.	
By signing this certificate, neither the Inspector nor his employer makes any warranty, expressed or implied, concerning the examinations and corrective measures described in the Owner's Report. Furthermore, neither the Inspector nor his employer shall be liable in any manner for any personal injury or property damage or a loss of any kind arising from or connected with this inspection.	
Inspector's Signature <u>Thomas J. Lape</u> Date <u>5/3/00</u>	Commissions <u>NB 9330 NIA OHIO</u> National Board, State, Province and Endorsements
CERTIFICATE OF COMPLIANCE	
We certify that the statements made in this report are correct and the repair, modification, or replacement of the pressure relief device described above conforms to Section XI and Section III of the ASME Code and the National Board rules as defined in publications NB-65 and NB-102, current editions.	
Certificate of Authorization No. <u>316</u> to use the "VR" stamp expires <u>Dec. 13</u> <u>2001</u>	
Certificate of Authorization No. <u>20</u> to use the "NR" stamp expires <u>July 17</u> <u>2001</u>	
Date _____, 19____, Signed <u>Toledo Edison</u>	Nuclear Safety & Inspection Manager <small>Title</small>
<small>Repair Organization</small>	<small>Authorized Representative</small>
CERTIFICATE OF INSPECTION	
I, the undersigned, holding a valid commission issued by the National Board of Boiler and Pressure Vessel Inspectors, and certificate of competency issued by the State or Province of <u>Ohio</u> and employed by <u>Factory Mutual Ins. Co.*</u> of <u>Johnston, R.I.</u> have inspected the repair, modification, or replacement described in this report on _____, 19____ and state that to the best of my knowledge and belief, this repair, modification, or replacement has been made or constructed in accordance with Section XI and Section III of the ASME Code and the National Board rules as defined in the publication NB-65 and NB-102, current editions.	
By signing this certificate, neither the Inspector nor his employer makes any warranty, expressed or implied, concerning the repair, modification or replacement described in this report. Furthermore, neither the Inspector nor his employer shall be liable in any manner for any personal injury or property damage or a loss of any kind arising from or connected with this inspection.	
Authorized Inspector's Signature _____	Commissions _____ National Board, State, Province and Endorsements
<small>Date</small>	<small>* Factory Mutual Ins. Co</small>

First Energy Nuclear Operating, 76 South Main St., Akron, Ohio 44308

Davis-Besse Nuclear Power Station Unit #1, 5501 N. St. Rt. 2, Oak Harbor, Ohio 43449

Commercial Service Date: 11/27/77

FORM NIS-2 / NR-1 / NVR-4 OWNER'S REPORT FOR REPAIRS OR REPLACEMENTS As Required by the Provisions of the ASME Code Section XI

1. Owner Toledo Edison Company Date 5/3/00
Name
300 Madison Ave., Toledo, OH 43652
Address Sheet 1 of 1
2. Plant Davis-Besse Nuclear Power Station Unit # 1
Name
5501 N. SR 2, Oak Harbor, OH 43449
Address WO 99-005148-014
Repair Organization P.O. No., Job No., etc.
3. Work Performed By Toledo Edison Company Type Code Stamp NR
Name
5501 N. SR 2, Oak Harbor, OH 43449
Address Authorization NO. 20
Expiration Date 7-17-01
4. Identification of Systems Reactor Coolant Pump RCP3-6 (N/S) C-305 (Subsys 064-03)
5. (a) Applicable Construction Code ANSI B31.1 Edition 19 87 Addendum N/A Code Case N/A
(b) Applicable Edition of Section XI Utilized for Repairs or Replacements 19 86
(c) Design Responsibilities ITT Grinnell

6. Identification of Components Repaired or Replaced and Replacement Components

Name of Component	Name of Manufacturer	Manufacturer's Serial NO.	National Board NO.	Other Identification/Jurisdictional NO.	Year Built	Repaired Replaced or Replacement	ASME Code Stamped
Snubber	ITT Grinnell	15102	N/A	C-305	N/A	Replaced	No
Snubber	ITT Grinnell	15103	N/A	C-305	N/A	Replacement	No

7. Description of Work

Existing ITT Grinnell hydraulic snubber S/N 15102 was removed and replaced with a like for like snubber S/N 15103.
Replacement snubber was functionally tested satisfactory prior to installation.

8. Tests Conducted: Hydrostatic (NO) Pneumatic (NO) Nominal Operating Pressure (NO) Other (NO)
Pressure N/A psi Test Temperature N/A °F
- Pressure Relief Valves:
Service N/A Size N/A
Opening Pressure N/A Blowdown (if applicable) N/A
Set Pressure and Blowdown Adjustments Made Using N/A
at N/A (Test Medium)

NOTE: Supplemental sheets in the form of lists, sketches, or drawings may be used, provided (1) size is 8 1/2 x 11 in. (2) information in items 1 through 6 on this report is included on each sheet, and (3) each sheet is numbered and the number of sheets is recorded at the top of this form.

12 RFO Inservice Inspection Summary

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First Energy Nuclear Operating, 76 South Main St., Akron, Ohio 44308

Davis-Besse Nuclear Power Station Unit #1, 5501 N. St. Rt. 2, Oak Harbor, Ohio 43449

Commercial Service Date: 11/27/77

FORM NIS-2 /NR-1 /NVR-4

8. REMARKS

Applicable Manufacturers Data Reports to be Attached

N/A

CERTIFICATE OF COMPLIANCE

We certify that the statements made in the report are correct and this replacement conforms to the rules of the ASME Code Section XI.

Type Code Symbol NR Certificate of Authorization No. 20 Expiration Date 7-17-01

Signed [Signature] Nuclear Safety & Inspection Manager Date 5/3/2000

Owner or Owner's Designee, Title

CERTIFICATE OF INSERVICE INSPECTION

I, the undersigned, holding a valid commission issued by the National Board of Boiler and Pressure Vessel Inspectors in the State or Province of Ohio and employed by Factory Mutual Ins. Co. of Johnston, R.I. have inspected the components described in this Owner's Report during the period 9/23/99 to 5/3/00, and state that to the best of my knowledge and belief, the Owner has performed examinations and taken corrective measures described in this Owner's Report in accordance with the requirements of the ASME Code, Section XI.

By signing this certificate, neither the Inspector nor his employer makes any warranty, expressed or implied, concerning the examinations and corrective measures described in the Owner's Report. Furthermore, neither the Inspector nor his employer shall be liable in any manner for any personal injury or property damage or a loss of any kind arising from or connected with this inspection.

Thomas H. Page
Inspector's Signature

5/3/00
Date

Commissions NB 9330 N I A OHIO
National Board, State, Province and Endorsements

CERTIFICATE OF COMPLIANCE

We certify that the statements made in this report are correct and the repair, modification, or replacement of the pressure relief device described above conforms to Section XI and Section III of the ASME Code and the National Board rules as defined in publications NB-65 and NB-102, current editions.

Certificate of Authorization No. 316 to use the "VR" stamp expires Dec. 13, 2001.

Certificate of Authorization No. 20 to use the "NR" stamp expires July 17, 2001.

Date _____, 19____, Signed Toledo Edison Nuclear Safety & Inspection Manager
Repair Organization Authorized Representative Title

CERTIFICATE OF INSPECTION

I, the undersigned, holding a valid commission issued by the National Board of Boiler and Pressure Vessel Inspectors, and certificate of competency issued by the State or Province of Ohio and employed by Factory Mutual Ins. Co.

of Johnston, R.I. have inspected the repair, modification, or replacement described in this report

on _____, 19____ and state that to the best of my knowledge and belief, this repair, modification, or replacement has been made or constructed in accordance with Section XI and Section III of the ASME Code and the National Board rules as defined in the publication NB-65 and NB-102, current editions.

By signing this certificate, neither the Inspector nor his employer makes any warranty, expressed or implied, concerning the repair, modification or replacement described in this report. Furthermore, neither the Inspector nor his employer shall be liable in any manner for any personal injury or property damage or a loss of any kind arising from or connected with this inspection.

Authorized Inspector's Signature

Date

Commissions _____
National Board, State, Province and Endorsements

* Factory Mutual Ins. Co

First Energy Nuclear Operating, 76 South Main St., Akron, Ohio 44308

Davis-Besse Nuclear Power Station Unit #1, 5501 N. St. Rt. 2, Oak Harbor, Ohio 43449

Commercial Service Date: 11/27/77

FORM NIS-2 / NR-1 / NVR-1 OWNER'S REPORT FOR REPAIRS OR REPLACEMENTS
As Required by the Provisions of the ASME Code Section XI

1. Owner Toledo Edison Company
Name
300 Madison Ave., Toledo, OH 43652
Address

Date 5/2/00

Sheet 1 of 1

2. Plant Davis-Besse Nuclear Power Station
Name
6501 N. SR 2, Oak Harbor, OH 43449
Address

Unit 1

WO 99-005148-016

Repair Organization P.O. No., Job No., etc

3. Work Performed By Toledo Edison Company
5501 N. SR 2, Oak Harbor, OH 43449

Type Code Stamp NR

Authorization NO. 20

Expiration Date 7-17-01

4. Identification of Systems MAIN STEAM SYSTEM EBB-1-SR8 WDR TOP (C-230) (SUB 083-01)

5. (a) Applicable Construction Code ANSI B31.1 Edition 19 67 Addendum N/A Code Case N/A

(b) Applicable Edition of Section XI Utilized for Repairs or Replacements 19 86

(c) Design Responsibilities LISEGA

6. Identification of Components Repaired or Replaced and Replacement Components					

6. Identification of Components Repaired or Replaced and Replacement Components							
Name of Component	Name of Manufacturer	Manufacturer's Serial NO.	National Board NO.	Other Identification/Jurisdictional NO.	Year Built	Repaired Replaced or Replacement	ASME Code Stamped
Snubber	LISEGA	61232/35	N/A	C-230	Unk	Replaced	No
Snubber	LISEGA	98614150/02	N/A	C-230	Unk	Replacement	No

7. Description of Work

7. Description of work
 Lieaga hydraulic snubber S/N61232/35 was removed and replaced with a like-for-like snubber S/N98614150/02.

Replacement snubber was functionally tested satisfactory prior to installation.

8. Tests Conducted: Hydrostatic (NO) Pneumatic (NO)
Pressure N/A psi

Nominal Operating Pressure (NO) Other (NO)
Test Temperature N/A °F

Pressure Relief Valves:

Service N/A

Opening Pressure N/A

Set Pressure and Blowdown Adjustments Made Using N/A

at N/A

Size N/A

Blowdown (if applicable) N/A

N/A

NOTE: Supplemental sheets in the form of lists, sketches, or drawings may be used, provided (1) size is 8 1/2 x 11 in. (2) Information in items 1 through 6 on this report is included on each sheet, and (3) each sheet is numbered and the number of sheets is recorded at the top of this form.

First Energy Nuclear Operating, 76 South Main St., Akron, Ohio 44308

Davis-Besse Nuclear Power Station Unit #1, 5501 N. St. Rt. 2, Oak Harbor, Ohio 43449

Commercial Service Date: 11/27/77

FORM NIS-2 /NR-1 / NVR-1

8. REMARKS

Applicable Manufacturers Data Reports to be Attached

NA

CERTIFICATE OF COMPLIANCE

We certify that the statements made in the report are correct and this replacement conforms to the rules of the ASME Code Section XI.

Type Code Symbol NR Certificate of Authorization No. 20 Expiration Date 7-17-01

Signed D.W. Starnes Nuclear Safety & Inspection Manager Date 5/2/2000
Owner or Owner's Designee, Title

CERTIFICATE OF INSERVICE INSPECTION

I, the undersigned, holding a valid commission issued by the National Board of Boiler and Pressure Vessel Inspectors in the State or Province of Ohio and employed by Factory Mutual Ins. Co. of Johnston, R.I. have inspected the components described in this Owner's Report during the period 4-27-99 to 5-2-00, and state that to the best of my knowledge and belief, the Owner has performed examinations and taken corrective measures described in this Owner's Report in accordance with the requirements of the ASME Code, Section XI.

By signing this certificate, neither the Inspector nor his employer makes any warranty, expressed or implied, concerning the examinations and corrective measures described in the Owner's Report. Furthermore, neither the Inspector nor his employer shall be liable in any manner for any personal injury or property damage or a loss of any kind arising from or connected with this inspection.

Inspector's Signature

Date

Commissions NB9808 N.E.A. Ohio
National Board, State, Province and Endorsements

CERTIFICATE OF COMPLIANCE

We certify that the statements made in this report are correct and the repair, modification, or replacement of the pressure relief device described above conforms to Section XI and Section III of the ASME Code and the National Board rules as defined in publications NB-65 and NB-102, current editions.

Certificate of Authorization No. 316 to use the "VR" stamp expires Dec. 13, 2001

Certificate of Authorization No. 20 to use the "NR" stamp expires July 17, 2001

Date _____, 19____, Signed Toledo Edison Nuclear Safety & Inspection Manager
Repair Organization Authorized Representative Title

CERTIFICATE OF INSPECTION

I, the undersigned, holding a valid commission issued by the National Board of Boiler and Pressure Vessel Inspectors, and certificate of competency issued by the State or Province of Ohio and employed by Factory Mutual Ins. Co.

of Johnston, R.I. have inspected the repair, modification, or replacement described in this report

on _____, 19____ and state that to the best of my knowledge and belief, this repair, modification, or replacement has been made or constructed in accordance with Section XI and Section III of the ASME Code and the National Board rules as defined in the publication NB-65 and NB-102, current editions.

By signing this certificate, neither the Inspector nor his employer makes any warranty, expressed or implied, concerning the repair, modification or replacement described in this report. Furthermore, neither the Inspector nor his employer shall be liable in any manner for any personal injury or property damage or a loss of any kind arising from or connected with this inspection.

Authorized Inspector's Signature

Date

Commissions

National Board, State, Province and Endorsements

* Factory Mutual Ins. Co

First Energy Nuclear Operating, 76 South Main St., Akron, Ohio 44308

Davis-Besse Nuclear Power Station Unit #1, 5501 N. St. Rt. 2, Oak Harbor, Ohio 43449

Commercial Service Date: 11/27/77

FORM NIS-2 / NR-1 / NVR-4 OWNER'S REPORT FOR REPAIRS OR REPLACEMENTS As Required by the Provisions of the ASME Code Section XI

1. Owner Toledo Edison Company Date 5/2/00
Name
300 Madison Ave., Toledo, OH 43652
Address
 Sheet 1 of 1

2. Plant Davis-Besse Nuclear Power Station Unit # 1
Name
5501 N. SR 2, Oak Harbor, OH 43449
Address
WO 89-005148-018
Repair Organization P.O. No., Job No., etc.

3. Work Performed By Toledo Edison Company Type Code Stamp NR
Name
5501 N. SR 2, Oak Harbor, OH 43449
Address
 Authorization NO. 20
 Expiration Date 7-17-01

4. Identification of Systems Main Steam System EBB-1-SR8 C-176 (Subsys 083-01)

5. (a) Applicable Construction Code ANSI B31.1 Edition 19 67 Addendum N/A Code Case N/A
 (b) Applicable Edition of Section XI Utilized for Repairs or Replacements 19 86
 (c) Design Responsibilities Lisega

6. Identification of Components Repaired or Replaced and Replacement Components

Name of Component	Name of Manufacturer	Manufacturer's Serial NO.	National Board NO.	Other Identification/Jurisdictional NO.	Year Built	Repaired Replaced or Replacement	ASME Code Stamped
Snubber	Lisega	61232/36	N/A	C-176	N/A	Replaced	No
Snubber	Lisega	98614150/03	N/A	C-176	N/A	Replacement	No

7. Description of Work

Existing Lisega hydraulic snubber S/N 61232/36 was removed and replaced with a like for like snubber S/N 98614150/03.

Replacement snubber was functionally tested satisfactory prior to installation.

8. Tests Conducted: Hydrostatic (NO) Pneumatic (NO) Nominal Operating Pressure (NO) Other (NO)
 Pressure N/A psi Test Temperature N/A °F

Pressure Relief Valves:
 Service N/A Size N/A
 Opening Pressure N/A Blowdown (if applicable) N/A
 Set Pressure and Blowdown Adjustments Made Using N/A
 at N/A (Test Medium)

NOTE: Supplemental sheets in the form of lists, sketches, or drawings may be used, provided (1) size is 8 1/2 x 11 in.
 (2) Information in items 1 through 6 on this report is included on each sheet, and (3) each sheet is numbered and the number of sheets is recorded at the top of this form.

