

Indian Point 3
Nuclear Power Plant
P.O. Box 215
Buchanan, New York 10511
914 736.8003



Mr. Fred R. Dacimo
Plant Manager

July 13, 2000
IPN-00-054

U.S. Nuclear Regulatory Commission
ATTN: Document Control Desk
Washington, D.C. 20555

Subject: Indian Point 3 Nuclear Power Plant
Docket No. 50-286
License No. DPR-64
Monthly Operating Report for June 2000

Dear Sir:

The attached monthly operating report, for the month of June 2000, is hereby submitted in accordance with Indian Point 3 Nuclear Power Plant Technical Specification 6.9.1.4.

The Authority is making no commitments in this letter.

Very truly yours,

A handwritten signature in black ink, appearing to be 'F. Dacimo'.

Fred R. Dacimo
Plant Manager
Indian Point 3 Nuclear Power Plant

cc: See next page

IE24

Attachments

cc: Mr. Hubert J. Miller
Regional Administrator
Region I
U.S. Nuclear Regulatory Commission
475 Allendale Road
King of Prussia, Pennsylvania 19406-1415

Resident Inspector's Office
Indian Point Unit 3
U.S. Nuclear Regulatory Commission
P.O. Box 337
Buchanan, NY 10511

U.S. Nuclear Regulatory Commission
ATTN: Director, Office of Information Resource Management
Washington, D.C. 20555

INPO Records Center
700 Galleria Parkway
Atlanta, Georgia 30339-5957

OPERATING DATA REPORT

DOCKET NO. 50-286
 UNIT: Indian Point 3
 DATE: 7-7-00
 COMPLETED BY: T. Orlando
 TELEPHONE NO: (914) 736-8340
 LETTER NO: IPN-00-054
 ATTACHMENT
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OPERATING STATUS

1. Unit Name: Indian Point No. 3 Nuclear Power Plant
2. Reporting Period: June 2000
3. Licensed Thermal Power (MWt): 3025
4. Nameplate Rating (Gross MWe): 1013
5. Design Electrical Rating (Net MWe): 965
6. Maximum Dependable Capacity (Gross MWe): 1000
7. Maximum Dependable Capacity (Net MWe): 965
8. If Changes Occur in Capacity Ratings (Items Number 3 through 7) Since Last Report Give Reasons: _____
9. Power Level to Which Restricted, If Any (Net MWe): _____
10. Reasons for Restrictions, If Any: _____

	This Month	Yr-to-Date	Cumulative
11. Hours In Reporting Period	<u>720</u>	<u>4,367</u>	<u>209,064</u>
12. Number Of Hours Reactor Was Critical	<u>686.48</u>	<u>4,333.48</u>	<u>125,061.85</u>
13. Reactor Reserve Shutdown Hours	<u>0</u>	<u>0</u>	<u>0</u>
14. Hours Generator On-Line	<u>635</u>	<u>4,282</u>	<u>122,347.18</u>
15. Unit Reserve Shutdown Hours	<u>0</u>	<u>0</u>	<u>0</u>
16. Gross Thermal Energy Generated (MWH)	<u>1,874,684</u>	<u>12,896,739</u>	<u>349,449,193</u>
17. Gross Electrical Energy Generated (MWH)	<u>628,840</u>	<u>4,347,440</u>	<u>111,301,305</u>
18. Net Electrical Energy Generated (MWH)	<u>606,534</u>	<u>4,210,099</u>	<u>107,441,836</u>
19. Unit Service Factor	<u>88.2</u>	<u>98.1</u>	<u>58.5</u>
20. Unit Availability Factor	<u>88.2</u>	<u>98.1</u>	<u>58.5</u>
21. Unit Capacity factor (Using MDC Net)	<u>87.3</u>	<u>99.9</u>	<u>54.0*</u>
22. Unit Capacity Factor (Using DER Net)	<u>87.3</u>	<u>99.9</u>	<u>53.3</u>
23. Unit Forced Outage Rate	<u>11.8</u>	<u>1.9</u>	<u>25.9</u>

24. Shutdowns Scheduled Over Next 6 Months (Type, Date and Duration of Each): _____
25. If Shut Down At End Of Report Period. Estimated Date of Startup: _____
26. Units In Test Status (Prior to Commercial Operation):

