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Nuclear Business Unit

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U. S. Nuclear Regulatory Commission
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Gentlemen:

**MONTHLY OPERATING REPORT
SALEM GENERATING STATION UNIT 2
DOCKET NO. 50-311**

In compliance with Section 6.9, Reporting Requirements for the Salem Unit 2 Technical Specifications, the operating statistics for **July 2000** are being forwarded. Also being forwarded, pursuant to the requirements of 10CFR50.59(b), is a summary of changes, tests, and experiments that were implemented in **July 2000**.

Sincerely,

A handwritten signature in black ink, appearing to read "Mark B. Bezilla".

Mark B. Bezilla
Vice President - Operations

RBK
Attachments

C Distribution

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DOCKET NO.: 50-311
 UNIT: Salem 2
 DATE: 8/15/00
 COMPLETED BY: R. Knieriem
 TELEPHONE: (856) 339-1782

Reporting Period July 2000

OPERATING DATA REPORT

Design Electrical Rating (MWe-Net)
 Maximum Dependable Capacity (MWe-Net)

 No. of hours reactor was critical
 No. of hours generator was on line (service hours)
 Unit reserve shutdown hours
 Net Electrical Energy (MWH)

1115		
1106		
Month	Year-to-date	Cumulative
744	5111	101216
744	5111	97893
0	0	0
791958	5528457	98450675

UNIT SHUTDOWNS

NO.	DATE	TYPE F=FORCED S=SCHEDULED	DURATION (HOURS)	REASON (1)	METHOD OF SHUTTING DOWN THE REACTOR (2)	CORRECTIVE ACTION/ COMMENT

(1) Reason

- A - Equipment Failure (Explain)
- B - Maintenance or Test
- C - Refueling
- D - Regulatory Restriction
- E - Operator Training/License Examination
- F - Administrative
- G - Operational Error (Explain)
- H - Other

(2) Method

- 1 - Manual
- 2 - Manual Trip/Scram
- 3 - Automatic Trip/Scram
- 4 - Continuation
- 5 - Other (Explain)

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Summary Of Monthly Operating Experience

- Salem Unit 2 began the month of July operating at full power.
- Power was reduced to 80% from July 15, until July 17 at the request of the load dispatcher in response to a solar magnetic disturbance.
- Salem Unit 2 operated at full power for the remainder of the month.

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SUMMARY OF CHANGES, TESTS, AND EXPERIMENTS
FOR THE SALEM GENERATING STATION – UNIT 2

MONTH July 2000

The following items completed during **July 2000** have been evaluated to determine:

1. If the probability of occurrence or the consequences of an accident or malfunction of equipment important to safety previously evaluated in the safety analysis report may be increased; or
2. If a possibility for an accident or malfunction of a different type than any evaluated previously in the safety analysis report may be created; or
3. If the margin of safety as defined in the basis for any technical specification is reduced.

The 10CFR50.59 Safety Evaluations showed that these items did not create a new safety hazard to the plant nor did they affect the safe shutdown of the reactor. These items did not change the plant effluent releases and did not alter the existing environmental impact. The 10CFR50.59 Safety Evaluations determined that no unreviewed safety or environmental questions are involved.

Design Changes Summary of Safety Evaluations

2EC-3590, Pkg. 4, Service Water Column Separation Protection 21 Nuclear Supply Header

This design change implemented corrective actions that were developed in response to Generic Letter 96-06 concerns. The design change modified components within the Service Water System 21 Nuclear Header to eliminate the potential for water column separation and flashing in Service Water piping.

Review of this design change under 10CFR50.59 was required because the modifications to the Service Water System 21 Nuclear Header constituted a change to the facility as described in the SAR. This modification was performed to ensure that the Service Water System can perform its design function to provide adequate cooling to the containment in the event of a Loss of Coolant Accident or a Main Steam Line Break

SUMMARY OF CHANGES, TESTS, AND EXPERIMENTS
FOR SALEM GENERATING STATION – UNIT 2 – Cont'd

inside containment. This design change does not increase the possibility or consequences of any accident or malfunction, does not reduce the margin of safety, and does not involve an Unreviewed Safety Question.

Temporary Modifications Summary of Safety Evaluations

There were no reportable changes in this category implemented during July 2000.

Procedures Summary of Safety Evaluations

There were no reportable changes in this category implemented during July 2000.

UFSAR Change Notices Summary of Safety Evaluations

There were no reportable changes in this category implemented during July 2000.

Other - Summary of Safety Evaluations

28VDC Distribution Calculations: Short Circuit and Voltage Drop, Battery and Battery Charger Sizing, and Component Study and Voltage Drop

This safety evaluation supported the revision of calculations related to the 28VDC Distribution system to increase the Loss of Coolant Accident/Loss of Offsite Power (LOCA/LOOP) design margin, to maximize future load growth margin, and to implement minor administrative improvements.

Evaluation of the 28VDC Distribution Calculations was required under 10CFR50.59 because these changes constituted a change to the facility as described in the SAR. The calculations considered in this safety evaluation revised battery load profiles to increase the LOCA/LOOP battery loading design margin from 5% to 7.5%. These revised load profiles will ensure that power is available for LOCA/LOOP loads when required. Therefore, the changes considered in this safety evaluation do not increase the probability or consequences of any accident or malfunction, do not reduce the margin of safety, and do not involve an Unreviewed Safety Question.

80010848, Freeze Seal Installation to Support Repair of Valve 21SW46

This safety evaluation supported the installation of a liquid nitrogen freeze seal to provide temporary isolation for the repair of Service Water System valve 21SW46.

Review of the installation of a freeze seal to provide temporary isolation for the repair of 21SW46 under 10CFR50.59 was required because the change in system configuration

SUMMARY OF CHANGES, TESTS, AND EXPERIMENTS
FOR SALEM GENERATING STATION – UNIT 2 – Cont'd

constituted a change to the facility as described in the SAR. The installation of the freeze seal and the alignment of the Service Water System during the repair were temporary in nature and conducted in accordance with approved procedures. Adequate cooling capability remained to support the required safety-related loads. Therefore, the changes considered in this safety evaluation do not increase the probability or consequences of any accident or malfunction, do not reduce the margin of safety, and do not involve an Unreviewed Safety Question.