First Energy Nuclear Operating, 76 South Main St., Akron, Ohio 44308

Davis-Besse Nuclear Power Station Unit #1, 5501 N. St. Rt. 2, Oak Harbor, Ohio 43449

Commercial Service Date: 11/27/77

FORM NIS-2 / NR-1 / NVR-4		
9. REMARKS		Applicable Manufacturers Data Reports to be Attached
NA		
CERTIFICATE OF COMPLIANCE		
<p>We certify that the statements made in the report are correct and this <u>replacement</u> conforms to the rules of the ASME Code Section XI.</p> <p>Type Code Symbol <u>NR</u> Certificate of Authorization No. <u>20</u> Expiration Date <u>7-17-01</u></p> <p>Signed <u>SW Starnes, Jr.</u> Nuclear Safety & Inspection Manager Date <u>5/3/2000</u></p> <p style="text-align: center; font-size: small;">Owner or Owner's Designee, Title</p>		
CERTIFICATE OF INSERVICE INSPECTION		
<p>I, the undersigned, holding a valid commission issued by the National Board of Boiler and Pressure Vessel Inspectors in the State or Province of <u>Ohio</u> and employed by <u>Factory Mutual Ins. Co. of Johnston, R.I.</u> have inspected the components described in this Owner's Report during the period <u>9/23/99</u> to <u>5/3/00</u>, and state that to the best of my knowledge and belief, the Owner has performed examinations and taken corrective measures described in this Owner's Report in accordance with the requirements of the ASME Code, Section XI.</p> <p>By signing this certificate, neither the Inspector nor his employer makes any warranty, expressed or implied, concerning the examinations and corrective measures described in the Owner's Report. Furthermore, neither the Inspector nor his employer shall be liable in any manner for any personal injury or property damage or a loss of any kind arising from or connected with this inspection.</p> <p>Inspector's Signature <u>Thomas E. Lige</u> Date <u>5/3/00</u> Commissions <u>NB 9338 "N" "A" OHIO COMM.</u></p> <p style="text-align: center; font-size: small;">National Board, State, Province and Endorsements</p>		
CERTIFICATE OF COMPLIANCE		
<p>We certify that the statements made in this report are correct and the repair, modification, or replacement of the pressure relief device described above conforms to Section XI and Section III of the ASME Code and the National Board rules as defined in publications NB-65 and NB-102, current editions.</p> <p>Certificate of Authorization No. <u>316</u> to use the "VR" stamp expires <u>Dec. 13</u>, <u>2001</u></p> <p>Certificate of Authorization No. <u>20</u> to use the "NR" stamp expires <u>July 17</u>, <u>2001</u></p> <p>Date <u> </u>, 19<u> </u>, Signed <u>Toledo Edison</u> Nuclear Safety & Inspection Manager</p> <p style="text-align: center; font-size: small;">Repair Organization Authorized Representative Title</p>		
CERTIFICATE OF INSPECTION		
<p>I, the undersigned, holding a valid commission issued by the National Board of Boiler and Pressure Vessel Inspectors, and certificate of competency issued by the State or Province of <u>Ohio</u> and employed by <u>Factory Mutual Ins. Co. of Johnston, R.I.</u> have inspected the repair, modification, or replacement described in this report on <u> </u>, 19<u> </u> and state that to the best of my knowledge and belief, this repair, modification, or replacement has been made or constructed in accordance with Section XI and Section III of the ASME Code and the National Board rules as defined in the publication NB-65 and NB-102, current editions.</p> <p>By signing this certificate, neither the Inspector nor his employer makes any warranty, expressed or implied, concerning the repair, modification or replacement described in this report. Furthermore, neither the Inspector nor his employer shall be liable in any manner for any personal injury or property damage or a loss of any kind arising from or connected with this inspection.</p> <p>Authorized Inspector's Signature <u> </u> Date <u> </u> Commissions <u> </u></p> <p style="text-align: center; font-size: small;">National Board, State, Province and Endorsements</p> <p style="text-align: center;">* Factory Mutual Ins. Co</p>		

First Energy Nuclear Operating, 76 South Main St., Akron, Ohio 44308

Davis-Besse Nuclear Power Station Unit #1, 5501 N. St. Rt. 2, Oak Harbor, Ohio 43449

Commercial Service Date: 11/27/77

FORM NIS-2 / NR-1 / NVR-4 OWNER'S REPORT FOR REPAIRS OR REPLACEMENTS As Required by the Provisions of the ASME Code Section XI

1. Owner Toledo Edison Company Date 4/19/00
Name
300 Madison Ave., Toledo, OH 43652
Address

2. Plant Davis-Besse Nuclear Power Station Unit # 1
Name
5501 N. SR 2, Oak Harbor, OH 43449
Address W.O. 99-005148-020
Repair Organization P.O. No., Job No., etc.

3. Work Performed By Toledo Edison Company Type Code Stamp NR
Name
5501 N. SR 2, Oak Harbor, OH 43449
Address Authorization NO. 20
 Expiration Date 7-17-01

4. Identification of Systems Steam Generator #2 Drain Line EBB-5-12-H2 C-455 (Subsys 063-01)

5. (a) Applicable Construction Code ANSI B31.1 Edition 19 67 Addendum N/A Code Case N/A
 (b) Applicable Edition of Section XI Utilized for Repairs or Replacements 19 85
 (c) Design Responsibilities Lisega

6. Identification of Components Repaired or Replaced and Replacement Components

Name of Component	Name of Manufacturer	Manufacturer's Serial NO.	National Board NO.	Other Identification/Jurisdictional NO.	Year Built	Repaired Replaced or Replacement	ASME Code Stamped
Snubber	Lisega	61195/84	N/A	C-455	N/A	Replaced	No
Snubber	Lisega	61195/52	N/A	C-455	N/A	Replacement	No

7. Description of Work

Lisega hydraulic snubber S/N 61195/84 was removed and replaced with a like for like Lisega snubber S/N 61195/52.
 Snubber was functionally tested satisfactory prior to installation.

8. Tests Conducted: Hydrostatic (NO) Pneumatic (NO) Nominal Operating Pressure (NO) Other (NO)
 Pressure N/A psi Test Temperature N/A °F

Pressure Relief Valves:
 Service N/A Size N/A
 Opening Pressure N/A Blowdown (if applicable) N/A
 Set Pressure and Blowdown Adjustments Made Using N/A
 at N/A (Test Medium)

NOTE: Supplemental sheets in the form of lists, sketches, or drawings may be used, provided (1) size is 8 1/2 x 11 in. (2) Information in items 1 through 6 on this report is included on each sheet, and (3) each sheet is numbered and the number of sheets is recorded at the top of this form.

First Energy Nuclear Operating, 76 South Main St., Akron, Ohio 44308

Davis-Besse Nuclear Power Station Unit #1, 5501 N. St. Rt. 2, Oak Harbor, Ohio 43449

Commercial Service Date: 11/27/77

FORM NIS-2 / NR-1 / NVR-1	
8. REMARKS	Applicable Manufacturers Data Reports to be Attached
NA	
CERTIFICATE OF COMPLIANCE	
<p>We certify that the statements made in the report are correct and this <u>replacement</u> conforms to the rules of the ASME Code Section XI.</p> <p>Type Code Symbol <u>NR</u> Certificate of Authorization No. <u>20</u> Expiration Date <u>7-17-01</u></p> <p>Signed <u>[Signature]</u> Nuclear Safety & Inspection Manager Date <u>4/20</u> 19 <u>2000</u></p> <p style="text-align: center; font-size: small;">Owner or Owner's Designee, Title</p>	
CERTIFICATE OF INSERVICE INSPECTION	
<p>I, the undersigned, holding a valid commission issued by the National Board of Boiler and Pressure Vessel Inspectors in the State or Province of <u>Ohio</u> and employed by <u>Factory Mutual Ins. Co.</u> of <u>Johnston, R.I.</u> have inspected the components described in this Owner's Report during the period <u>9-23-99</u> to <u>4-20-00</u>, and state that to the best of my knowledge and belief, the Owner has performed examinations and taken corrective measures described in this Owner's Report in accordance with the requirements of the ASME Code, Section XI.</p> <p>By signing this certificate, neither the Inspector nor his employer makes any warranty, expressed or implied, concerning the examinations and corrective measures described in the Owner's Report. Furthermore, neither the Inspector nor his employer shall be liable in any manner for any personal injury or property damage or a loss of any kind arising from or connected with this inspection.</p> <p>Inspector's Signature <u>[Signature]</u> Date <u>4-20-00</u> Commissions <u>NB99-01</u> <u>NIA</u> <u>OHIO</u></p> <p style="text-align: center; font-size: small;">National Board, State, Province and Endorsements</p>	
CERTIFICATE OF COMPLIANCE	
<p>We certify that the statements made in this report are correct and the repair, modification, or replacement of the pressure relief device described above conforms to Section XI and Section III of the ASME Code and the National Board rules as defined in publications NB-65 and NB-102, current editions.</p> <p>Certificate of Authorization No. <u>316</u> to use the "VR" stamp expires <u>Dec. 13</u>, 2001</p> <p>Certificate of Authorization No. <u>20</u> to use the "NR" stamp expires <u>July 17</u>, 2001</p> <p>Date _____, 19____, Signed <u>Toledo Edison</u> <u>[Signature]</u> Nuclear Safety & Inspection Manager</p> <p style="text-align: center; font-size: small;">Repair Organization Authorized Representative Title</p>	
CERTIFICATE OF INSPECTION	
<p>I, the undersigned, holding a valid commission issued by the National Board of Boiler and Pressure Vessel Inspectors, and certificate of competency issued by the State or Province of <u>Ohio</u> and employed by <u>Factory Mutual Ins. Co.</u> of <u>Johnston, R.I.</u> have inspected the repair, modification, or replacement described in this report on _____, 19____ and state that to the best of my knowledge and belief, this repair, modification, or replacement has been made or constructed in accordance with Section XI and Section III of the ASME Code and the National Board rules as defined in the publication NB-65 and NB-102, current editions.</p> <p>By signing this certificate, neither the Inspector nor his employer makes any warranty, expressed or implied, concerning the repair, modification or replacement described in this report. Furthermore, neither the Inspector nor his employer shall be liable in any manner for any personal injury or property damage or a loss of any kind arising from or connected with this inspection.</p> <p>Authorized Inspector's Signature _____ Date _____ Commissions _____</p> <p style="text-align: center; font-size: small;">National Board, State, Province and Endorsements</p> <p style="text-align: center;">* Factory Mutual Ins. Co</p>	

First Energy Nuclear Operating, 76 South Main St., Akron, Ohio 44308

Davis-Besse Nuclear Power Station Unit #1, 5501 N. St. Rt. 2, Oak Harbor, Ohio 43449

Commercial Service Date: 11/27/77

FORM NIS-2 / NR-1 / NVR-1 OWNER'S REPORT FOR REPAIRS OR REPLACEMENTS As Required by the Provisions of the ASME Code Section XI

1. Owner Toledo Edison Company Date 4/19/00
Name
300 Madison Ave., Toledo, OH 43652
Address
 Sheet 1 of 1

2. Plant Davis-Besse Nuclear Power Station Unit #1
Name
5501 N. SR 2, Oak Harbor, OH 43449
Address
W.O. 99-005148-026
Repair Organization P.O. No., Job No., etc.

3. Work Performed By Toledo Edison Company Type Code Stamp NR
Name
5501 N. SR 2, Oak Harbor, OH 43449
Address
 Authorization NO. 20
 Expiration Date 7-17-01

4. Identification of Systems Sample Line from Steam Generator #2 EBB-5-17-H1 C-465 (Subsys 063-01)

5. (a) Applicable Construction Code ANSI B31.1 Edition 19 67 Addendum N/A Code Case N/A
 (b) Applicable Edition of Section XI Utilized for Repairs or Replacements 19 86
 (c) Design Responsibilities Lisega

6. Identification of Components Repaired or Replaced and Replacement Components

Name of Component	Name of Manufacturer	Manufacturer's Serial NO.	National Board NO.	Other Identification/Jurisdictional NO.	Year Built	Repaired or Replacement	ASME Code Stamped
Snubber	Lisega	61195/32	N/A	C-465	N/A	Replaced	No
Snubber	Lisega	61195/39	N/A	C-465	N/A	Replacement	No

7. Description of Work

Lisega hydraulic snubber S/N 61195/32 was removed and replaced with a like for like Lisega snubber S/N 61195/39.

Snubber was functionally tested satisfactory prior to installation.

8. Tests Conducted: Hydrostatic (NO) Pneumatic (NO) Nominal Operating Pressure (NO) Other (NO)
 Pressure N/A psi Test Temperature N/A °F

Pressure Relief Valves:
 Service N/A Size N/A
 Opening Pressure N/A Blowdown (if applicable) N/A
 Set Pressure and Blowdown Adjustments Made Using N/A (Test Medium)
 at N/A

NOTE: Supplemental sheets in the form of lists, sketches, or drawings may be used, provided (1) size is 8 1/2 x 11 in. (2) Information in items 1 through 6 on this report is included on each sheet, and (3) each sheet is numbered and the number of sheets is recorded at the top of this form.

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First Energy Nuclear Operating, 76 South Main St., Akron, Ohio 44308

Davis-Besse Nuclear Power Station Unit #1, 5501 N. St. Rt. 2, Oak Harbor, Ohio 43449

Commercial Service Date: 11/27/77

FORM NIS-2 / NR-1 / NVR-1	
8. REMARKS	Applicable Manufacturers Data Reports to be Attached
NA	
CERTIFICATE OF COMPLIANCE	
<p>We certify that the statements made in the report are correct and this replacement conforms to the rules of the ASME Code Section XI.</p> <p>Type Code Symbol <u>NR</u> Certificate of Authorization No. <u>20</u> Expiration Date <u>7-17-01</u></p> <p>Signed <u>[Signature]</u> Nuclear Safety & Inspection Manager Date <u>4/20</u> 19 <u>2000</u></p> <p style="text-align: center; font-size: small;">Owner or Owner's Designee, Title</p>	
CERTIFICATE OF INSERVICE INSPECTION	
<p>I, the undersigned, holding a valid commission issued by the National Board of Boiler and Pressure Vessel Inspectors in the State or Province of <u>Ohio</u> and employed by <u>Factory Mutual Ins. Co.</u> of <u>Johnston, R.I.</u> have inspected the components described in this Owner's Report during the period <u>9-23-79</u> to <u>4-20-00</u>, and state that to the best of my knowledge and belief, the Owner has performed examinations and taken corrective measures described in this Owner's Report in accordance with the requirements of the ASME Code, Section XI.</p> <p>By signing this certificate, neither the Inspector nor his employer makes any warranty, expressed or implied, concerning the examinations and corrective measures described in the Owner's Report. Furthermore, neither the Inspector nor his employer shall be liable in any manner for any personal injury or property damage or a loss of any kind arising from or connected with this inspection.</p> <p>Inspector's Signature <u>[Signature]</u> Date <u>4-20-00</u> Commissions <u>NB9801 "N.E.A." "OHIO" "F.M.I."</u> National Board, State, Province and Endorsements</p>	
CERTIFICATE OF COMPLIANCE	
<p>We certify that the statements made in this report are correct and the repair, modification, or replacement of the pressure relief device described above conforms to Section XI and Section III of the ASME Code and the National Board rules as defined in publications NB-65 and NB-102, current editions.</p> <p>Certificate of Authorization No. <u>316</u> to use the "VR" stamp expires <u>Dec. 13</u>, <u>2001</u></p> <p>Certificate of Authorization No. <u>20</u> to use the "NR" stamp expires <u>July 17</u>, <u>2001</u></p> <p>Date _____, 19____, Signed <u>Toledo Edison</u> Nuclear Safety & Inspection Manager Repair Organization Authorized Representative Title</p>	
CERTIFICATE OF INSPECTION	
<p>I, the undersigned, holding a valid commission issued by the National Board of Boiler and Pressure Vessel Inspectors, and certificate of competency issued by the State or Province of <u>Ohio</u> and employed by <u>Factory Mutual Ins. Co.</u> of <u>Johnston, R.I.</u> have inspected the repair, modification, or replacement described in this report on _____, 19____ and state that to the best of my knowledge and belief, this repair, modification, or replacement has been made or constructed in accordance with Section XI and Section III of the ASME Code and the National Board rules as defined in the publication NB-65 and NB-102, current editions.</p> <p>By signing this certificate, neither the Inspector nor his employer makes any warranty, expressed or implied, concerning the repair, modification or replacement described in this report. Furthermore, neither the Inspector nor his employer shall be liable in any manner for any personal injury or property damage or a loss of any kind arising from or connected with this inspection.</p> <p>Authorized Inspector's Signature _____ Date _____ Commissions _____ National Board, State, Province and Endorsements</p> <p style="text-align: center;">* Factory Mutual Ins. Co</p>	

First Energy Nuclear Operating, 76 South Main St., Akron, Ohio 44308

Davis-Besse Nuclear Power Station Unit #1, 5501 N. St. Rt. 2, Oak Harbor, Ohio 43449

Commercial Service Date: 11/27/77

FORM NIS-2 / NR-1 / NVR-1 OWNER'S REPORT FOR REPAIRS OR REPLACEMENTS As Required by the Provisions of the ASME Code Section XI

1. Owner Toledo Edison Company Date 5/2/00
Name
300 Madison Ave., Toledo, OH 43652
Address Sheet 1 of 1
2. Plant Davis-Besse Nuclear Power Station Unit #1
Name
5501 N. SR 2, Oak Harbor, OH 43449 WO 89-005148-034
Address Repair Organization P.O. No., Job No., etc.
3. Work Performed By Toledo Edison Company Type Code Stamp NR
Name
5501 N. SR 2, Oak Harbor, OH 43449 Authorization NO. 20
Address Expiration Date 7-17-01
4. Identification of Systems Main Steam from Stm Gen 1-2 3A-EBB-1-H1 (X) A-42 (Subsys 083-01)
5. (a) Applicable Construction Code ANSI B31.1 Edition 19 67 Addendum N/A Code Case N/A
 (b) Applicable Edition of Section XI Utilized for Repairs or Replacements 19 86
 (c) Design Responsibilities ITT Grinnell

6. Identification of Components Repaired or Replaced and Replacement Components

Name of Component	Name of Manufacturer	Manufacturer's Serial NO.	National Board NO.	Other Identification/Jurisdictional NO.	Year Built	Repaired Replaced or Replacement	ASME Code Stamped
Snubber	ITT Grinnell	18288	N/A	A-42	N/A	Replaced	No
Snubber	ITT Grinnell	14478	N/A	A-42	N/A	Replacement	No

7. Description of Work

Existing ITT Grinnell hydraulic snubber S/N18288 was removed and replaced with a like for like snubber S/N 14478.

