

July 21, 2000

Mr. Oliver D. Kingsley
President, Nuclear Generation Group
Commonwealth Edison Company
ATTN: Regulatory Services
Executive Towers West III
1400 Opus Place, Suite 500
Downers Grove, IL 60515

SUBJECT: QUAD CITIES- NRC INSPECTION REPORT 50-254/2000009(DRS);
50-265/2000009(DRS)

Dear Mr. Kingsley:

On July 14, 2000, the NRC completed a baseline inspection at your Quad Cities Nuclear Power Station, Units 1 and 2. The results of this inspection were discussed on July 14, 2000, with Mr. J. Dimmette and other members of your staff. The enclosed report presents the results of that inspection.

The inspection was an examination of activities conducted under your license as they relate to emergency preparedness and to compliance with the Commission's rules and regulations and with the conditions of your license. Within these areas the inspection consisted of a selective examination of procedures and representative records, observations of activities, and interviews with personnel. Specifically, this inspection focused on performance during your biennial emergency preparedness exercise and your staff's capability to self-assess your participants' performance.

Based on the results of this inspection, no findings were identified.

In accordance with 10 CFR 2.790 of the NRC's "Rules of Practice," a copy of this letter and its enclosure will be available **electronically** for public inspection in the NRC Public Document Room **or** from the *Publicly Available Records (PARS) component of NRC's document system (ADAMS)*. ADAMS is accessible from the NRC Web site at <http://www.nrc.gov/NRC/ADAMS/index.html> (the Public Electronic Reading Room).

We will gladly discuss any question you have concerning this inspection.

Sincerely,

/RA by Wayne Slawinski Acting for/

Gary L. Shear, Chief
Plant Support Branch
Division of Reactor Safety

Docket Nos. 50-254; 50-265
License Nos. DPR-29; DPR-30

Enclosure: Inspection Report 50-254/2000009(DRS);
50-265/2000009(DRS)

cc w/encl: D. Helwig, Senior Vice President, Nuclear Services
C. Crane, Senior Vice President, Nuclear Operations
H. Stanley, Vice President, Nuclear Operations
R. Krich, Vice President, Regulatory Services
DCD - Licensing
J. Dimmette, Jr., Site Vice President
G. Barnes, Quad Cities Station Manager
C. Peterson, Regulatory Affairs Manager
M. Aguilar, Assistant Attorney General
State Liaison Officer, State of Illinois
State Liaison Officer, State of Iowa
Chairman, Illinois Commerce Commission
W. Leech, Manager of Nuclear
MidAmerican Energy Company
W. Curtis, FEMA, Region V
E. Jenkins, FEMA, Region VII

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 M. Aguilar, Assistant Attorney General
 State Liaison Officer, State of Illinois
 State Liaison Officer, State of Iowa
 Chairman, Illinois Commerce Commission
 W. Leech, Manager of Nuclear
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U.S. NUCLEAR REGULATORY COMMISSION

REGION III

Docket Nos: 50-254; 50-265
License Nos: DPR-29; DPR-30

Report No: 50-254/2000009(DRS); 50-265/2000009(DRS)

Licensee: Commonwealth Edison Company

Facility: Quad Cities Nuclear Power Station, Units 1 and 2

Location: 22710 206th Avenue North
Cordova, IL 61242

Dates: July 11-14, 2000

Inspectors: T. Ploski, Senior Emergency Preparedness Analyst
C. Miller, Senior Resident Inspector
D. Funk, Emergency Preparedness Analyst
J. Foster, Emergency Response Coordinator

Approved by: Gary L. Shear, Chief, Plant Support Branch
Division of Reactor Safety

NRC's REVISED REACTOR OVERSIGHT PROCESS

The federal Nuclear Regulatory Commission (NRC) recently revamped its inspection, assessment, and enforcement programs for commercial nuclear power plants. The new process takes into account improvements in the performance of the nuclear industry over the past 25 years and improved approaches of inspecting and assessing safety performance at NRC licensed plants.

