



UNITED STATES
NUCLEAR REGULATORY COMMISSION

WASHINGTON, D.C. 20555-0001

July 26, 2000

Mr. M. L. Marchi
Site Vice President-Kewaunee Plant
Wisconsin Public Service Corporation
P.O. Box 19002
Green Bay, WI 54307-9002

SUBJECT: KEWAUNEE NUCLEAR POWER PLANT - ISSUANCE OF CORRECTED
TECHNICAL SPECIFICATION PAGES FOR LICENSE AMENDMENT NO. 148,
REGARDING STEAM GENERATOR SLEEVED TUBES AT KEWAUNEE
(TAC NO. MA3949)

Dear Mr. Marchi:

On June 27, 2000, the U.S. Nuclear Regulatory Commission issued Amendment No. 148 to Facility Operating License No. DPR-43 for the Kewaunee Nuclear Power Plant. This amendment revised the Technical Specifications (TSs) in response to your application dated October 27, 1998, as supplemented on February 23, 2000.

In a letter dated July 25, 2000, you brought to our attention the administrative errors on TS pages TS 4.2-7 and TS 4.2-8 that were issued with Amendment No. 148. The errors involve inadvertent deletion of semicolons on TS 4.2-7, and an inadvertent deletion of the reference to operating cycle 24 in the footer of TS 4.2-8. Therefore, we have corrected the identified errors and the affected pages TS 4.2-7 and TS 4.2-8 are being reissued for Amendment No. 148. We regret any inconveniences this may have caused.

Sincerely,

Tae Kim, Senior Project Manager, Section 1
Project Directorate III
Division of Licensing Project Management
Office of Nuclear Reactor Regulation

Docket No. 50-305

Enclosures: TS 4.2-7 and TS 4.2-8

cc w/encls: See next page

July 26, 2000

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Mr. M. L. Marchi
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Wisconsin Public Service Corporation
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/RA/

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DATE	7/25/00		7/26/00		7/26/00

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cc:

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2. A seismic occurrence greater than the Operating Basis Earthquake, or
 3. A loss-of-coolant accident requiring actuation of the engineering safeguards, where the cooldown rate of the Reactor Coolant System exceeded 100°F/hr, or
 4. A main steam line or feedwater line break, where the cooldown rate of the Reactor Coolant System exceeded 100°F/hr.
- d. If the type of steam generator chemistry treatment is changed significantly, the steam generators shall be inspected at the next outage of sufficient duration following 3 months of power operation since the change.

4. Plugging Limit Criteria

The following criteria apply independently to tube and sleeve wall degradation except as specified in TS 4.2.b.5 for the tube support plate intersections for which voltage-based plugging criteria are applied or for degradation except as specified in TS 4.2.b.6 for tubesheet crevice region in which the F* and EF* criteria is applied.

- a. Any tube which, upon inspection, exhibits tube wall degradation of 50% or more shall be plugged or repaired prior to returning the steam generator to service. If significant general tube thinning occurs, this criterion will be reduced to 40% wall degradation. Tube repair shall be in accordance with the methods described in the following:

WCAP-14685, Revision 4, "Laser Welded Repair of Hybrid Expansion Joint Sleeves for Kewaunee Nuclear Power Plant;"

WCAP-14685, Revision 2, Addendum 1, "Laser Welded Repair of Hybrid Expansion Joint Sleeves for Kewaunee Nuclear Power Plant Addendum 1: Evaluation of Weld Repaired HEJ Sleeved Tubes;"

WCAP-11643, "Kewaunee Steam Generator Sleaving Report (Mechanical Sleeves);"

CEN-629-P Revision 2, "Repair of Westinghouse Series 44 and 51 Steam Generator Tubes Using Leak Tight Sleeves;"

CEN-632-P Revision 0, "Repair of Kewaunee Steam Generator Tubes Using a Resleeving Technique;" or

WCAP-13088, Revision 3, "Westinghouse Series 44 and 51 Steam Generator Generic Sleeving Report" including Addendum 1 to Revision 4.

- b. Any Westinghouse mechanical hybrid expansion joint (HEJ) sleeve which, upon inspection, exhibits wall degradation of 23% or more shall be plugged or repaired prior to returning the steam generator to service. Figure TS 4.2-1 depicts a Westinghouse HEJ sleeve.
- c. For disposition of parent tube indications in the upper joint of Westinghouse HEJ sleeved tubes,* as depicted in Figure TS 4.2-1, the following requirements will apply:
 - 1. HEJ sleeved tubes shall be inspected with a non-destructive examination technique capable of locating the bottom of the hardroll upper transition. HEJ sleeved tubes with circumferential parent tube indications located ≥ 0.92 inch (plus an allowance for NDE uncertainty) below the bottom of the hardroll upper transition, as measured on the inside of the sleeve, may remain in service.
 - 2. HEJ sleeved tubes with circumferential parent tube indications located < 0.92 inch (plus an allowance for NDE uncertainty) from the bottom of the hardroll upper transition, as measured on the inside of the sleeve, shall be plugged or repaired prior to returning the steam generator to service.
 - 3. HEJ sleeved tubes with axial parent tube indications located in the parent tube pressure boundary, as depicted in Figure TS 4.2-1, shall be plugged or repaired prior to returning the steam generator to service.
- d. Any Combustion Engineering leak tight sleeve which, upon inspection, exhibits wall degradation shall be plugged prior to returning the steam generator to service. This plugging limit applies to the sleeve up to and including the weld region.
- e. Any Westinghouse laser welded sleeve which, upon inspection, exhibits wall degradation of 23% or more, shall be plugged prior to returning the steam generator to service. This plugging limit applies to the sleeve up to and including the weld.

*TS 4.2.b.4.c is applicable for operating cycles 23 and 24 only.