Replacement snubber was functionally tested satisfactory prior to installation.

8. Tests Conducted: Hydrostatic (NO) Pneumatic (NO) Nominal Operating Pressure (NO) Other (NO)
 Pressure N/A psi Test Temperature N/A °F
- Pressure Relief Valves:
 Service N/A Size N/A
 Opening Pressure N/A Blowdown (if applicable) N/A
 Set Pressure and Blowdown Adjustments Made Using N/A (Test Medium)
 at N/A (Location)

NOTE: Supplemental sheets in the form of lists, sketches, or drawings may be used, provided (1) size is 8 1/2 x 11 in. (2) Information in items 1 through 6 on this report is included on each sheet, and (3) each sheet is numbered and the number of sheets is recorded at the top of this form.

FORM NIS-2 / NR-1 / NVR-1

9. REMARKS

Applicable Manufacturers Data Reports to be Attached

NA

CERTIFICATE OF COMPLIANCE

We certify that the statements made in the report are correct and this replacement conforms to the rules of the ASME Code Section XI.

Type Code Symbol NR Certificate of Authorization No. 20 Expiration Date 7-17-01

Signed J.W. Starnes, Jr. Nuclear Safety & Inspection Manager Date 5/3/2000 1999
 Owner or Owner's Designee, Title

CERTIFICATE OF INSERVICE INSPECTION

I, the undersigned, holding a valid commission issued by the National Board of Boiler and Pressure Vessel Inspectors in the State or Province of Ohio and employed by Factory Mutual Ins. Co. of Johnston, R.I. have inspected the components described in this Owner's Report during the period 4/13/99 to 5/3/00, and state that to the best of my knowledge and belief, the Owner has performed examinations and taken corrective measures described in this Owner's Report in accordance with the requirements of the ASME Code, Section XI.

By signing this certificate, neither the Inspector nor his employer makes any warranty, expressed or implied, concerning the examinations and corrective measures described in the Owner's Report. Furthermore, neither the Inspector nor his employer shall be liable in any manner for any personal injury or property damage or a loss of any kind arising from or connected with this inspection.

Inspector's Signature Thomas A. Lape Date 5/3/00 Commissions NB 9330 "NIA" OHIO COMM.
 National Board, State, Province and Endorsements

CERTIFICATE OF COMPLIANCE

We certify that the statements made in this report are correct and the repair, modification, or replacement of the pressure relief device described above conforms to Section XI and Section III of the ASME Code and the National Board rules as defined in publications NB-65 and NB-102, current editions.

Certificate of Authorization No. 316 to use the "VR" stamp expires Dec. 13 2001

Certificate of Authorization No. 20 to use the "NR" stamp expires July 17 2001

Date , 19 , Signed Toledo Edison Nuclear Safety & Inspection Manager
 Repair Organization Authorized Representative Title

CERTIFICATE OF INSPECTION

I, the undersigned, holding a valid commission issued by the National Board of Boiler and Pressure Vessel Inspectors, and certificate of competency issued by the State or Province of Ohio and employed by Factory Mutual Ins. Co.

of Johnston, R.I. have inspected the repair, modification, or replacement described in this report

on , 19 and state that to the best of my knowledge and belief, this repair, modification, or replacement has been made or constructed in accordance with Section XI and Section III of the ASME Code and the National Board rules as defined in the publication NB-65 and NB-102, current editions.

By signing this certificate, neither the Inspector nor his employer makes any warranty, expressed or implied, concerning the repair, modification or replacement described in this report. Furthermore, neither the Inspector nor his employer shall be liable in any manner for any personal injury or property damage or a loss of any kind arising from or connected with this inspection.

Authorized Inspector's Signature Date Commissions
 National Board, State, Province and Endorsements

* Factory Mutual Ins. Co

First Energy Nuclear Operating, 76 South Main St., Akron, Ohio 44308

Davis-Besse Nuclear Power Station Unit #1, 5501 N. St. Rt. 2, Oak Harbor, Ohio 43449

Commercial Service Date: 11/27/77

FORM NIS-2 / NR-1 / NVR-1 OWNER'S REPORT FOR REPAIRS OR REPLACEMENTS As Required by the Provisions of the ASME Code Section XI

1. Owner Toledo Edison Company Date 5/2/00
Name
300 Madison Ave., Toledo, OH 43652
Address
 Sheet 1 of 1

2. Plant Davis-Besse Nuclear Power Station Unit # 1
Name
5501 N. SR 2, Oak Harbor, OH 43449
Address
WO 99-005148-036
Repair Organization P.O. No., Job No., etc.

3. Work Performed By Toledo Edison Company Type Code Stamp NR
Name
5501 N. SR 2, Oak Harbor, OH 43449
Address
 Authorization NO. 20
 Expiration Date 7-17-01

4. Identification of Systems Main Steam from Stm Gen 1-2 3A-EBB-1-H1 (Z) A-43 (Subsys 083-01)

5. (a) Applicable Construction Code ANSI B31.1 Edition 19 6Z Addendum N/A Code Case N/A
 (b) Applicable Edition of Section XI Utilized for Repairs or Replacements 19 85
 (c) Design Responsibilities ITT Grinnell

6. Identification of Components Repaired or Replaced and Replacement Components

Name of Component	Name of Manufacturer	Manufacturer's Serial NO.	National Board NO.	Other Identification/Jurisdictional NO.	Year Built	Repaired Replaced or Replacement	ASME Code Stamped
Snubber	ITT Grinnell	18292	N/A	A-43	N/A	Replaced	No
Snubber	ITT Grinnell	74402397	N/A	A-43	N/A	Replacement	No

7. Description of Work

Existing ITT Grinnell hydraulic snubber S/N 18292 was removed and replaced with a like for like snubber S/N 74402397.

Replacement snubber was functionally tested satisfactory prior to installation.

8. Tests Conducted: Hydrostatic (NO) Pneumatic (NO) Nominal Operating Pressure (NO) Other (NO)
 Pressure N/A psi Test Temperature N/A °F

Pressure Relief Valves:
 Service N/A Size N/A
 Opening Pressure N/A Blowdown (If applicable) N/A
 Set Pressure and Blowdown Adjustments Made Using N/A
 at N/A (Test Medium)

NOTE: Supplemental sheets in the form of lists, sketches, or drawings may be used, provided (1) size is 8 1/2 x 11 in. (2) Information in items 1 through 6 on this report is included on each sheet, and (3) each sheet is numbered and the number of sheets is recorded at the top of this form.

FORM NIS-2 /NR-1 /NVR-1

9. REMARKS

Applicable Manufacturers Data Reports to be Attached

NA

CERTIFICATE OF COMPLIANCE

We certify that the statements made in the report are correct and this replacement conforms to the rules of the ASME Code Section XI.

Type Code Symbol NR Certificate of Authorization No. 20 Expiration Date 7-17-01

Signed [Signature] Nuclear Safety & Inspection Manager Date 5/3/2000
 Owner or Owner's Designee, Title

CERTIFICATE OF INSERVICE INSPECTION

I, the undersigned, holding a valid commission issued by the National Board of Boiler and Pressure Vessel Inspectors in the State or Province of Ohio and employed by Factory Mutual Ins. Co. of Johnston, R.I. have inspected the components described in this Owner's Report during the period 4/23/99 to 5/3/00, and state that to the best of my knowledge and belief, the Owner has performed examinations and taken corrective measures described in this Owner's Report in accordance with the requirements of the ASME Code, Section XI.

By signing this certificate, neither the Inspector nor his employer makes any warranty, expressed or implied, concerning the examinations and corrective measures described in the Owner's Report. Furthermore, neither the Inspector nor his employer shall be liable in any manner for any personal injury or property damage or a loss of any kind arising from or connected with this inspection.

Inspector's Signature Thomas A. Lipe Date 5/3/00 Commissions NB9336 "N I A" OHIO COMM.
 National Board, State, Province and Endorsements

CERTIFICATE OF COMPLIANCE

We certify that the statements made in this report are correct and the repair, modification, or replacement of the pressure relief device described above conforms to Section XI and Section III of the ASME Code and the National Board rules as defined in publications NB-65 and NB-102, current editions.

Certificate of Authorization No. 316 to use the "VR" stamp expires Dec. 13, 2001

Certificate of Authorization No. 20 to use the "NR" stamp expires July 17, 2001

Date , 19 , Signed Toledo Edison Nuclear Safety & Inspection Manager
 Repair Organization Authorized Representative Title

CERTIFICATE OF INSPECTION

I, the undersigned, holding a valid commission issued by the National Board of Boiler and Pressure Vessel Inspectors, and certificate of competency issued by the State or Province of Ohio and employed by Factory Mutual Ins. Co. of Johnston, R.I. have inspected the repair, modification, or replacement described in this report

on , 19 and state that to the best of my knowledge and belief, this repair, modification, or replacement has been made or constructed in accordance with Section XI and Section III of the ASME Code and the National Board rules as defined in the publication NB-65 and NB-102, current editions.

By signing this certificate, neither the Inspector nor his employer makes any warranty, expressed or implied, concerning the repair, modification or replacement described in this report. Furthermore, neither the Inspector nor his employer shall be liable in any manner for any personal injury or property damage or a loss of any kind arising from or connected with this inspection.

Authorized Inspector's Signature Date Commissions
 National Board, State, Province and Endorsements
 * Factory Mutual Ins. Co

First Energy Nuclear Operating, 76 South Main St., Akron, Ohio 44308

Davis-Besse Nuclear Power Station Unit #1, 5501 N. St. Rt. 2, Oak Harbor, Ohio 43449

Commercial Service Date: 11/27/77

FORM NIS-2 / NR-1 / NVR-1 OWNER'S REPORT FOR REPAIRS OR REPLACEMENTS As Required by the Provisions of the ASME Code Section XI

1. Owner Toledo Edison Company Date 5/2/00
Name
300 Madison Ave., Toledo, OH 43652
Address
 2. Plant Davis-Besse Nuclear Power Station Unit # 1
Name
5501 N. SR 2, Oak Harbor, OH 43449
Address
 3. Work Performed By Toledo Edison Company Type Code Stamp NR
Name
5501 N. SR 2, Oak Harbor, OH 43449
Address
 Authorization NO. 20
 Expiration Date 7-17-01
 4. Identification of Systems Containment Spray System 34-GCB-5-H2 A-82 (Subsys 061-01)
 5. (a) Applicable Construction Code ANSI B31.1 Edition 19 87 Addendum N/A Code Case N/A
 (b) Applicable Edition of Section XI Utilized for Repairs or Replacements 19 86
 (c) Design Responsibilities ITT Grinnell

6. Identification of Components Repaired or Replaced and Replacement Components

Name of Component	Name of Manufacturer	Manufacturer's Serial NO.	National Board NO.	Other Identification/Jurisdictional NO.	Year Built	Repaired Replaced or Replacement	ASME Code Stamped
Snubber	ITT Grinnell	16150	N/A	A-82	N/A	Replaced	No
Snubber	ITT Grinnell	21303	N/A	A-82	N/A	Replacement	No

7. Description of Work

Existing ITT Grinnell hydraulic snubber S/N 16150 was removed and replaced with a like for like snubber S/N 21303.

Replacement snubber was functionally tested satisfactory prior to installation.

8. Tests Conducted: Hydrostatic (NO) Pneumatic (NO) Nominal Operating Pressure (NO) Other (NO)
 Pressure N/A psi Test Temperature N/A °F
 Pressure Relief Valves:
 Service N/A Size N/A
 Opening Pressure N/A Blowdown (if applicable) N/A
 Set Pressure and Blowdown Adjustments Made Using N/A
 at N/A

NOTE: Supplemental sheets in the form of lists, sketches, or drawings may be used, provided (1) size is 8 1/2 x 11 in. (2) Information in items 1 through 6 on this report is included on each sheet, and (3) each sheet is numbered and the number of sheets is recorded at the top of this form.

12 RFO Inservice Inspection Summary

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First Energy Nuclear Operating, 76 South Main St., Akron, Ohio 44308

Davis-Besse Nuclear Power Station Unit #1, 5501 N. St. Rt. 2, Oak Harbor, Ohio 43449

Commercial Service Date: 11/27/77

FORM NIS-2 /NR-1 /AVR-1

9. REMARKS

Applicable Manufacturers Data Reports to be Attached

NA

CERTIFICATE OF COMPLIANCE

We certify that the statements made in the report are correct and this replacement conforms to the rules of the ASME Code Section XI.

Type Code Symbol NR Certificate of Authorization No. 20 Expiration Date 7-17-01

Signed John S. Lanning, Jr. Nuclear Safety & Inspection Manager Date 5/3/2000 '99

Owner or Owner's Designee, Title

CERTIFICATE OF INSERVICE INSPECTION

I, the undersigned, holding a valid commission issued by the National Board of Boiler and Pressure Vessel Inspectors in the State or Province of Ohio and employed by Factory Mutual Ins. Co. of Johnston, R.I. have inspected the components described in this Owner's Report during the period 4/23/98 to 5/3/00, and state that to the best of my knowledge and belief, the Owner has performed examinations and taken corrective measures described in this Owner's Report in accordance with the requirements of the ASME Code, Section XI.

By signing this certificate, neither the inspector nor his employer makes any warranty, expressed or implied, concerning the examinations and corrective measures described in the Owner's Report. Furthermore, neither the inspector nor his employer shall be liable in any manner for any personal injury or property damage or a loss of any kind arising from or connected with this inspection.

Inspector's Signature Thomas A. LajoieDate 5/3/00Commissions NB 9330 N E A OHIO
National Board, State, Province and Endorsements CEMPL.

CERTIFICATE OF COMPLIANCE

We certify that the statements made in this report are correct and the repair, modification, or replacement of the pressure relief device described above conforms to Section XI and Section III of the ASME Code and the National Board rules as defined in publications NB-65 and NB-102, current editions.

Certificate of Authorization No. 318 to use the "VR" stamp expires Dec. 13, 2001

Certificate of Authorization No. 20 to use the "NR" stamp expires July 17, 2001

Date _____, 19____, Signed Toledo Edison Nuclear Safety & Inspection Manager
Repair Organization Authorized Representative Title

CERTIFICATE OF INSPECTION

I, the undersigned, holding a valid commission issued by the National Board of Boiler and Pressure Vessel Inspectors, and certificate of competency issued by the State or Province of Ohio and employed by Factory Mutual Ins. Co.

of Johnston, R.I. have inspected the repair, modification, or replacement described in this report

on _____, 19____ and state that to the best of my knowledge and belief, this repair, modification, or replacement has been made or constructed in accordance with Section XI and Section III of the ASME Code and the National Board rules as defined in the publication NB-65 and NB-102, current editions.