The new process monitors licensee performance in three broad areas (called strategic performance areas): reactor safety (avoiding accidents and reducing the consequences of accidents if they occur), radiation safety (protecting plant employees and the public during routine operations), and safeguards (protecting the plant against sabotage or other security threats). The process focuses on licensee performance within each of seven cornerstones of safety in the three areas:

Reactor Safety	Radiation Safety	Safeguards
<ul style="list-style-type: none">● Initiating Events● Mitigating Systems● Barrier Integrity● Emergency Preparedness	<ul style="list-style-type: none">● Occupational● Public	<ul style="list-style-type: none">● Physical Protection

To monitor these seven cornerstones of safety, the NRC uses two processes that generate information about the safety significance of plant operations: inspections and performance indicators. Inspection findings will be evaluated according to their potential significance for safety, using the Significance Determination Process, and assigned colors of GREEN, WHITE, YELLOW or RED. GREEN findings are indicative of issues that, while they may not be desirable, represent very low safety significance. WHITE findings indicate issues that are of low to moderate safety significance. YELLOW findings are issues that are of substantial safety significance. RED findings represent issues that are of high safety significance with a significant reduction in safety margin.

Performance indicator data will be compared to established criteria for measuring licensee performance in terms of potential safety. Based on prescribed thresholds, the indicators will be classified by color representing varying levels of performance and incremental degradation in safety: GREEN, WHITE, YELLOW, and RED. GREEN indicators represent performance at a level requiring no additional NRC oversight beyond the baseline inspections. WHITE corresponds to performance that may result in increased NRC oversight. YELLOW represents performance that minimally reduces safety margin and requires even more NRC oversight. And RED indicates performance that represents a significant reduction in safety margin but still provides adequate protection to public health and safety.

The assessment process integrates performance indicators and inspection so the agency can reach objective conclusions regarding overall plant performance. The agency will use an Action Matrix to determine in a systematic, predictable manner which regulatory actions should be taken based on a licensee's performance. The NRC's actions in response to the significance (as represented by the color) of issues will be the same for performance indicators as for inspection findings. As a licensee's safety performance degrades, the NRC will take more and increasingly significant action, which can include shutting down a plant, as described in the Action Matrix.

More information can be found at: <http://www.nrc.gov/NRR/OVERSIGHT/index.html>.

SUMMARY OF FINDINGS

Quad Cities Nuclear Power Station, Units 1 and 2
NRC Inspection Report 50-254/2000009(DRS); 50-265/2000009(DRS)

The report covers a one week period of announced inspection by regional emergency preparedness inspectors and a resident inspector. This inspection focused on the Reactor Safety, Emergency Preparedness Cornerstone, and included the following: evaluation of licensee staff's capability to assess licensee participants' performance during the biennial emergency preparedness exercise and review of procedural guidance for identifying key emergency response positions related to the NRC's performance indicator.

REACTOR SAFETY

Cornerstone: Emergency Preparedness

- There were no findings identified.

Report Details

1. REACTOR SAFETY

Cornerstone: Emergency Preparedness (EP)

1EP1 Drill, Exercise, and Actual Events

a. Inspection Scope

The inspectors reviewed the 2000 exercise's objectives and scenario to ensure that the exercise would acceptably test major elements of the licensee's emergency plan. The scenario included multiple equipment failures and a radiological release. The inspectors verified that these simulated problems provided an acceptable framework to support demonstration of the licensee's, States', and counties' capabilities to implement their emergency plans.

The inspectors evaluated the licensee's exercise performance, focusing on the risk-significant activities of emergency classification, notification, and protective action recommendations, as well as implementation of accident mitigation strategies in the following emergency response facilities:

- Simulator Control Room (SCR)
- Technical Support Center (TSC)
- Operational Support Center (OSC)
- Emergency Operations Facility (EOF)

The inspectors also assessed the licensee's recognition of abnormal plant conditions, transfer of responsibilities between facilities, internal communications, interface with State officials, readiness of emergency facilities and related equipment, and overall implementation of the Generating Stations Emergency Plan (GSEP).

Inspectors attended post-exercise critiques in the SCR, TSC, OSC, and EOF to evaluate the licensee's initial assessment of its exercise performance. The inspectors later met with the licensee's lead exercise evaluators to discuss the licensee's refined assessments of its exercise participants' performances. These self-assessments were then compared with the inspectors' independent observations and related assessments to verify that all performance concerns were identified by the licensee. On July 13 and 14, 2000, an inspector represented the NRC at the public and media briefings hosted by Federal Emergency Management Agency (FEMA) Region V and VII staffs, respectively.

b. Findings

There were no findings identified.

4. **OTHER ACTIVITIES**

40A5 Temporary Instruction 2515/144

a. Inspection Scope

The inspectors reviewed and discussed the licensee's procedural guidance related to the identification of key members of the Emergency Response Organization (ERO) for Performance Indicator (PI) tracking and reporting purposes. The procedural guidance was compared to relevant industry guidance in Nuclear Energy Institute publication number 99-02, Revision 0.

b. Findings

There were no findings identified.