INITIAL CRITICALITY

INITIAL ELECTRICITY

COMMERCIAL OPERATION

Forecast

Achieved

* Weighted Average

AVERAGE DAILY UNIT POWER LEVEL

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MONTH June 2000

DAY	AVERAGE DAILY POWER	DAY	AVERAGE DAILY POWER LEVEL (MWe-Net)
1	<u>984</u>	17	<u>980</u>
2	<u>677</u>	18	<u>981</u>
3	<u>0</u>	19	<u>981</u>
4	<u>3</u>	20	<u>982</u>
5	<u>332</u>	21	<u>981</u>
6	<u>979</u>	22	<u>981</u>
7	<u>981</u>	23	<u>981</u>
8	<u>982</u>	24	<u>981</u>
9	<u>543</u>	25	<u>981</u>
10	<u>176</u>	26	<u>981</u>
11	<u>970</u>	27	<u>983</u>
12	<u>978</u>	28	<u>984</u>
13	<u>980</u>	29	<u>984</u>
14	<u>981</u>	30	<u>983</u>
15	<u>981</u>	31	<u>---</u>
16	<u>980</u>		

INSTRUCTIONS: On this format, list the average daily unit power level in MWe-Net for each day in the reporting month. Compute to the nearest whole megawatt.

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UNIT SHUTDOWNS AND POWER REDUCTIONS

REPORT MONTH June 2000

NO.	DATE	TYPE 1	DURATION (HOURS)	REASON 2	METHOD OF SHUTTING DOWN REACTOR 3	LICENSEE EVENT REPORT #	SYSTEM CODE 4	COMPONENT CODE 5	CAUSE & CORRECTIVE ACTION TO PREVENT RECURRENCE
1	000602	F	34.9	A	NA	NA	XX	PIPEXXC	Unit was removed from service in order to repair a weld leak in a line to the heater drain tank.
2	000604	F	25.08	G	3	2000-007	HB	HTEXCHF	Reactor trip due to low level in No. 31 steam generator as a result of a turbine trip caused by a high level in steam generator No. 33.
3	000609	F	25.02	A	3	2000-008	EA	XXXXXX	Reactor trip as a result of a direct trip from the Buchanan 345 KV Substation. The most probable cause was low insulation resistance between several conductors in a cable for the direct trip circuitry.

1
 F: Forced
 S: Scheduled

2
 Reason:
 A- Equipment
 B- Maintenance or Test
 C- Refueling
 D- Regulatory Restriction
 E- Operator Training & Licensee Examination
 F- Administrative
 G- Operational Error
 H- Other (Explain)

3
 Method:
 1-Manual
 2-Manual Scram
 3-Automatic Scram
 4-Other (Explain)

4
 Exhibit G - Instructions
 for Preparation of Data
 Entry Sheets for Licensee
 Event Report (LER) File
 (NUREG - 0161)

5
 Exhibit 1 -
 Same Source

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SUMMARY OF OPERATING EXPERIENCE

June 2000

The Indian Point Unit No. 3 Nuclear Power Plant was synchronized to the bus for a total of 635 hours, producing a gross generation of 628,840 MWH.

On June 2, at 1500 hours, a load reduction commenced to remove the unit from service in order to repair a line to the heater drain tank, which had developed a crack in a weld. The unit was removed from service at 1758 hours. Following successful repairs, the unit was synchronized to the bus on June 4, at 0452 hours. At 0554, an automatic reactor trip occurred during load escalation after an adjustment was made to increase main boiler feed pump (MBFP) speed that caused increased feedwater flow and level in the No. 33 steam generator (SG). Water level in SG 33 reached the high-high level setpoint initiating a turbine trip. The turbine trip caused a rapid decrease in SG water level when No. 31 SG reached its low-low level setpoint initiating a reactor trip (see LER 2000-007-00).

The reactor was taken critical at 2322 hours and the unit was synchronized to the bus on June 5, at 0659 hours. The unit achieved full power on June 6, at 0316 hours.

On June 9, at 1315 hours, an automatic reactor trip occurred. The reactor trip was initiated by the electrical protective relaying direct trip circuitry from the offsite 345 kv Buchanan Substation. Investigation revealed the most probable cause of the direct trip circuitry actuation was low insulation resistance between several conductors in a cable routed between Indian Point 3 and the Buchanan substation 345 kv south ring bus control house (see LER 2000-008-00).

Following successful repairs, the reactor was taken critical on June 10, at 0518 hours, and the unit was synchronized to the bus at 1417 hours. The unit achieved full power on June 11, at 0205 hours.

The unit remained on line at full load for the remainder of the reporting period.