By signing this certificate, neither the inspector nor his employer makes any warranty, expressed or implied, concerning the repair, modification or replacement described in this report. Furthermore, neither the inspector nor his employer shall be liable in any manner for any personal injury or property damage or a loss of any kind arising from or connected with this inspection.

Authorized Inspector's Signature _____

Date _____

Commissions _____
National Board, State, Province and Endorsements

* Factory Mutual Ins. Co

FORM NIS-2 / NR-1 / NVR-1 OWNER'S REPORT FOR REPAIRS OR REPLACEMENTS
 As Required by the Provisions of the ASME Code Section XI

1. Owner Toledo Edison Company Date 5/9/00
Name
300 Madison Ave., Toledo, OH 43652
Address
 Sheet 1 of 1

2. Plant Davis-Besse Nuclear Power Station Unit # 1
Name
5501 N. St. Rt. 2, Oak Harbor, OH 43449
Address
WO 89-005148-046
Repair Organization P.O. No., Job No., etc.

3. Work Performed By Toledo Edison Company Type Code Stamp NR
Name
5501 N. St. Rt. 2, Oak Harbor, OH 43449
Address
 Authorization NO. 20
 Expiration Date 7-17-01

4. Identification of Systems Pressurizer 30-CCA-8-H4 C-125 (Subsys 064-04)

5. (a) Applicable Construction Code ANSI B31.1 Edition 19 67 Addendum N/A Code Case N/A
 (b) Applicable Edition of Section XI Utilized for Repairs or Replacements 19 86
 (c) Design Responsibilities ITT Grinnell

6. Identification of Components Repaired or Replaced and Replacement Components

Name of Component	Name of Manufacturer	Manufacturer's Serial NO.	National Board NO.	Other Identification/Jurisdictional NO.	Year Built	Repaired Replaced or Replacement	ASME Code Stamped
Snubber	ITT Grinnell	11188	N/A	C-125	N/A	Replaced	No
Snubber	ITT Grinnell	30224	N/A	C-125	N/A	Replacement	No

7. Description of Work

Existing ITT Grinnell hydraulic snubber S/N 11188 was removed and replaced with a like for like snubber S/N 30224.
 Replacement snubber S/N 30224 was rebuilt by WO 89-005150-000. Replacement snubber was functionally tested satisfactory prior to installation.

8. Tests Conducted: Hydrostatic (NO) Pneumatic (NO) Nominal Operating Pressure (NO) Other (NO)
 Pressure N/A psi Test Temperature N/A °F

Pressure Relief Valves:
 Service N/A Size N/A
 Opening Pressure N/A Blowdown (if applicable) N/A
 Set Pressure and Blowdown Adjustments Made Using N/A
 at N/A (Test Station)

NOTE: Supplemental sheets in the form of lists, sketches, or drawings may be used, provided (1) size is 8 1/2 x 11 in. (2) information in items 1 through 6 on this report is included on each sheet, and (3) each sheet is numbered and the number of sheets is recorded at the top of this form.

12 RFO Inservice Inspection Summary

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First Energy Nuclear Operating, 76 South Main St., Akron, Ohio 44308

Davis-Besse Nuclear Power Station Unit #1, 5501 N. St. Rt. 2, Oak Harbor, Ohio 43449

Commercial Service Date: 11/27/77

FORM NIS-2 / NR-1 / NVR-1-

9. REMARKS

Applicable Manufacturers Data Reports to be Attached

N/A

CERTIFICATE OF COMPLIANCE

We certify that the statements made in the report are correct and this replacement conforms to the rules of the ASME Code Section XI.

Type Code Symbol NR Certificate of Authorization No. 20 Expiration Date 7-17-01

Signed John S. Lantz, Jr. Nuclear Safety & Inspection Manager Date 5/10/2000
Owner or Owner's Designee, Title

CERTIFICATE OF INSERVICE INSPECTION

I, the undersigned, holding a valid commission issued by the National Board of Boiler and Pressure Vessel Inspectors in the State or Province of Ohio and employed by Factory Mutual Ins. Co.* of Johnston, R.I. have inspected the components described in this Owner's Report during the period 5/8/00 to 5/11/00, and state that to the best of my knowledge and belief, the Owner has performed examinations and taken corrective measures described in this Owner's Report in accordance with the requirements of the ASME Code, Section XI.

By signing this certificate, neither the Inspector nor his employer makes any warranty, expressed or implied, concerning the examinations and corrective measures described in the Owner's Report. Furthermore, neither the Inspector nor his employer shall be liable in any manner for any personal injury or property damage or a loss of any kind arising from or connected with this inspection.

Inspector's Signature Thomas H. Lantz Date 5/11/00 Commissions NB9330 N I A "OHIO" COMM.
National Board, State, Province and Endorsements

CERTIFICATE OF COMPLIANCE

We certify that the statements made in this report are correct and the repair, modification, or replacement of the pressure relief device described above conforms to Section XI and Section III of the ASME Code and the National Board rules as defined in publications NB-65 and NB-102, current editions.

Certificate of Authorization No. 316 to use the "VR" stamp expires Dec. 13, 2001.

Certificate of Authorization No. 20 to use the "NR" stamp expires July 17, 2001.

Date _____, 19____, Signed Toledo Edison Nuclear Safety & Inspection Manager
Repair Organization Authorized Representative Title

CERTIFICATE OF INSPECTION

I, the undersigned, holding a valid commission issued by the National Board of Boiler and Pressure Vessel Inspectors, and

certificate of competency issued by the State or Province of Ohio and employed by Factory Mutual Ins. Co.*

of Johnston, R.I. have inspected the repair, modification, or replacement described in this report

on _____, 19____ and state that to the best of my knowledge and belief, this repair, modification, or replacement has been made or constructed in accordance with Section XI and Section III of the ASME Code and the National Board rules as defined in the publication NB-65 and NB-102, current editions.

By signing this certificate, neither the Inspector nor his employer makes any warranty, expressed or implied, concerning the repair, modification or replacement described in this report. Furthermore, neither the Inspector nor his employer shall be liable in any manner for any personal injury or property damage or a loss of any kind arising from or connected with this inspection.

Authorized Inspector's Signature _____ Date _____ Commissions _____
National Board, State, Province and Endorsements

* Factory Mutual Ins. Co

12 RFO Inservice Inspection Summary

First Energy Nuclear Operating, 76 South Main St., Akron, Ohio 44308

Davis-Besse Nuclear Power Station Unit #1, 5501 N. St. Rt. 2, Oak Harbor, Ohio 43449

Commercial Service Date: 11/27/77

FORM NIS-2 / NR-1 / NVR-1 OWNER'S REPORT FOR REPAIRS OR REPLACEMENTS
 As Required by the Provisions of the ASME Code Section XI

1. Owner Toledo Edison Company Date 4/30/00
Name
300 Madison Ave., Toledo, OH 43652
Address
 Sheet 1 of 1

2. Plant Davis-Besse Nuclear Power Station Unit # 1
Name
5501 N. SR 2, Oak Harbor, OH 43449
Address
WO 99-005149-004
Repair Organization P.O. No., Job No., etc.

3. Work Performed By Toledo Edison Company Type Code Stamp NR
Name
5501 N. SR 2, Oak Harbor, OH 43449
Address
 Authorization NO. 20
 Expiration Date 7-17-01

4. Identification of Systems Feedwater Piping EBB-3-SR24 (A) C-179 (Subsys 045-01)

5. (a) Applicable Construction Code ANSI B31.1 Edition 19 67 Addendum N/A Code Case N/A
 (b) Applicable Edition of Section XI Utilized for Repairs or Replacements 19 86
 (c) Design Responsibilities ITT Grinnell

6. Identification of Components Repaired or Replaced and Replacement Components

Name of Component	Name of Manufacturer	Manufacturer's Serial NO.	National Board NO.	Other Identification/Jurisdictional NO.	Year Built	Repaired Replaced or Replacement	ASME Code Stamped
Snubber	ITT Grinnell	12484	N/A	C-179	N/A	Replaced	No
Snubber	ITT Grinnell	14200	N/A	C-179	N/A	Replacement	No

7. Description of Work

ITT Grinnell hydraulic snubber S/N 12484 was removed and replaced with a like for like snubber S/N 14200.
 Snubber was functionally tested satisfactory prior to installation.

8. Tests Conducted: Hydrostatic (NO) Pneumatic (NO) Nominal Operating Pressure (NO) Other (NO)
 Pressure N/A psi Test Temperature N/A °F

Pressure Relief Valves:
 Service N/A Size N/A
 Opening Pressure N/A Blowdown (If applicable) N/A
 Set Pressure and Blowdown Adjustments Made Using N/A
 at N/A (Test Station)

NOTE: Supplemental sheets in the form of lists, sketches, or drawings may be used, provided (1) size is 8 1/2 x 11 in. (2) Information in items 1 through 6 on this report is included on each sheet, and (3) each sheet is numbered and the number of sheets is recorded at the top of this form.

12 RFO Inservice Inspection Summary

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First Energy Nuclear Operating, 76 South Main St., Akron, Ohio 44308

Davis-Besse Nuclear Power Station Unit #1, 5501 N. St. Rt. 2, Oak Harbor, Ohio 43449

Commercial Service Date: 11/27/77

FORM NIS-2 /NR-1 /NVR-4

B. REMARKS

Applicable Manufacturers Data Reports to be Attached

NA

CERTIFICATE OF COMPLIANCE

We certify that the statements made in the report are correct and this replacement conforms to the rules of the ASME Code Section XI.

Type Code Symbol NR Certificate of Authorization No. 20 Expiration Date 7-17-01

Signed H. W. Starnes, Jr. Nuclear Safety & Inspection Manager Date 5/2/2000
Owner or Owner's Designee, Title

CERTIFICATE OF INSERVICE INSPECTION

I, the undersigned, holding a valid commission issued by the National Board of Boiler and Pressure Vessel Inspectors in the State or Province of Ohio and employed by Factory Mutual Ins. Co. of Johnston, R.I. have inspected the components described in this Owner's Report during the period 9-23-99 to 5-2-00, and state that to the best of my knowledge and belief, the Owner has performed examinations and taken corrective measures described in this Owner's Report in accordance with the requirements of the ASME Code, Section XI.

By signing this certificate, neither the Inspector nor his employer makes any warranty, expressed or implied, concerning the examinations and corrective measures described in the Owner's Report. Furthermore, neither the Inspector nor his employer shall be liable in any manner for any personal injury or property damage or a loss of any kind arising from or connected with this inspection.

Inspector's Signature [Signature] Date 5-2-00 Commissions NB-9801 N.D.A. OHIO
National Board, State, Province and Endorsements

CERTIFICATE OF COMPLIANCE

We certify that the statements made in this report are correct and the repair, modification, or replacement of the pressure relief device described above conforms to Section XI and Section III of the ASME Code and the National Board rules as defined in publications NB-65 and NB-102, current editions.

Certificate of Authorization No. 316 to use the "VR" stamp expires Dec. 19, 2001.

Certificate of Authorization No. 20 to use the "NR" stamp expires July 17, 2001.

Date , 19 , Signed Toledo Edison
Repair Organization Authorized Representative Title

Nuclear Safety &
Inspection Manager

CERTIFICATE OF INSPECTION

I, the undersigned, holding a valid commission issued by the National Board of Boiler and Pressure Vessel Inspectors, and certificate of competency issued by the State or Province of Ohio and employed by Factory Mutual Ins. Co. of Johnston, R.I. have inspected the repair, modification, or replacement described in this report

on , 19 and state that to the best of my knowledge and belief, this repair, modification, or replacement has been made or constructed in accordance with Section XI and Section III of the ASME Code and the National Board rules as defined in the publication NB-65 and NB-102, current editions.

By signing this certificate, neither the Inspector nor his employer makes any warranty, expressed or implied, concerning the repair, modification or replacement described in this report. Furthermore, neither the Inspector nor his employer shall be liable in any manner for any personal injury or property damage or a loss of any kind arising from or connected with this inspection.

Authorized Inspector's Signature Date Commissions
National Board, State, Province and Endorsements

* Factory Mutual Ins. Co

First Energy Nuclear Operating, 76 South Main St., Akron, Ohio 44308

Davis-Besse Nuclear Power Station Unit #1, 5501 N. St. Rt. 2, Oak Harbor, Ohio 43449

Commercial Service Date: 11/27/77

FORM NIS-2 / NR-1 / NVR-1 OWNER'S REPORT FOR REPAIRS OR REPLACEMENTS As Required by the Provisions of the ASME Code Section XI

1. Owner Toledo Edison Company Date 5/8/00
Name
300 Madison Ave., Toledo, OH 43652
Address Sheet 1 of 1
2. Plant Davis-Besse Nuclear Power Station Unit #1
Name
5501 N. SR 2, Oak Harbor, OH 43449
Address WO 99-005149-006
Repair Organization P.O. No., Job No., etc.
3. Work Performed By Toledo Edison Company Type Code Stamp NR
Name
5501 N. SR 2, Oak Harbor, OH 43449
Address Authorization NO. 20
 Expiration Date 7-17-01
4. Identification of Systems Feedwater Piping EBB-3-SR25 (A) C-181 (Subsys 045-01)
5. (a) Applicable Construction Code ANSI B31.1 Edition 19 67 Addendum N/A Code Case N/A
 (b) Applicable Edition of Section XI Utilized for Repairs or Replacements 19 86
 (c) Design Responsibilities ITT Grinnell

6. Identification of Components Repaired or Replaced and Replacement Components

Name of Component	Name of Manufacturer	Manufacturer's Serial NO.	National Board NO.	Other Identification/Jurisdictional NO.	Year Built	Repaired Replaced or Replacement	ASME Code Stamped
Snubber	ITT Grinnell	42500 <u>12850</u>	N/A	C-181	N/A	Replaced	No
Snubber	ITT Grinnell	12476	N/A	C-181	N/A	Replacement	No

7. Description of Work

12850 Existing ITT Grinnell hydraulic snubber S/N ~~42500~~ was removed and replaced with a like for like snubber S/N 12476.

Replacement snubber was functionally tested satisfactory prior to installation.

8. Tests Conducted: Hydrostatic (NO) Pneumatic (NO) Nominal Operating Pressure (NO) Other (NO)
 Pressure N/A psi Test Temperature N/A °F
- Pressure Relief Valves:
 Service N/A Size N/A
 Opening Pressure N/A Blowdown (if applicable) N/A
 Set Pressure and Blowdown Adjustments Made Using N/A
 at N/A

NOTE: Supplemental sheets in the form of lists, sketches, or drawings may be used, provided (1) size is 8 1/2 x 11 in. (2) information in items 1 through 6 on this report is included on each sheet, and (3) each sheet is numbered and the number of sheets is recorded at the top of this form.