40A6 Management Meetings

.1 Exit Meeting Summary

The inspectors presented the inspection results to Mr. J. Dimmette and other members of licensee management and staff at the conclusion of the inspection on July 14, 2000. The licensee acknowledged the information presented and did not identify any information discussed as proprietary.

PARTIAL LIST OF PERSONS CONTACTED

Licensee

E. Anderson, Radiation Protection Manager
J. Anderson, Emergency Planning Coordinator
R. Armitage, Acting Training Manager
P. Behrens, Chemistry Manager
R. Chrzanowski, Nuclear Oversight Manager
J. Dimmette, Site Vice President
T. Fuhs, Regulatory Assurance Staff
J. Kudalis, Support Services Director
M. McDowell, Operations Manager
C. Peterson, Regulatory Assurance Manager
J. Siper, Director of Licensing and Compliance
J. Sirovy, Nuclear Oversight Staff
R. Svalesson, Shift Operations Supervisor
F. Tague, Assistant Emergency Planning Coordinator
M. Vonk, Emergency Planning Manager - Nuclear Generation Group
M. Warren, Operations Staff

ITEMS OPENED, CLOSED, AND DISCUSSED

Opened

None

Closed

None

Discussed

None

LIST OF ACRONYMS USED

CEPIP	Corporate Emergency Plan Implementing Procedure
CFR	Code of Federal Regulations
DRS	Division of Reactor Safety
EOF	Emergency Operations Facility
EP	Emergency Preparedness
ERO	Emergency Response Organization
FEMA	Federal Emergency Management Agency
GSEP	Generating Stations Emergency Plan
NRC	Nuclear Regulatory Commission
OSC	Operational Support Center
PI	Performance Indicator
SCR	Simulator Control Room
TI	Temporary Instruction
TSC	Technical Support Center

INSPECTION PROCEDURES USED

71114	Reactor Safety-Emergency Preparedness
71114.01	Exercise Evaluation
TI 2515/144	Performance Indicator Data Collecting and Reporting Process Review

LIST OF DOCUMENTS REVIEWED

Miscellaneous

Records of the May 31 practice drill
Scenario manual for the July 12 exercise
“EP Department Training and Reference Material-ERO Drill Participation NRC PI-09 (S.19) Guidance,” Revision 5
Draft “Corrective Actions Listing” for the July 12 exercise
“PI Data Summary” for the July 12 exercise

Problem Identification Forms

Q2000-02527; Q2000-02548

Procedures

QEP 0100-01, Revision 9, “Acting Station Director Implementing Procedure”
QEP 0105-01, Revision 7, “Station Director Implementing Procedure”
QEP 0107-01, Revision 7, “Assistant Station Director Implementing Procedure”
QEP 0107-01, Revision 1, “Communicators and Status Board Recorders”
QEP 0110-01, Revision 19, “Operations Director Implementing Procedure”
QEP 0120-01, Revision 14, “Technical Director Implementing Procedure”
QEP 0125-01, Revision 5, “Administrative Director Implementing Procedure”
QEP 0130-01, Revision 11, “Maintenance Director Implementing Procedure”
QEP 0135-01, Revision 2, “Stores Director Implementing Procedure”
QEP 0140-01, Revision 10, “Security Director Implementing Procedure”
QEP 0150-01, Revision 10, “Radiation Protection Director Implementing Procedure”
QEP 0155-01, Revision 7, “Onsite Environs Director Implementing Procedure”
QEP 0160-01, Revision 11, “OSC Director Implementing Procedure”
QEP 0165-01, Revision 10, “OSC Supervisor Implementing Procedure”
QEP 0175-01, Revision 6, “Chemistry Director Implementing Procedure”
QEP 0200-01, Revision 13, “Classification of GSEP Condition”
QEP 0300-01, Revision 24, “Notification for GSEP Emergencies”
QEP 0400-01, Revision 13, “Plant Assembly”
QEP 0400-02, Revision 4, “Site Evacuation”
QEP 0400-03, Revision 7, “Emergency Teams”
QEP 0500-01, Revision 6, “Recovery Operations”
QEP 0600-01, Revision 6, “Onsite Emergency Facilities”
CEPIP 2021-01, Revision 3, “EOF Activation”
CEPIP 2200-01, Revision 8, “Manager of Emergency Operations”
CEPIP 2210-01, Revision 5, “Technical Support Manager”
CEPIP 2220-01, Revision 10, “Protective Measures Director”
CEPIP 2231-01, Revision 4, “Emergency Planner”