FORM NIS-2 / NR-1 / NVR-1	
9. REMARKS	Applicable Manufacturers Data Reports to be Attached
NA	
CERTIFICATE OF COMPLIANCE	
<p>We certify that the statements made in the report are correct and this <u>replacement</u> conforms to the rules of the ASME Code Section XI.</p> <p>Type Code Symbol <u>NR</u> Certificate of Authorization No. <u>20</u> Expiration Date <u>7-17-01</u></p> <p>Signed <u>[Signature]</u> Nuclear Safety & Inspection Manager Date <u>5/9/2000</u></p> <p style="text-align: center; font-size: x-small;">Owner or Owner's Designee, Title</p>	
CERTIFICATE OF INSERVICE INSPECTION	
<p>I, the undersigned, holding a valid commission issued by the National Board of Boiler and Pressure Vessel Inspectors in the State or Province of <u>Ohio</u> and employed by <u>Factory Mutual Ins. Co.</u> of <u>Johnston, R.I.</u> have inspected the components described in this Owner's Report during the period <u>4/23/99</u> to <u>5/9/00</u>, and state that to the best of my knowledge and belief, the Owner has performed examinations and taken corrective measures described in this Owner's Report in accordance with the requirements of the ASME Code, Section XI.</p> <p>By signing this certificate, neither the inspector nor his employer makes any warranty, expressed or implied, concerning the examinations and corrective measures described in the Owner's Report. Furthermore, neither the inspector nor his employer shall be liable in any manner for any personal injury or property damage or a loss of any kind arising from or connected with this inspection.</p> <p>Inspector's Signature <u>[Signature]</u> Date <u>5/9/00</u> Commissions <u>NB9336 NIA Ohio Coun.</u></p> <p style="text-align: center; font-size: x-small;">National Board, State, Province and Endorsements</p>	
CERTIFICATE OF COMPLIANCE	
<p>We certify that the statements made in this report are correct and the repair, modification, or replacement of the pressure relief device described above conforms to Section XI and Section III of the ASME Code and the National Board rules as defined in publications NB-65 and NB-102, current editions.</p> <p>Certificate of Authorization No. <u>316</u> to use the "VR" stamp expires <u>Dec. 13</u>, <u>2001</u></p> <p>Certificate of Authorization No. <u>20</u> to use the "NR" stamp expires <u>July 17</u>, <u>2001</u></p> <p>Date _____, 19____, Signed <u>Toledo Edison</u> Nuclear Safety & Inspection Manager</p> <p style="text-align: center; font-size: x-small;">Repair Organization Authorized Representative Title</p>	
CERTIFICATE OF INSPECTION	
<p>I, the undersigned, holding a valid commission issued by the National Board of Boiler and Pressure Vessel Inspectors, and certificate of competency issued by the State or Province of <u>Ohio</u> and employed by <u>Factory Mutual Ins. Co.</u> of <u>Johnston, R.I.</u> have inspected the repair, modification, or replacement described in this report on _____, 19____ and state that to the best of my knowledge and belief, this repair, modification, or replacement has been made or constructed in accordance with Section XI and Section III of the ASME Code and the National Board rules as defined in the publication NB-65 and NB-102, current editions.</p> <p>By signing this certificate, neither the inspector nor his employer makes any warranty, expressed or implied, concerning the repair, modification or replacement described in this report. Furthermore, neither the inspector nor his employer shall be liable in any manner for any personal injury or property damage or a loss of any kind arising from or connected with this inspection.</p> <p>Authorized Inspector's Signature _____ Date _____ Commissions <u>Factory Mutual Ins. Co</u></p> <p style="text-align: center; font-size: x-small;">National Board, State, Province and Endorsements</p>	

First Energy Nuclear Operating, 76 South Main St., Akron, Ohio 44308

Davis-Besse Nuclear Power Station Unit #1, 5501 N. St. Rt. 2, Oak Harbor, Ohio 43449

Commercial Service Date: 11/27/77

FORM NIS-2 / NR-1 / NVR-1 OWNER'S REPORT FOR REPAIRS OR REPLACEMENTS As Required by the Provisions of the ASME Code Section XI

1. Owner Toledo Edison Company Date 4/29/00
Name
300 Madison Ave., Toledo, OH 43652
Address
 Sheet 1 of 1
2. Plant Davis-Besse Nuclear Power Station Unit #1
Name WO 99-005145-008
5501 N. SR 2, Oak Harbor, OH 43449
Address Repair Organization P.O. No., Job No., etc.
3. Work Performed By Toledo Edison Company Type Code Stamp NR
Name Authorization NO. 20
5501 N. SR 2, Oak Harbor, OH 43449
Address Expiration Date 7-17-01
4. Identification of Systems Feedwater Piping EBB-3-SR20 C-254 (Subsys 045-01)
5. (a) Applicable Construction Code ANSI B31.1 Edition 19 67 Addendum N/A Code Case N/A
 (b) Applicable Edition of Section XI Utilized for Repairs or Replacements 19 86
 (c) Design Responsibilities ITT Grinnell

6. Identification of Components Repaired or Replaced and Replacement Components

Name of Component	Name of Manufacturer	Manufacturer's Serial NO.	National Board NO.	Other Identification/Jurisdictional NO.	Year Built	Repaired Replaced or Replacement	ASME Code Stamped
Snubber	ITT Grinnell	12493	N/A	C-254	N/A	Replaced	No
Snubber	ITT Grinnell	12491	N/A	C-254	N/A	Replacement	No

7. Description of Work

ITT Grinnell hydraulic snubber S/N 12493 was removed and replaced with a like for like snubber S/N 12491.

Snubber was functionally tested satisfactory prior to installation.

8. Tests Conducted: Hydrostatic (NO) Pneumatic (NO) Nominal Operating Pressure (NO) Other (NO)
 Pressure N/A psi Test Temperature N/A °F
- Pressure Relief Valves:
 Service N/A Size N/A
 Opening Pressure N/A Blowdown (if applicable) N/A
 Set Pressure and Blowdown Adjustments Made Using N/A
 at N/A

NOTE: Supplemental sheets in the form of lists, sketches, or drawings may be used, provided (1) size is 8 1/2 x 11 in. (2) information in items 1 through 6 on this report is included on each sheet, and (3) each sheet is numbered and the number of sheets is recorded at the top of this form.

FORM NIS-2 /NR-1 /NVR-1

9. REMARKS		Applicable Manufacturers Data Reports to be Attached
NA		

CERTIFICATE OF COMPLIANCE

We certify that the statements made in the report are correct and this replacement conforms to the rules of the ASME Code Section XI.

Type Code Symbol NR Certificate of Authorization No. 20 Expiration Date 7-17-01

Signed [Signature] Nuclear Safety & Inspection Manager Date 4/30/2000
 Owner or Owner's Designee, Title

CERTIFICATE OF INSERVICE INSPECTION

I, the undersigned, holding a valid commission issued by the National Board of Boiler and Pressure Vessel Inspectors in the State or Province of Ohio and employed by Factory Mutual Ins. Co. of Johnston, R.I. have inspected the components described in this Owner's Report during the period 7-23-99 to 4-30-00, and state that to the best of my knowledge and belief, the Owner has performed examinations and taken corrective measures described in this Owner's Report in accordance with the requirements of the ASME Code, Section XI.

By signing this certificate, neither the Inspector nor his employer makes any warranty, expressed or implied, concerning the examinations and corrective measures described in the Owner's Report. Furthermore, neither the Inspector nor his employer shall be liable in any manner for any personal injury or property damage or a loss of any kind arising from or connected with this inspection.

Inspector's Signature [Signature] Date 4-30-00 Commissions NB9901 N I A Ohio
 National Board, State, Province and Endorsements

CERTIFICATE OF COMPLIANCE

We certify that the statements made in this report are correct and the repair, modification, or replacement of the pressure relief device described above conforms to Section XI and Section III of the ASME Code and the National Board rules as defined in publications NB-65 and NB-102, current editions.

Certificate of Authorization No. 316 to use the "VR" stamp expires Dec. 13, 2001.

Certificate of Authorization No. 20 to use the "NR" stamp expires July 17, 2001.

Date _____, 19____, Signed Toledo Edison Nuclear Safety & Inspection Manager
 Repair Organization Authorized Representative Title

CERTIFICATE OF INSPECTION

I, the undersigned, holding a valid commission issued by the National Board of Boiler and Pressure Vessel Inspectors, and certificate of competency issued by the State or Province of Ohio and employed by Factory Mutual Ins. Co. of Johnston, R.I. have inspected the repair, modification, or replacement described in this report

on _____, 19____ and state that to the best of my knowledge and belief, this repair, modification, or replacement has been made or constructed in accordance with Section XI and Section III of the ASME Code and the National Board rules as defined in the publication NB-65 and NB-102, current editions.

By signing this certificate, neither the Inspector nor his employer makes any warranty, expressed or implied, concerning the repair, modification or replacement described in this report. Furthermore, neither the Inspector nor his employer shall be liable in any manner for any personal injury or property damage or a loss of any kind arising from or connected with this inspection.

Authorized Inspector's Signature _____ Date _____ Commissions _____
 National Board, State, Province and Endorsements

* Factory Mutual Ins. Co

First Energy Nuclear Operating, 76 South Main St., Akron, Ohio 44308

Davis-Besse Nuclear Power Station Unit #1, 5501 N. St. Rt. 2, Oak Harbor, Ohio 43449

Commercial Service Date: 11/27/77

FORM NIS-2 / NR-1 / NVR-1 OWNER'S REPORT FOR REPAIRS OR REPLACEMENTS As Required by the Provisions of the ASME Code Section XI

1. Owner Toledo Edison Company Date 4/23/00
Name
300 Madison Ave., Toledo, OH 43652
Address
 Sheet 1 of 1

2. Plant Davis-Besse Nuclear Power Station Unit # 1
Name
5501 N. SR 2, Oak Harbor, OH 43449
Address
WO 99-005149-014
Repair Organization P.O. No., Job No., etc.

3. Work Performed By Toledo Edison Company Type Code Stamp NR
Name
5501 N. SR 2, Oak Harbor, OH 43449
Address
 Authorization NO. 20
 Expiration Date 7-17-01

4. Identification of Systems Stm Gen # 2 Secondary Drain Line EBB-5-9-H1 C-451 (Subsys 063-01)

5. (a) Applicable Construction Code ANSI B31.1 Edition 19 87 Addendum N/A Code Case N/A
 (b) Applicable Edition of Section XI Utilized for Repairs or Replacements 19 86
 (c) Design Responsibilities Lisega

6. Identification of Components Repaired or Replaced and Replacement Components

Name of Component	Name of Manufacturer	Manufacturer's Serial NO.	National Board NO.	Other Identification/Jurisdictional NO.	Year Built	Repaired Replaced or Replacement	ASME Code Stamped
Snubber	Lisega	61200/30	N/A	C-451	N/A	Replaced	No
Snubber	Lisega	61200/24	N/A	C-451	N/A	Replacement	No

7. Description of Work

Lisega Snubber S/N 61200/30 was removed and replaced with a like for like Lisega snubber S/N 61200/24.

The snubber was functionally tested satisfactory prior to installation.

8. Tests Conducted: Hydrostatic (NO) Pneumatic (NO) Nominal Operating Pressure (NO) Other (NO)
 Pressure N/A psi Test Temperature N/A °F

Pressure Relief Valves:
 Service N/A Size N/A
 Opening Pressure N/A Blowdown (if applicable) N/A
 Set Pressure and Blowdown Adjustments Made Using N/A
 at N/A (Test Station)

NOTE: Supplemental sheets in the form of lists, sketches, or drawings may be used, provided (1) size is 8 1/2 x 11 in. (2) Information in items 1 through 6 on this report is included on each sheet, and (3) each sheet is numbered and the number of sheets is recorded at the top of this form.

FORM NIS-2 /NR-1 /NVR-1

9. REMARKS	Applicable Manufacturers Data Reports to be Attached
NA	

CERTIFICATE OF COMPLIANCE

We certify that the statements made in the report are correct and this replacement conforms to the rules of the ASME Code Section XI.

Type Code Symbol NR Certificate of Authorization No. 20 Expiration Date 7-17-01

Signed [Signature] Nuclear Safety & Inspection Manager Date 4/24/00
Owner or Owner's Designee, Title

CERTIFICATE OF INSERVICE INSPECTION

I, the undersigned, holding a valid commission issued by the National Board of Boiler and Pressure Vessel Inspectors in the State or Province of Ohio and employed by Factory Mutual Ins. Co. of Johnston, R.I. have inspected the components described in this Owner's Report during the period 9/23/99 to 4/24/00, and state that to the best of my knowledge and belief, the Owner has performed examinations and taken corrective measures described in this Owner's Report in accordance with the requirements of the ASME Code, Section XI.

By signing this certificate, neither the Inspector nor his employer makes any warranty, expressed or implied, concerning the examinations and corrective measures described in the Owner's Report. Furthermore, neither the Inspector nor his employer shall be liable in any manner for any personal injury or property damage or a loss of any kind arising from or connected with this inspection.

[Signature]
Inspector's Signature

4/24/00
Date

Commissions NB9330 N I A OHIO COMM.
National Board, State, Province and Endorsements

CERTIFICATE OF COMPLIANCE

We certify that the statements made in this report are correct and the repair, modification, or replacement of the pressure relief device described above conforms to Section XI and Section III of the ASME Code and the National Board rules as defined in publications NB-65 and NB-102, current editions.

Certificate of Authorization No. 316 to use the "VR" stamp expires Dec. 13, 2001

Certificate of Authorization No. 20 to use the "NR" stamp expires July 17, 2001

Date _____, 19____, Signed Toledo Edison Nuclear Safety & Inspection Manager
Repair Organization Authorized Representative Title

CERTIFICATE OF INSPECTION

I, the undersigned, holding a valid commission issued by the National Board of Boiler and Pressure Vessel Inspectors, and certificate of competency issued by the State or Province of Ohio and employed by Factory Mutual Ins. Co. of Johnston, R.I. have inspected the repair, modification, or replacement described in this report

on _____, 19____ and state that to the best of my knowledge and belief, this repair, modification, or replacement has been made or constructed in accordance with Section XI and Section III of the ASME Code and the National Board rules as defined in the publication NB-65 and NB-102, current editions.

By signing this certificate, neither the Inspector nor his employer makes any warranty, expressed or implied, concerning the repair, modification or replacement described in this report. Furthermore, neither the Inspector nor his employer shall be liable in any manner for any personal injury or property damage or a loss of any kind arising from or connected with this inspection.

Authorized Inspector's Signature

Date

Commissions _____
National Board, State, Province and Endorsements

* Factory Mutual Ins. Co

FORM NIS-2 / NR-1 / NVR-1 OWNER'S REPORT FOR REPAIRS OR REPLACEMENTS
 As Required by the Provisions of the ASME Code Section XI

1. Owner Toledo Edison Company Date 5/2/00
300 Madison Ave., Toledo, OH 43652 Sheet 1 of 1
 2. Plant Davis-Besse Nuclear Power Station Unit #1
5501 N. SR 2, Oak Harbor, OH 43449 WO 99-005149-024
 3. Work Performed By Toledo Edison Company Type Code Stamp NR
5501 N. SR 2, Oak Harbor, OH 43449 Authorization NO. 20
 Expiration Date 7-17-01

4. Identification of Systems MAIN STEAM SYSTEM EBB-1-SR11 West (A-40) (SUB 083-01)

5. (a) Applicable Construction Code ANSI B31.1 Edition 19 87 Addendum N/A Code Case N/A
 (b) Applicable Edition of Section XI Utilized for Repairs or Replacements 19 86
 (c) Design Responsibilities ITT GRINNELL

6. Identification of Components Repaired or Replaced and Replacement Components

Name of Component	Name of Manufacturer	Manufacturer's Serial NO.	National Board NO.	Other Identification/Jurisdictional NO.	Year Built	Repaired or Replacement	ASME Code Stamped
Snubber	ITT GRINNELL	11891	N/A	A-40	Unk	Replaced	No
Snubber	ITT GRINNELL	11892	N/A	A-40	Unk	Replacement	No

7. Description of Work

ITT Grinnell hydraulic snubber S/N11891 was removed and replaced with a like-for-like snubber S/N11892.
 Replacement snubber was rebuilt by WO 99-005150-007. Replacement snubber was functionally tested satisfactory prior to installation.

8. Tests Conducted: Hydrostatic (NO) Pneumatic (NO) Nominal Operating Pressure (NO) Other (NO)
 Pressure N/A psi Test Temperature N/A °F

Pressure Relief Valves:
 Service N/A Size N/A
 Opening Pressure N/A Blowdown (if applicable) N/A
 Set Pressure and Blowdown Adjustments Made Using N/A
 at N/A

NOTE: Supplemental sheets in the form of lists, sketches, or drawings may be used, provided (1) size is 8 1/2 x 11 in. (2) Information in items 1 through 6 on this report is included on each sheet, and (3) each sheet is numbered and the number of sheets is recorded at the top of this form.

FORM NIS-2 /NR-1 /NVR-1

9. REMARKS

Applicable Manufacturers Data Reports to be Attached

NA

CERTIFICATE OF COMPLIANCE

We certify that the statements made in the report are correct and this replacement conforms to the rules of the ASME Code Section XI.

Type Code Symbol NR Certificate of Authorization No. 20 Expiration Date 7-17-01

Signed [Signature] Nuclear Safety & Inspection Manager Date 5/2/2000
Owner or Owner's Designee, Title

CERTIFICATE OF INSERVICE INSPECTION

I, the undersigned, holding a valid commission issued by the National Board of Boiler and Pressure Vessel Inspectors in the State or Province of Ohio and employed by Factory Mutual Ins. Co. of Johnston, R.I. have inspected the components described in this Owner's Report during the period 9-28-99 to 5-2-00, and state that to the best of my knowledge and belief, the Owner has performed examinations and taken corrective measures described in this Owner's Report in accordance with the requirements of the ASME Code, Section XI.

By signing this certificate, neither the Inspector nor his employer makes any warranty, expressed or implied, concerning the examinations and corrective measures described in the Owner's Report. Furthermore, neither the Inspector nor his employer shall be liable in any manner for any personal injury or property damage or a loss of any kind arising from or connected with this inspection.

Inspector's Signature [Signature] Date 5-2-00 Commissions NB9901 N.I.A. Ohio
National Board, State, Province and Endorsements Comm

CERTIFICATE OF COMPLIANCE

We certify that the statements made in this report are correct and the repair, modification, or replacement of the pressure relief device described above conforms to Section XI and Section III of the ASME Code and the National Board rules as defined in publications NB-65 and NB-102, current editions.

Certificate of Authorization No. 316 to use the "VR" stamp expires Dec. 13, 2001

Certificate of Authorization No. 20 to use the "NR" stamp expires July 17, 2001

Date , 19 , Signed Toledo Edison Nuclear Safety & Inspection Manager
Repair Organization Authorized Representative Title

CERTIFICATE OF INSPECTION

I, the undersigned, holding a valid commission issued by the National Board of Boiler and Pressure Vessel Inspectors, and certificate of competency issued by the State or Province of Ohio and employed by Factory Mutual Ins. Co. of Johnston, R.I. have inspected the repair, modification, or replacement described in this report

on , 19 and state that to the best of my knowledge and belief, this repair, modification, or replacement has been made or constructed in accordance with Section XI and Section III of the ASME Code and the National Board rules as defined in the publication NB-65 and NB-102, current editions.

By signing this certificate, neither the Inspector nor his employer makes any warranty, expressed or implied, concerning the repair, modification or replacement described in this report. Furthermore, neither the Inspector nor his employer shall be liable in any manner for any personal injury or property damage or a loss of any kind arising from or connected with this inspection.

Authorized Inspector's Signature Date Commissions
National Board, State, Province and Endorsements
* Factory Mutual Ins. Co

First Energy Nuclear Operating, 76 South Main St., Akron, Ohio 44308

Davis-Besse Nuclear Power Station Unit #1, 5501 N. St. Rt. 2, Oak Harbor, Ohio 43449

Commercial Service Date: 11/27/77

FORM NIS-2 / NR-1 / NVR-1 OWNER'S REPORT FOR REPAIRS OR REPLACEMENTS As Required by the Provisions of the ASME Code Section XI

1. Owner Toledo Edison Company Date 10/26/1999
Name
300 Madison Ave., Toledo, OH 43652
Address
 Sheet 1 of 1

2. Plant Davis-Besse Nuclear Power Station Unit # 1
Name
5501 N. SR 2, Oak Harbor, OH 43449
Address
WO# 99-005152-000
Repair Organization P.O. No., Job No., etc.

3. Work Performed By Toledo Edison Company Type Code Stamp NR / VR
Name
5501 N. SR 2, Oak Harbor, OH 43449
Address
 Authorization NO. 20 / 316
 Expiration Date 7-17-01 / 12-13-01

4. Identification of Systems REACTOR COOLANT SYSTEM SPARE PRESSURIZER SAFETY VALVE (RC13A)

5. (a) Applicable Construction Code ASME III CL1 Edition 19 58 Addendum 570 Code Case 1440
 (b) Applicable Edition of Section XI Utilized for Repairs or Replacements 19 86
 (c) Design Responsibilities CROSBY VALVE & GAGE COMPANY

6. Identification of Components Repaired or Replaced and Replacement Components

Name of Component	Name of Manufacturer	Manufacturer's Serial NO.	National Board NO.	Other Identification/Jurisdictional NO.	Year Built	Repaired Replaced or Replacement	ASME Code Stamped
DISC INSERT	CROSBY	NA	NA	N94145-32-0004	NA	Replacement	No
SPRG WSR ASSY	CROSBY	NA	NA	K76740-31-0001	NA	Replacement	No

7. Description of Work

SPARE PRESSURIZER SAFETY VALVE RC13A, IDENTIFIED BY SERIAL NUMBER N54891-00-0002, WAS REBUILT REPLACING THE DISC INSERT AND SPRING WASHER ASSEMBLY. THE VALVE WAS THEN SET PYESSURE TESTED WITHIN NORMAL OPERATING PRESSURE AND TEMPERATURE RANGES. VALVE TESTED SATISFACTORY.

DISC INSERT - PN # N94145-32-0004 B&W SPEC 1028/NSS-14/0370, PO # 072-G-71127A-MI, ASME III 1968 ED, §70 ADD, CLASS 1

SPRING WASHER ASSEMBLY - PO # 8026220 D91, SN K76740-31-0001, ASME III 1968 ED, §70 ADD, CLASS 1

8. Tests Conducted: Hydrostatic (NO) Pneumatic (NO) Nominal Operating Pressure (YES) Other (NO)
 Pressure 2500 psi Test Temperature 485 °F

Pressure Relief Valves:
 Service STEAM Size 4X M1 X6
 Opening Pressure 2489 Blowdown (if applicable) NA
 Set Pressure and Blowdown Adjustments Made Using SATURATED STEAM
(Boiling Medium)
 at NWS TECHNOLOGIES, INC., SPARTANBURG, SOUTH CAROLINA
(Location)

NOTE: Supplemental sheets in the form of lists, sketches, or drawings may be used, provided (1) size is 8 1/2 x 11 in. (2) Information in items 1 through 6 on this report is included on each sheet, and (3) each sheet is numbered and the number of sheets is recorded at the top of this form.

FORM NIS-2 /NR-1 / NVR-1

8. REMARKS		Applicable Manufacturer's Data Reports to be Attached
NA		

CERTIFICATE OF COMPLIANCE

We certify that the statements made in the report are correct and this repair conforms to the rules of the ASME Code Section XI.

Type Code Symbol NR Certificate of Authorization No. 20 Expiration Date 7-17-01

Signed St. W. S. Starn Nuclear Safety & Inspection Manager Date 11/8 19 99

Owner or Owner's Designee, Title

CERTIFICATE OF INSERVICE INSPECTION

I, the undersigned, holding a valid commission issued by the National Board of Boiler and Pressure Vessel Inspectors in the State or Province of Ohio and employed by Protection Mutual * of Park Ridge, Illinois have inspected the components described in this Owner's Report during the period 9/22/99 to 11/9/99, and state that to the best of my knowledge and belief, the Owner has performed examinations and taken corrective measures described in this Owner's Report in accordance with the requirements of the ASME Code, Section XI.

By signing this certificate, neither the Inspector nor his employer makes any warranty, expressed or implied, concerning the examinations and corrective measures described in the Owner's Report. Furthermore, neither the Inspector nor his employer shall be liable in any manner for any personal injury or property damage or a loss of any kind arising from or connected with this inspection.

Thomas H. Page
Inspector's Signature

11/9/99
Date

Commissions NR 9330 "N I A" OHIO COMM.
National Board, State, Province and Endorsements

CERTIFICATE OF COMPLIANCE

We certify that the statements made in this report are correct and the repair, modification, or replacement of the pressure relief device described above conforms to Section XI and Section III of the ASME Code and the National Board rules as defined in publications NB-65 and NB-102, current editions.

Certificate of Authorization No. 316 to use the "VR" stamp expires Dec. 13, 2001

Certificate of Authorization No. 20 to use the "NR" stamp expires July 17, 2001

Date 11/8, 19 99, Signed Toledo Edison St. W. S. Starn Nuclear Safety & Inspection Manager
Repair Organization Authorized Representative Title

CERTIFICATE OF INSPECTION

I, the undersigned, holding a valid commission issued by the National Board of Boiler and Pressure Vessel Inspectors, and certificate of competency issued by the State or Province of Ohio and employed by Protection Mutual *

of Park Ridge, Illinois have inspected the repair, modification, or replacement described in this report

on NOV. 9, 19 99 and state that to the best of my knowledge and belief, this repair, modification, or replacement has been made or constructed in accordance with Section XI and Section III of the ASME Code and the National Board rules as defined in the publication NB-65 and NB-102, current editions.

By signing this certificate, neither the Inspector nor his employer makes any warranty, expressed or implied, concerning the repair, modification or replacement described in this report. Furthermore, neither the Inspector nor his employer shall be liable in any manner for any personal injury or property damage or a loss of any kind arising from or connected with this inspection.

Thomas H. Page
Authorized Inspector's Signature

11/9/99
Date

Commissions NR 9330 "N I A" OHIO COMM.
National Board, State, Province and Endorsements

* Factory Mutual Engineering Association

* FACTORY MUTUAL INS. CO.

FORM NIS-2 / NR-1 / NVR-4 OWNER'S REPORT FOR REPAIRS OR REPLACEMENTS
 As Required by the Provisions of the ASME Code Section XI

1. Owner Toledo Edison Company Date 5/23/00
300 Madison Ave., Toledo, OH 43652 Sheet 1 of 1
 2. Plant Davis-Besse Nuclear Power Station Unit # 1
5501 N. SR 2, Oak Harbor, OH 43449 WO 99-006520-001
 3. Work Performed By Toledo Edison Company Type Code Stamp NR
5501 N. SR 2, Oak Harbor, OH 43449 Authorization NO. 20
 Expiration Date 7-17-01
 4. Identification of Systems Process & Area Radiation Monitoring (Subsys 079-01)
 5. (a) Applicable Construction Code ASME III C1.2 Edition 19 71 Addendum N/A Code Case N/A
 (b) Applicable Edition of Section XI Utilized for Repairs or Replacements 19 86
 (c) Design Responsibilities Toledo Edison Co.

6. Identification of Components Repaired or Replaced and Replacement Components

Name of Component	Name of Manufacturer	Manufacturer's Serial NO.	National Board NO.	Other Identification/Jurisdictional NO.	Year Built	Repaired Replaced or Replacement	ASME Code Stamped
Radiation Monitor.	Toledo Edison Co.	N/A	N/A	N/A	2000	Repaired	No

7. Description of Work

Toledo Edison Co implemented Modification 99-0050 which involved fabrication and installation of an additional new line (FSK-M-HCB-38-28) tied into an existing line (FSK-M-HCB-38-16). This was to facilitate the draining of moisture from a radiation monitoring systems (RE4597AA).
 This modification involved the fabrication and installation of new piping, valves, collection tank, and one new hanger (FSK-M-HCB-38-28-H).
 All additional materials installed are referenced and listed in the body of this work order. After installation,
 a VT-2 pneumatic test was performed satisfactory.

8. Tests Conducted: Hydrostatic (NO) Pneumatic (YES) Nominal Operating Pressure (NO) Other (NO)
 Pressure 70 psi Test Temperature NOT °F
 Pressure Relief Valves:
 Service N/A Size N/A
 Opening Pressure N/A Blowdown (if applicable) N/A
 Set Pressure and Blowdown Adjustments Made Using N/A
 at N/A

NOTE: Supplemental sheets in the form of lists, sketches, or drawings may be used, provided (1) size is 8 1/2 x 11 in. (2) information in items 1 through 6 on this report is included on each sheet, and (3) each sheet is numbered and the number of sheets is recorded at the top of this form.

12 RFO Inservice Inspection Summary

Page 234

First Energy Nuclear Operating, 76 South Main St., Akron, Ohio 44308

Davis-Besse Nuclear Power Station Unit #1, 5501 N. St. Rt. 2, Oak Harbor, Ohio 43449

Commercial Service Date: 11/27/77

FORM NIS-2 /NR-1 /NVR-4-

9. REMARKS

Applicable Manufacturers Data Reports to be Attached

NA

CERTIFICATE OF COMPLIANCE

We certify that the statements made in the report are correct and this replacement conforms to the rules of the ASME Code Section XI.

Type Code Symbol NR Certificate of Authorization No. 20 Expiration Date 7-17-01

Signed [Signature] Nuclear Safety & Inspection Manager Date 5/25/00
Owner or Owner's Designee, Title

CERTIFICATE OF INSERVICE INSPECTION

I, the undersigned, holding a valid commission issued by the National Board of Boiler and Pressure Vessel Inspectors in the State or Province of Ohio and employed by Factory Mutual Ins. Co. of Johnston, R.I. have inspected the components described in this Owner's Report during the period 3/22/00 to 5/25/00, and state that to the best of my knowledge and belief, the Owner has performed examinations and taken corrective measures described in this Owner's Report in accordance with the requirements of the ASME Code, Section XI.

By signing this certificate, neither the Inspector nor his employer makes any warranty, expressed or implied, concerning the examinations and corrective measures described in the Owner's Report. Furthermore, neither the Inspector nor his employer shall be liable in any manner for any personal injury or property damage or a loss of any kind arising from or connected with this inspection.

Inspector's Signature Thomas H. Papp Date 5/25/00 Commissions NB9330 N I A OHIO COMM.
National Board, State, Province and Endorsements

CERTIFICATE OF COMPLIANCE

We certify that the statements made in this report are correct and the repair, modification, or replacement of the pressure relief device described above conforms to Section XI and Section III of the ASME Code and the National Board rules as defined in publications NB-65 and NB-102, current editions.

Certificate of Authorization No. 316 to use the "VR" stamp expires Dec. 13, 2001

Certificate of Authorization No. 20 to use the "NR" stamp expires July 17, 2001

Date _____, 19____, Signed Toledo Edison [Signature] Nuclear Safety & Inspection Manager
Repair Organization Authorized Representative Title

CERTIFICATE OF INSPECTION

I, the undersigned, holding a valid commission issued by the National Board of Boiler and Pressure Vessel Inspectors, and certificate of competency issued by the State or Province of Ohio and employed by Factory Mutual Ins. Co. of Johnston, R.I. have inspected the repair, modification, or replacement described in this report

on _____, 19____ and state that to the best of my knowledge and belief, this repair, modification, or replacement has been made or constructed in accordance with Section XI and Section III of the ASME Code and the National Board rules as defined in the publication NB-65 and NB-102, current editions.

By signing this certificate, neither the Inspector nor his employer makes any warranty, expressed or implied, concerning the repair, modification or replacement described in this report. Furthermore, neither the Inspector nor his employer shall be liable in any manner for any personal injury or property damage or a loss of any kind arising from or connected with this inspection.

Authorized Inspector's Signature _____ Date _____ Commissions _____
National Board, State, Province and Endorsements

* Factory Mutual Ins. Co

FORM NIS-2 / NR-1 / NVR-1 OWNER'S REPORT FOR REPAIRS OR REPLACEMENTS

As Required by the Provisions of the ASME Code Section XI

1. Owner Toledo Edison Company Date 5/17/00
Name
300 Madison Ave., Toledo, OH 43652
Address
 2. Plant Davis-Besse Nuclear Power Station Unit # 1
Name
5501 N. SR 2, Oak Harbor, OH 43449
Address
 3. Work Performed By Toledo Edison Company Type Code Stamp NR
Name
5501 N. SR 2, Oak Harbor, OH 43449
Address
 Authorization NO. 20
 Expiration Date 7-17-01

4. Identification of Systems Steam Generator 1-1 Aux Fd Wtr Hdr - Riser #4 (Subsys 063-01)
 5. (a) Applicable Construction Code ASME III CL2 Edition 19 71 Addendum W72 Code Case N/A
 (b) Applicable Edition of Section XI Utilized for Repairs or Replacements 19 86
 (c) Design Responsibilities Babcock & Wilcox

6. Identification of Components Repaired or Replaced and Replacement Components

Name of Component	Name of Manufacturer	Manufacturer's Serial NO.	National Board NO.	Other Identification/Jurisdictional NO.	Year Built	Repaired Replaced or Replacement	ASME Code Stamped
Riser #4 (1 nut)	Nova Machine Prod	HT# 8099455	N/A	N/A	1998	Replacement	No

7. Description of Work

During disassembly of steam generator 1-1 auxiliary feed water header riser #4 for maintenance, one 3/4"-10 heavy hex nut was dropped and not found. This nut was replaced with a like for like replacement. After installation, a VT-2 inservice test was performed satisfactory.

8. Tests Conducted: Hydrostatic (NO) Pneumatic (NO) Nominal Operating Pressure (YES) Other (NO)
 Pressure 870 psi Test Temperature NOT °F
 Pressure Relief Valves:
 Service N/A Size N/A
 Opening Pressure N/A Blowdown (if applicable) N/A
 Set Pressure and Blowdown Adjustments Made Using N/A
 at N/A

NOTE: Supplemental sheets in the form of lists, sketches, or drawings may be used, provided (1) size is 8 1/2 x 11 in. (2) Information in items 1 through 6 on this report is included on each sheet, and (3) each sheet is numbered and the number of sheets is recorded at the top of this form.

12 RFO Inservice Inspection Summary

Page 236

First Energy Nuclear Operating, 76 South Main St., Akron, Ohio 44308

Davis-Besse Nuclear Power Station Unit #1, 5501 N. St. Rt. 2, Oak Harbor, Ohio 43449

Commercial Service Date: 11/27/77

FORM NIS-2 / NR-1 / NVR-1

9. REMARKS

Applicable Manufacturers Data Reports to be Attached

NA

CERTIFICATE OF COMPLIANCE

We certify that the statements made in the report are correct and this replacement conforms to the rules of the ASME Code Section XI.

Type Code Symbol NR Certificate of Authorization No. 20 Expiration Date 7-17-01

Signed J.W.S. [Signature] Nuclear Safety & Inspection Manager Date 5/17/00

Owner or Owner's Designee, Title

CERTIFICATE OF INSERVICE INSPECTION

I, the undersigned, holding a valid commission issued by the National Board of Boiler and Pressure Vessel Inspectors in the State or Province of Ohio and employed by Factory Mutual Ins. Co. of Johnston, R.I. have inspected the components described in this Owner's Report during the period 4-23-00 to 5-17-00, and state that to the best of my knowledge and belief, the Owner has performed examinations and taken corrective measures described in this Owner's Report in accordance with the requirements of the ASME Code, Section XI.

By signing this certificate, neither the Inspector nor his employer makes any warranty, expressed or implied, concerning the examinations and corrective measures described in the Owner's Report. Furthermore, neither the Inspector nor his employer shall be liable in any manner for any personal injury or property damage or a loss of any kind arising from or connected with this inspection.

Inspector's Signature

Date 5-17-00Commissions NB9901 N.E.A. Ohio
National Board, State, Province and Endorsements

CERTIFICATE OF COMPLIANCE

We certify that the statements made in this report are correct and the repair, modification, or replacement of the pressure relief device described above conforms to Section XI and Section III of the ASME Code and the National Board rules as defined in publications NB-65 and NB-102, current editions.

Certificate of Authorization No. 318 to use the "VR" stamp expires Dec. 13, 2001

Certificate of Authorization No. 20 to use the "NR" stamp expires July 17, 2001

Date 19, Signed Toledo Edison Nuclear Safety & Inspection Manager
Repair Organization Authorized Representative Title

CERTIFICATE OF INSPECTION

I, the undersigned, holding a valid commission issued by the National Board of Boiler and Pressure Vessel Inspectors, and certificate of competency issued by the State or Province of Ohio and employed by Factory Mutual Ins. Co.

of Johnston, R.I. have inspected the repair, modification, or replacement described in this report

on 5-17-00, 1900 and state that to the best of my knowledge and belief, this repair, modification, or replacement has been made or constructed in accordance with Section XI and Section III of the ASME Code and the National Board rules as defined in the publication NB-65 and NB-102, current editions.

By signing this certificate, neither the Inspector nor his employer makes any warranty, expressed or implied, concerning the repair, modification or replacement described in this report. Furthermore, neither the Inspector nor his employer shall be liable in any manner for any personal injury or property damage or a loss of any kind arising from or connected with this inspection.

Authorized Inspector's Signature

Date

Commissions

National Board, State, Province and Endorsements

* Factory Mutual Ins. Co

First Energy Nuclear Operating, 76 South Main St., Akron, Ohio 44308

Davis-Besse Nuclear Power Station Unit #1, 5501 N. St. Rt. 2, Oak Harbor, Ohio 43449

Commercial Service Date: 11/27/77

FORM NIS-2 / NR-1 / NVR-1 OWNER'S REPORT FOR REPAIRS OR REPLACEMENTS As Required by the Provisions of the ASME Code Section XI

1. Owner Toledo Edison Company Date 05/31/2000
300 Madison Ave., Toledo, OH 43652 Sheet 1 of 2
 2. Plant Davis-Besse Nuclear Power Station Unit # 1
5501 N. SR 2, Oak Harbor, OH 43449 WO # 00-001712-000
 3. Work Performed By Toledo Edison Company Type Code Stamp NR
5501 N. SR 2, Oak Harbor, OH 43449 Authorization NO. 20
 Expiration Date 7-17-01

4. Identification of Systems Main Steam / MS 700 Main Steam Line #2 Warmup Isolation Valve (SUB83-01)

5. (a) Applicable Construction Code ASME III CL2 Edition 19 Z1 Addendum W72 Code Case NA

(b) Applicable Edition of Section XI Utilized for Repairs or Replacements 19 86

(c) Design Responsibilities Velan Inc. / Toledo Edison Company

6. Identification of Components Repaired or Replaced and Replacement Components

Name of Component	Name of Manufacturer	Manufacturer's Serial NO.	National Board NO.	Other Identification/Jurisdictional NO.	Year Built	Repaired Replaced or Replacement	ASME Code Stamped
1-1/2" Gate Valve	Velan Inc.	941076-2	NA	MS 700	1994	Replacement	NA
1-1/2" Sch. 80 Pipe	Energy & Process Corp.	HT# 263444	NA	NA	2000	Replacement	NA
1-1/2" Gate Valve	Velan Engineering Co.	227-14	NA	MS 700	1972	Replaced	NA

7. Description of Work

Removed existing gate valve and associated piping and replaced with new like for like replacement.

1-1/2" Velan Gate Valve, S/N 941076-2, P.O. # S052842 D93, ASME III, Cl 1, 1989 Ed., No Add.

1-1/2" Sch. 80 pipe, seamless, SA 106 GR. B, HT # 263444, P.O. # 7022920, ASME III, Cl 2, Subsection NC, 1989 Ed., No Add.

After installation, a VT-2 Inservice Test was performed satisfactorily.

8. Tests Conducted: Hydrostatic (NO) Pneumatic (NO) Nominal Operating Pressure (YES) Other (NO)
 Pressure 870 psi Test Temperature NOT °F
 Pressure Relief Valves:
 Service NA Size NA
 Opening Pressure NA Blowdown (if applicable) NA
 Set Pressure and Blowdown Adjustments Made Using NA
 at NA

NOTE: Supplemental sheets in the form of lists, sketches, or drawings may be used, provided (1) size is 8 1/2 x 11 in. (2) Information in items 1 through 6 on this report is included on each sheet, and (3) each sheet is numbered and the number of sheets is recorded at the top of this form.

FORM NIS-2 /NR-1 /NVR-1

8. REMARKS	Applicable Manufacturers Data Reports to be Attached
Veian Inc.	

CERTIFICATE OF COMPLIANCE

We certify that the statements made in the report are correct and this replacement conforms to the rules of the ASME Code Section XI.

Type Code Symbol NR Certificate of Authorization No. 20 Expiration Date 7-17-01

Signed R. W. Starnes Nuclear Safety & Inspection Manager Date 5/31/00 19
Owner or Owner's Designee, Title

CERTIFICATE OF INSERVICE INSPECTION

I, the undersigned, holding a valid commission issued by the National Board of Boiler and Pressure Vessel Inspectors in the State or Province of Ohio and employed by Factory Mutual Ins. Co. of Johnston, R.I. have inspected the components described in this Owner's Report during the period 4/23/00 to 6/6/00, and state that to the best of my knowledge and belief, the Owner has performed examinations and taken corrective measures described in this Owner's Report in accordance with the requirements of the ASME Code, Section XI.

By signing this certificate, neither the Inspector nor his employer makes any warranty, expressed or implied, concerning the examinations and corrective measures described in the Owner's Report. Furthermore, neither the Inspector nor his employer shall be liable in any manner for any personal injury or property damage or a loss of any kind arising from or connected with this inspection.

Inspector's Signature Thomas H. Page Date 6/6/00 Commissions NB 9330 N I A OHIO
National Board, State, Province and Endorsements COMM.

CERTIFICATE OF COMPLIANCE

We certify that the statements made in this report are correct and the repair, modification, or replacement of the pressure relief device described above conforms to Section XI and Section III of the ASME Code and the National Board rules as defined in publications NB-65 and NB-102, current editions.

Certificate of Authorization No. 316 to use the "VR" stamp expires Dec. 13, 2001.

Certificate of Authorization No. 20 to use the "NR" stamp expires July 17, 2001.

Date , 19 , Signed Toledo Edison Nuclear Safety & Inspection Manager
Repair Organization Authorized Representative Title

CERTIFICATE OF INSPECTION

I, the undersigned, holding a valid commission issued by the National Board of Boiler and Pressure Vessel Inspectors, and certificate of competency issued by the State or Province of Ohio and employed by Factory Mutual Ins. Co. of Johnston, R.I. have inspected the repair, modification, or replacement described in this report

on , 19 and state that to the best of my knowledge and belief, this repair, modification, or replacement has been made or constructed in accordance with Section XI and Section III of the ASME Code and the National Board rules as defined in the publication NB-65 and NB-102, current editions.

By signing this certificate, neither the Inspector nor his employer makes any warranty, expressed or implied, concerning the repair, modification or replacement described in this report. Furthermore, neither the Inspector nor his employer shall be liable in any manner for any personal injury or property damage or a loss of any kind arising from or connected with this inspection.

Authorized Inspector's Signature Date Commissions
National Board, State, Province and Endorsements
* Factory Mutual Ins. Co

First Energy Nuclear Operating, 76 South Main St., Akron, Ohio 44308

Davis-Besse Nuclear Power Station Unit #1, 5501 N. St. Rt. 2, Oak Harbor, Ohio 43449

Commercial Service Date: 11/27/77

DATA PACKAGE NO. 941076 P.5
FORM NPV-1 CERTIFICATE HOLDERS' DATA REPORT FOR NUCLEAR PUMPS OR VALVES *
As Required by the Provisions of the ASME Code, Section III, Division 1

Pg. 1 of 2

1. Manufactured and certified by VELAN INC. 2125 WARD AVE. MTL. CANADA
(name and address of N Certificate Holder)
2. Manufactured for TOLKOD KOLSON COMPANY
(name and address of purchaser)
3. Location of installation DAVIS-BESSE STATION
(name and address)
4. Model No., Series No., or Type GATE Drawing PL-76800-M01 Rev F CRM N/A
5. ASME Code, Section III, Division 1: 1989 NONE 1 N/A
(edition) (addenda date) (class) (Code Case no.)
6. Pump or valve VALVE Nominal inlet size 1 1/2" Outlet size 1 1/2"
(in.) (in.)
7. Material: Body SA-105 Bonnet SA-105 Disk SA-351 CF8M Bolting SA-193B7
SA-19472H

(a) Cert. Holder's Serial No.	(b) Nat'l Board No.	(c) Body Serial No.	(d) Bonnet Serial No.	(e) Disk Serial No.
941076-1	N/A	H/C: AWV	H/C: V-1	H/C: V-1
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941076-99		"	"	"
941076-100		"	"	"

Supplemental information in form of data, sketches, or drawings may be used provided (1) size is 8 1/2 x 11 (2) information is clearly legible (3) (4) (5) (6) (7) (8) (9) (10) (11) (12) (13) (14) (15) (16) (17) (18) (19) (20) (21) (22) (23) (24) (25) (26) (27) (28) (29) (30) (31) (32) (33) (34) (35) (36) (37) (38) (39) (40) (41) (42) (43) (44) (45) (46) (47) (48) (49) (50) (51) (52) (53) (54) (55) (56) (57) (58) (59) (60) (61) (62) (63) (64) (65) (66) (67) (68) (69) (70) (71) (72) (73) (74) (75) (76) (77) (78) (79) (80) (81) (82) (83) (84) (85) (86) (87) (88) (89) (90) (91) (92) (93) (94) (95) (96) (97) (98) (99) (100) (101) (102) (103) (104) (105) (106) (107) (108) (109) (110) (111) (112) (113) (114) (115) (116) (117) (118) (119) (120) (121) (122) (123) (124) (125) (126) (127) (128) (129) (130) (131) (132) (133) (134) (135) (136) (137) (138) (139) (140) (141) (142) (143) (144) (145) (146) (147) (148) (149) (150) (151) (152) (153) (154) (155) (156) (157) (158) (159) (160) (161) (162) 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FORM NPV-1 (Back - Pg. 2 of 2)

P.6

Certificate Holder's Serial No. 941076

8. Design conditions 1975 psi 100 °F or valve pressure class 800 (1)
(pressure) (temperature)
9. Cold working pressure 1975 psi at 100°F
10. Hydrostatic test 3000 psi. Disk differential test pressure 2200 psi
11. Remarks: MATERIALS MEET ASME SECTION II EDITION: 1989 ADDENDA: NONE

CERTIFICATE OF DESIGN

Design Specification certified by J.M. FARRELL P.E. State QUE Reg. no. 30039
Design report certified by S. ISBITSKY P.E. State QUE Reg. no. 22015

CERTIFICATE OF COMPLIANCE

We certify that the statements made in this report are correct and that this pump or valve conforms to the rules for construction of the ASME Code, Section III, Division 1.
N Certificate of Authorization No. N-2797-1 Expires MAY 2, 85
Date JUNE 10, 84 Name VELAN INC. Signed [Signature]
(N Certificate Holder) (Authorized Representative)

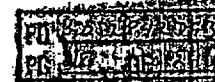
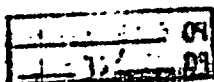
CERTIFICATE OF INSPECTION

I, the undersigned, holding a valid commission issued by the National Board of Boiler and Pressure Vessel Inspectors and the State or Province of QUEBEC and employed by PROVINCE of QUEBEC have inspected the pump or valve, described in this Data Report on JUNE 10, 84, and state that to the best of my knowledge and belief, the Certificate Holder has constructed this pump or valve, in accordance with the ASME Code, Section III, Division 1.

By signing this certificate, neither the inspector nor his employer makes any warranty, expressed or implied, concerning the component described in this Data Report. Furthermore, neither the inspector nor his employer shall be liable in any manner for any personal injury or property damage or a loss of any kind arising from or connected with this inspection.

Date 940610 signed [Signature] Commissions JACQUES MARCHAND
(Authorized Inspector) (Authorized Representative)

(1) For manually operated valves only.



12 RFO Inservice Inspection Summary

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First Energy Nuclear Operating, 76 South Main St., Akron, Ohio 44308

Davis-Besse Nuclear Power Station Unit #1, 5501 N. St. Rt. 2, Oak Harbor, Ohio 43449

Commercial Service Date: 11/27/77

FORM NR-1 REPORT OF REPAIR MODIFICATION OR REPLACEMENT (X)
TO NUCLEAR COMPONENTS AND SYSTEMS IN NUCLEAR POWER PLANTS

1. Work performed by Framatome Technologies Inc. First Energy P.O.s 7022415 & 7005137
(name of NR certificate holder) (P.O. no., job no., etc.)
3315 Old Forest Road Lynchburg, VA 24501 (address)
2. Owner First Energy Company
76 South Main St. Akron Ohio 44308 (address)
3. Name, address and identification of nuclear power plant Davis-Besse Nuclear Power Station
5501 N. SR 2, Oak Harbor, Ohio 43449
4. System Cooling Water Manifolds for Control Rod Drives
5. a: Items Which Required Repair, Modification, or Replacement Activities

Identification								Construction Code			Activity	
No	Type of Item	Mfg. Name	Mfg. Serial No.	Nat'l Bd. No.	Jurisd. No.	Other	Year Built	Name/Section/Division	Edition/Addenda	Code Case(s)	Code Class	Repair/Mod/Replace
1	(2)	(1)	(2)	N/A	N/A	N/A	N/A	(3)	(4)	N/A	2	Replace
2												
3												
4												
5												
6												
7												
8												
9												
10												
11												
12												

5. b: Items Installed During Replacement Activities

Identification								Construction Code			
Type of Item	Installed or Replaced 5a Item No.	Mfg. Name	Mfg. Serial No.	Nat'l Bd. No.	Jurisd. No.	Other	Year Built	Name/Section/Division	Edition/Addenda	Code Case(s)	Code Class
(6)	1	(5)	(6)	N/A	N/A	N/A	N/A	(3)	(4)	None	2
(6)	1	(5)	(6)	N/A	N/A	N/A	N/A	(3)	(4)	None	2

6. ASME Code Section XI applicable for inservice inspection: 1986 No No
(edition) (addenda) (Code Cases(s))
7. ASME Code Section XI used for repairs, modifications, or replacements: 1986 No No
(edition) (addenda) (Code Cases(s))
8. Construction Code used for repairs, modifications, or replacements: 1971 No No
(edition) (addenda) (Code Cases(s))
9. Design responsibilities: (7)
10. Tests conducted (8) hydrostatic pneumatic design pressure Y pressure _____ psi Code Case(s) _____

First Energy Nuclear Operating, 76 South Main St., Akron, Ohio 44308

Davis-Besse Nuclear Power Station Unit #1, 5501 N. St. Rt. 2, Oak Harbor, Ohio 43449

Commercial Service Date: 11/27/77

11. Description of work:

(use of properly identified additional sheet(s) or sketch(es) is acceptable)

Replaced flange bolting at cooling water manifolds for control rod drives.

12. Remarks:

(1) First Energy

(2) Replacement of flange bolting at cooling water manifolds for control rod drives

(3) ASME Section III, Division I Class 2

(4) Construction Code ASME III, 1971 Edition, No Addenda

(5) First Energy

(6) Stud-Continuous Thread; 5/8 in, 5-1/2 in, 11 TPI (Heat # 715200), Nut-Hex; Heavy Hex, 5/8 in, 11 TPI (Heat # 713514)

(7) Framatome Technologies, Inc

(8) By others - First Energy

CERTIFICATE OF COMPLIANCE

I, Victor P. Montalbano Certify that to the best of my knowledge and belief the statements made in this report are correct and the repair, modification or replacement activities described above conforms to Section XI of the ASME Code and The National Board Inspection Code "NR" rules.

National Board Certificate of Authorization No. NR-64 to use the "NR" stamp expires May 17 2000

NR Certificate Holder Framatome Technologies Inc.

Date May 31, 2000 Signed Victor P. Montalbano (name) Manager, FTI Quality (title)
(authorized representative)

CERTIFICATE OF INSPECTION

I, Thomas G. Laps holding a valid commission issued by The National Board of Boiler and Pressure Vessel Inspectors and certificate of competency issued by the jurisdiction of Ohio and employed by F.M.I.C.* of Johnston, RI have

inspected the repair, modification or replacement described in this report on April 28, 2000 and state that to the best of my knowledge and belief, this repair, modification or replacement activity has been completed in accordance with Section XI of the ASME Code and the National Board Inspection Code "NR" rules.

By signing this certificate, neither the undersigned nor my employer makes any warranty, expressed or implied, concerning the work described in this report. Furthermore, neither the undersigned nor my employer shall be liable in any manner for any personal injury, property damage or loss of any kind arising from or connected with this inspection.

Date 6/8/00 Signed Thomas G. Laps Commissions NB9330 N 1 A Ohio Comm
(inspector) (National Board (incl. endorsements), jurisdiction, and no.)
*Factory Mutual Insurance Company

12 RFO Inservice Inspection Summary

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First Energy Nuclear Operating, 76 South Main St., Akron, Ohio 44308

Davis-Besse Nuclear Power Station Unit #1, 5501 N. St. Rt. 2, Oak Harbor, Ohio 43449

Commercial Service Date: 11/27/77

FORM NR-1 REPORT OF REPAIR MODIFICATION OR REPLACEMENT [X]
TO NUCLEAR COMPONENTS AND SYSTEMS IN NUCLEAR POWER PLANTS

1. Work performed by Framatome Technologies Inc. First Energy P.O.s 7022415 & 7005137
(name of NR certificate holder) (P.O. no., job no., etc.)
3315 Old Forest Road Lynchburg, VA 24501 (address)
2. Owner First Energy Company
76 South Main St. Akron Ohio 44308 (address)
3. Name, address and identification of nuclear power plant Davis-Besse Nuclear Power Station
5501 N. SR 2, Oak Harbor, Ohio 43449
4. System Reactor Pressure Vessel
5. a: Items Which Required Repair, Modification, or Replacement Activities

No	Type of Item	Identification						Construction Code				Activity
		Mfg. Name	Mfg. Serial No.	Nat'l Bd. No.	Jurisd. No.	Other	Year Built	Name/Section/Division	Edition/Addenda	Code Case(s)	Code Class	
1	(2)	(1)	(2)	N/A	N/A	N/A	N/A	(3)	(4)	N/A	1	Repair/Mod/Replace
2												
3												
4												
5												
6												
7												
8												
9												
10												
11												
12												

5. b: Items Installed During Replacement Activities

Type of Item	Installed or Replaced 5a Item No.	Identification						Construction Code			
		Mfg. Name	Mfg. Serial No.	Nat'l Bd. No.	Jurisd. No.	Other	Year Built	Name/Section/Division	Edition/Addenda	Code Case(s)	Code Class
(5)	1	(8)	(5)	N/A	N/A	N/A	N/A	(3)	(4)	None	1
(6)	1	(8)	(6)	N/A	N/A	N/A	N/A	(3)	(4)	None	1

6. ASME Code Section XI applicable for inservice inspection: 1986 No No
(edition) (addenda) (Code Cases(s))
7. ASME Code Section XI used for repairs, modifications, or replacements: 1986 No No
(edition) (addenda) (Code Cases(s))
8. Construction Code used for repairs, modifications, or replacements: 1968 Summer 68 N/A
(edition) (addenda) (Code Cases(s))
9. Design responsibilities: (7)
10. Tests conducted (8) hydrostatic pneumatic design pressure Y pressure _____ psi Code Case(s) _____

First Energy Nuclear Operating, 76 South Main St., Akron, Ohio 44308

Davis-Besse Nuclear Power Station Unit #1, 5501 N. St. Rt. 2, Oak Harbor, Ohio 43449

Commercial Service Date: 11/27/77

11. Description of work:

(use of properly identified additional sheet(s) or sketch(es) is acceptable)

Replaced split rings and hold down bolts on five CRD to RV flange locations C11, D10, F8, F10, & G9.

12. Remarks:

(1) Babcock and Wilcox Company

(2) Replacement hold down bolts and split rings on Control Rod Drive to RV locations C11, D10, F8, F10, & G9

(3) ASME Section III, Division 1

(4) Construction Code ASME III, 1968 Edition, 1968 Summer Addenda

(5) Split Rings S/Ns - D10 / 167, F8 / 185, C11 / 162, F10 / 247, & G9 / 209, Heat # B3687 applicable for all Split Rings

(6) Hold down bolts D10, F10, and G9 - Heat # 20233-3 (Qty-24), C11 - Heat # 20233-2 (Qty-8), and

F8 - (Qty 5) Heat # 20233-2 and (Qty 3) Heat # 20233-3

(7) Framatome Technologies, Inc

(8) By others - First Energy

CERTIFICATE OF COMPLIANCE

I, Victor P. Montalbano Certify that to the best of my knowledge and belief the statements made in this report are correct and the repair, modification or replacement activities described above conforms to Section XI of the ASME Code and The National Board Inspection Code "NR" rules.

National Board Certificate of Authorization No. NR-64 to use the "NR" stamp expires May 17 2000

NR Certificate Holder Framatome Technologies Inc.

Date May 31, 2000 Signed *Victor P. Montalbano* Manager, FTI Quality
(name) (authorized representative) (title)

CERTIFICATE OF INSPECTION

I, Thomas G. Laps holding a valid commission issued by The National Board of Boiler and Pressure Vessel Inspectors and certificate of competency issued by the jurisdiction of Ohio and employed by F.M.I.C.* of Johnston, RI have

inspected the repair, modification or replacement described in this report on April 28, 2000 and state that to the best of my knowledge and belief, this repair, modification or replacement activity has been completed in accordance with Section XI of the ASME Code and the National Board Inspection Code "NR" rules.

By signing this certificate, neither the undersigned nor my employer makes any warranty, expressed or implied, concerning the work described in this report. Furthermore, neither the undersigned nor my employer shall be liable in any manner for any personal injury, property damage or loss of any kind arising from or connected with this inspection.

Date 6/8/00 Signed *Thomas G. Laps* Commissions NB9330 N I A Ohio Comm
(inspector) (National Board (incl. endorsements), jurisdiction, and no.)
*Factory Mutual Insurance Company

12 RFO Inservice Inspection Summary
 First Energy Nuclear Operating, 76 South Main St., Akron, Ohio 44308
 Davis-Besse Nuclear Power Station Unit #1, 5501 N. St. Rt. 2, Oak Harbor, Ohio 43449
 Commercial Service Date: 11/27/77

FORM NR-1 REPORT OF REPAIR X MODIFICATION Y OR REPLACEMENT []
 TO NUCLEAR COMPONENTS AND SYSTEMS IN NUCLEAR POWER PLANTS

1. Work performed by Framatome Technologies Inc. First Energy P.O.s 7022415 & 7005137
 (name of NR certificate holder) (P.O. no., job no., etc.)
3315 Old Forest Road Lynchburg, VA 24501 (address)
2. Owner First Energy Company
76 South Main St. Akron Ohio 44308 (address)
3. Name, address and identification of nuclear power plant Davis-Besse Nuclear Power Station
5501 N. SR 2, Oak Harbor, Ohio 43449
4. System Steam Generator #E24-1
5. a: Items Which Required Repair, Modification, or Replacement Activities

Identification								Construction Code			Activity	
No	Type of Item	Mfg. Name	Mfg. Serial No.	Nat'l Bd. No.	Jurisd. No.	Other	Year Built	Name/Section/Division	Edition/Addenda	Code Case(s)	Code Class	Repair/Mod/Replace
1	(2)	(1)	(2)	158	N/A	N/A	1972	(3)	(4)	(5)	A	Repair
2												
3												
4												
5												
6												
7												
8												
9												
10												
11												
12												

5. b: Items Installed During Replacement Activities

Identification								Construction Code			
Type of Item	Installed or Replaced 5a Item No.	Mfg. Name	Mfg. Serial No.	Nat'l Bd. No.	Jurisd. No.	Other	Year Built	Name/Section/Division	Edition/Addenda	Code Case(s)	Code Class
(6)	1	(8)	(7)	N/A	N/A	N/A	N/A	(3)	(4)	None	1

6. ASME Code Section XI applicable for inservice inspection: 1986 No No
 (edition) (addenda) (Code Cases(s))
7. ASME Code Section XI used for repairs, modifications, or replacements: 1986 No No
 (edition) (addenda) (Code Cases(s))
8. Construction Code used for repairs, modifications, or replacements: 1968 Summer 68 1332-4, 1407-1
 (edition) (addenda) (Code Cases(s))
9. Design responsibilities: (8)
10. Tests conducted: (9) hydrostatic pneumatic design pressure Y pressure psi Code Case(s)

12 RFO Inservice Inspection Summary

First Energy Nuclear Operating, 76 South Main St., Akron, Ohio 44308

Davis-Besse Nuclear Power Station Unit #1, 5501 N. St. Rt. 2, Oak Harbor, Ohio 43449

Commercial Service Date: 11/27/77

11. Description of work: Repaired B Steam Generator tubes by rolled plugging utilizing procedure 1275284-02. Additionally see
(use of properly identified additional sheet(s) or sketch(es) is acceptable)
QCIR 6002827 for acceptance of owner supplied material.

12. Remarks:

- (1) Babcock and Wilcox Company
 (2) Steam Generator #E24-1, S/N 620-0014-55-12
 (3) ASME Section III, Division I
 (4) Construction Code ASME III, 1968 Edition, 1968 Summer Addenda
 (5) Code Cases 1332-4, 1407-1
 (6) Rolled Plug - FTI P/Ns, Stabilizer Rolled Plug - 1196122-002, and Threaded Rolled Plug - 1222118-005
 (7) Rolled Plug Serial Numbers: See QCIR 6002828-00
 (8) Framatome Technologies, Inc.
 (9) By Others-First Energy

CERTIFICATE OF COMPLIANCE

I, Victor P. Montalbano Certify that to the best of my knowledge and belief the statements made in this report are correct and the repair, modification or replacement activities described above conforms to Section XI of the ASME Code and The National Board Inspection Code "NR" rules.
 National Board Certificate of Authorization No. NR-64 to use the "NR" stamp expires May 17 2000
 NR Certificate Holder Framatome Technologies Inc.
 Date May 31, 2000 Signed *Victor P. Montalbano* Manager, FTI Quality
 (name) (authorized representative) (title)

CERTIFICATE OF INSPECTION

I, Thomas G. Laps holding a valid commission issued by The National Board of Boiler and Pressure Vessel Inspectors and certificate of competency issued by the jurisdiction of Ohio and employed by Johnston, RI have F.M.I.C.*
 inspected the repair, modification or replacement described in this report on April 28, 2000 and state that to the best of my knowledge and belief, this repair, modification or replacement activity has been completed in accordance with Section XI of the ASME Code and the National Board Inspection Code "NR" rules.
 By signing this certificate, neither the undersigned nor my employer makes any warranty, expressed or implied, concerning the work described in this report. Furthermore, neither the undersigned nor my employer shall be liable in any manner for any personal injury, property damage or loss of any kind arising from or connected with this inspection.
 Date 6/8/00 Signed *Thomas G. Laps* Commissions NB9330 N I A Ohio Comm
 (inspector) (National Board (incl. endorsements), jurisdiction, and no.)
 *Factory Mutual Insurance Company

First Energy Nuclear Operating, 76 South Main St., Akron, Ohio 44308

Davis-Besse Nuclear Power Station Unit #1, 5501 N. St. Rt. 2, Oak Harbor, Ohio 43449

Commercial Service Date: 11/27/77

FORM NR-1 REPORT OF REPAIR X MODIFICATION Y OR REPLACEMENT []
TO NUCLEAR COMPONENTS AND SYSTEMS IN NUCLEAR POWER PLANTS

1. Work performed by Framatome Technologies Inc. First Energy P.O.s 7022415 & 7005137
(name of NR certificate holder) (P.O. no., job no., etc.)
3315 Old Forest Road Lynchburg, VA 24501
(address)
2. Owner First Energy Company (name)
76 South Main St. Akron Ohio 44308
(address)
3. Name, address and identification of nuclear power plant Davis-Besse Nuclear Power Station
5501 N. SR 2, Oak Harbor, Ohio 43449
4. System Steam Generator #E24-2
5. a: Items Which Required Repair, Modification, or Replacement Activities

[illegible]

5. b: Items Installed During Replacement Activities

[illegible]

- | | | | | |
|-----|--|--------------------------|-------------------------------|--|
| 6. | ASME Code Section XI applicable for inservice inspection: | <u>1986</u>
(edition) | <u>No</u>
(addenda) | <u>No</u>
(Code Cases(s)) |
| 7. | ASME Code Section XI used for repairs, modifications, or replacements: | <u>1986</u>
(edition) | <u>No</u>
(addenda) | <u>No</u>
(Code Cases(s)) |
| 8. | Construction Code used for repairs, modifications, or replacements: | <u>1968</u>
(edition) | <u>Summer 68</u>
(addenda) | <u>1332-4, 1407-1</u>
(Code Cases(s)) |
| 9. | Design responsibilities: | <u>(8)</u> | | |
| 10. | Tests conducted: (9) hydrostatic pneumatic design pressure Y | pressure | <u> </u> psi | Code Case(s) <u> </u> |

12 RFO Inservice Inspection Summary

First Energy Nuclear Operating, 76 South Main St., Akron, Ohio 44308

Davis-Besse Nuclear Power Station Unit #1, 5501 N. St. Rt. 2, Oak Harbor, Ohio 43449

Commercial Service Date: 11/27/77

11. Description of work: Repaired A Steam Generator tubes by rolled plugging utilizing procedure 1275284-02. Additionally see
(use of properly identified additional sheet(s) or sketch(es) is acceptable)
QCIR 6002827 for acceptance of owner supplied material.

12. Remarks:

- (1) Babcock and Wilcox Company
 (2) Steam Generator #E24-2, S/N 620-0014-55-11
 (3) ASME Section III, Division 1
 (4) Construction Code ASME III, 1968 Edition, 1968 Summer Addenda
 (5) Code Cases 1332-4, 1407-1
 (6) Rolled Plug - FTI P/Ns, Stabilizer Rolled Plug - 1196122-002, and Threaded Rolled Plug - 1222118-005
 (7) Rolled Plug Serial Numbers: See QCIR 6002828-00
 (8) Framatome Technologies, Inc.
 (9) By Others-First Energy

CERTIFICATE OF COMPLIANCE

I, Victor P. Montalbano Certify that to the best of my knowledge and belief the statements made in this report are correct and the repair, modification or replacement activities described above conforms to Section XI of the ASME Code and The National Board Inspection Code "NR" rules.
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 (name) (authorized representative) (title)

CERTIFICATE OF INSPECTION

I, Thomas G. Laps holding a valid commission issued by The National Board of Boiler and Pressure Vessel Inspectors and certificate of competency issued by the jurisdiction of Ohio and employed by Johnston, RI have F.M.I.C.*
 inspected the repair, modification or replacement described in this report on April 28, 2000 and state that to the best of my knowledge and belief, this repair, modification or replacement activity has been completed in accordance with Section XI of the ASME Code and the National Board Inspection Code "NR" rules.
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Date 6/8/00 Signed *Thomas G. Laps* Commissions NB9330 N I A Ohio Comm
 (inspector) (National Board (incl. endorsements), jurisdiction, and no.)
 *Factory Mutual Insurance Company